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 RECIP. NAME                      RECIPIENT AFFILIATION  
 MURLEY, T.E.                      Document Control Branch (Document Control Desk)

SUBJECT: Suppls 880901 telcon re integrated startup testing program  
 including, testing re vibration measurements of RCS, natural  
 circulation of RCS & steam generator level control  
 instrumentation.

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American Electric Power  
Service Corporation  
1 Riverside Plaza  
Columbus, OH 43215  
614 223 1000



DATE: August 13, 1991

TIME SENT: \_\_\_\_\_

TO: John Stang

NUMBER OF PAGES INCLUDING

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FROM: Leanne Holter

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Document Control Desk,

This letter never got recorded in NUDOC's. Please create a NUDOCs record w/ fiche. [No RTD's distribution is needed]. Include PDR for availability. (This letter has been re-quested under FOIA) -

W. Long  
8/14/91

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Indiana Michigan  
Power Company  
P.O. Box 16631  
Columbus, OH 43216



AEP:NRG:09800

Donald C. Cook Nuclear Plant Unit 2  
Docket No. 50-316  
License No. DPR-74  
INTEGRATED STARTUP TESTING PROGRAM

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

Attn: T. E. Murley

September 7, 1988

Dear Dr. Murley:

This letter is being sent as a follow-up to a September 1, 1988 telephone conversation between American Electric Power Service Corporation and Nuclear Regulatory Commission personnel.

The phone call was made to respond to three questions regarding the Unit 2 Integrated Startup Testing Program. The three areas questioned concerned testing relative to vibration measurements of the reactor coolant system, natural circulation of the reactor coolant system, and steam generator level control instrumentation. The attachment to this letter provides a discussion of each of the items listed above.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Sincerely,

M. P. Alexich  
Vice President

MPA/eh

Attachment

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Dr. T. E. Murley }

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AEP:NRC:09800

cc: D. H. Williams, Jr.  
W. G. Smith, Jr. - Bridgman  
R. C. Callen  
G. Charnoff  
G. Bruchmann  
NRC Resident Inspector - Bridgman  
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Attachment 1 to AEP:NRG:09800

Page 1

QUESTION 1

What procedure(s) will be used to conduct vibration measurements of the Unit 2 reactor coolant system components?

RESPONSE

Reactor internals vibration measurements are deemed unnecessary since no changes were made to the internals package. Changes in replacement steam generators have slightly reduced the reactor coolant system resistance, with only a negligible impact on system flow velocities.

Reactor coolant system vibration measurements are also deemed unnecessary. The replacement steam generators offer only a slight increase in total system mass and are expected to have negligible impact on system vibration response.

During start-up and operation the reactor coolant system is routinely monitored by operator observations and monitoring instrumentation. These same activities are deemed sufficient to notice any unusual system vibration which might occur as a result of Steam Generator Repair Project activities.

QUESTION 2

What procedure(s) will be used to conduct natural circulation tests of the reactor coolant system to confirm that the design heat removal capability exists or to verify that flow (without pumps) or temperature data are comparable to prototype designs for which equivalent tests have been successfully completed?

RESPONSE

Ramp down testing was conducted during initial start-up, and the overall reactor coolant system flow resistance was compared to that of our prototype plant, Trojan Nuclear Plant. System resistance was lower than Trojan's and therefore a natural circulation test was not performed.

The replacement steam generators have a lower pressure drop than the original steam generators. Specifically the tube ends are flush with the tube sheet surface, rounded and smooth which result in lower inlet pressure losses. In addition the replacement steam generators have an increased number of tubes. This will reduce the system resistance to a value less than during initial start-up. Therefore, no natural circulation test is required.



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UNITED STATES DEPARTMENT OF JUSTICE

Washington, D.C.

TO THE HONORABLE ATTORNEY GENERAL

Dear Sir:

I have the honor to acknowledge the receipt of your letter of the 10th instant, in relation to the above-captioned matter, and in reply to inform you that the same has been forwarded to the proper authorities for their consideration.

I am, Sir, very respectfully,  
Your obedient servant,  
[Signature]

I am, Sir, very respectfully,  
Your obedient servant,  
[Signature]

Very truly yours,

I am, Sir, very respectfully,  
Your obedient servant,  
[Signature]

Very truly yours,

I am, Sir, very respectfully,  
Your obedient servant,  
[Signature]

I am, Sir, very respectfully,  
Your obedient servant,  
[Signature]

Attachment 1 to AEP:NRC:09800

Page 2

Discussions have been held with Westinghouse regarding the licensee's response to Questions 1 and 2 and Westinghouse concurs with both responses.

QUESTION 3

What procedure(s) will be used to calibrate and verify the performance of the steam generator level control system?

RESPONSE

Steam generator level control will be calibrated, using Plant Procedures THP 6030 IMP.204, .205, .206, and .207. The individual inputs to the steam generator level control/feedflow (steam flow, steam pressure, steam generator level) will be calibrated using appropriate plant procedures. No additional testing or calibration is planned as a result of the Steam Generator Repair Project.

