

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

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ACCESSION NBR: 9207230290 DOC. DATE: 92/07/17 NOTARIZED: NO DOCKET #
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
 AUTH. NAME AUTHOR AFFILIATION
 WEBER, G.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 BLIND, A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 92-007-00: on 920705, containment type b & c leakage exceeds L.C.O. value. Final rept will be issued 921020. W/920717 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 2
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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INTERNAL:	ACNW	2	2		AEOD/DOA	1	1		
	AEOD/DSP/TPAB	1	1		AEOD/ROAB/DSP	2	2		
	NRR/DET/EMEB 7E	1	1		NRR/DLPQ/LHFB10	1	1		
	NRR/DLPQ/LPEB10	1	1		NRR/DOEA/OEAB	1	1		
	NRR/DREP/PRPB11	2	2		NRR/DST/SELB 8D	1	1		
	NRR/DST/SICB8H3	1	1		NRR/DST/SPLB8D1	1	1		
	NRR/DST/SRXB 8E	1	1		<u>REG FILE</u> 02	1	1		
	RES/DSIR/EIB	1	1		RGN3 FILE 01	1	1		
EXTERNAL:	EG&G BRYCE, J.H	3	3		L ST LOBBY WARD	1	1		
	NRC PDR	1	1		NSIC MURPHY, G.A	1	1		
	NSIC POORE, W.	1	1		NUDOCS FULL TXT	1	1		

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616 465 5901



INDIANA
MICHIGAN
POWER

July 17, 1992

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by
10 CFR 50.73 entitled Licensee Event Report System, the
following report is being submitted:

92-007-00

Sincerely,

A. A. Blind
A. A. Blind
Plant Manager

/sb

Attachment

c: D. H. Williams, Jr.
A. B. Davis, Region III
E. E. Fitzpatrick
P. A. Barrett
R. F. Kroeger
B. Walters - Ft. Wayne
NRC Resident Inspector
J. F. Stang - NRC
J. G. Keppler
M. R. Padgett
G. Charnoff, Esq.
D. Hahn
INPO
S. J. Brewer/B. P. Lauzau
B. A. Svensson

9207230290 920717
PDR ADDCK 05000315
S PDR

Handwritten signature

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) D. C. COOK NUCLEAR PLANT - UNIT 1						DOCKET NUMBER (2) 0 5 0 0 0 3 1 5			PAGE (3) 1 OF 0 1		
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TITLE (4)
CONTAINMENT TYPE B AND C LEAKAGE EXCEEDS L.C.O. VALUE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)										
0	7	0	5	9	2	9	2	0	0	7	0	0	0	0	0	0	0	0	0	0	0

OPERATING MODE (9) 6

POWER LEVEL (10) 0 | 0 | 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME G. A. WEBER - PLANT ENGINEERING SUPERINTENDENT		TELEPHONE NUMBER AREA CODE 6 1 6 4 6 5 - 5 9 0 1	
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED SUBMISSION DATE (15)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

1 | 0 | 2 | 0 | 9 | 2

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 05, 1992, with the Reactor Coolant System in Mode 6 (Refueling), the accumulated leakage found while performing the Type B and C Leak Rate Tests on Containment penetrations exceeded the L.C.O. value (0.60 La) of Technical Specification 3.6.1.2.b.

Containment Isolation Valves 1-WCR-951 and 1-WCR-955 on the Non-Essential Service Water Supply and Return Lines for the No. 11 Reactor Coolant Pump Air Cooler Vent, has been the major contributing factor for exceeding the Technical Specification limit. Valves 1-WCR-951 and 955 exhibited a leakage rate of 67,000 sccm. This is 93 percent of the total leakage obtained from all tested penetrations. Thirty-seven percent of the Type B and C Leak Rate Testing has been completed.

Those Containment Isolation Valves that exhibit leak rates in excess of the acceptance criteria are being repaired and retested to ensure the leak rates are within allowable limits. The B and C Leak Rate Testing is still in progress and will not be completed until the end of the Refueling Outage.

This is an interim report. The final report will be submitted by October 20, 1992, following completion of the B and C Leak Rate Tests.