

RECOMMENDED TECHNICAL SPECIFICATIONS MARK UPS

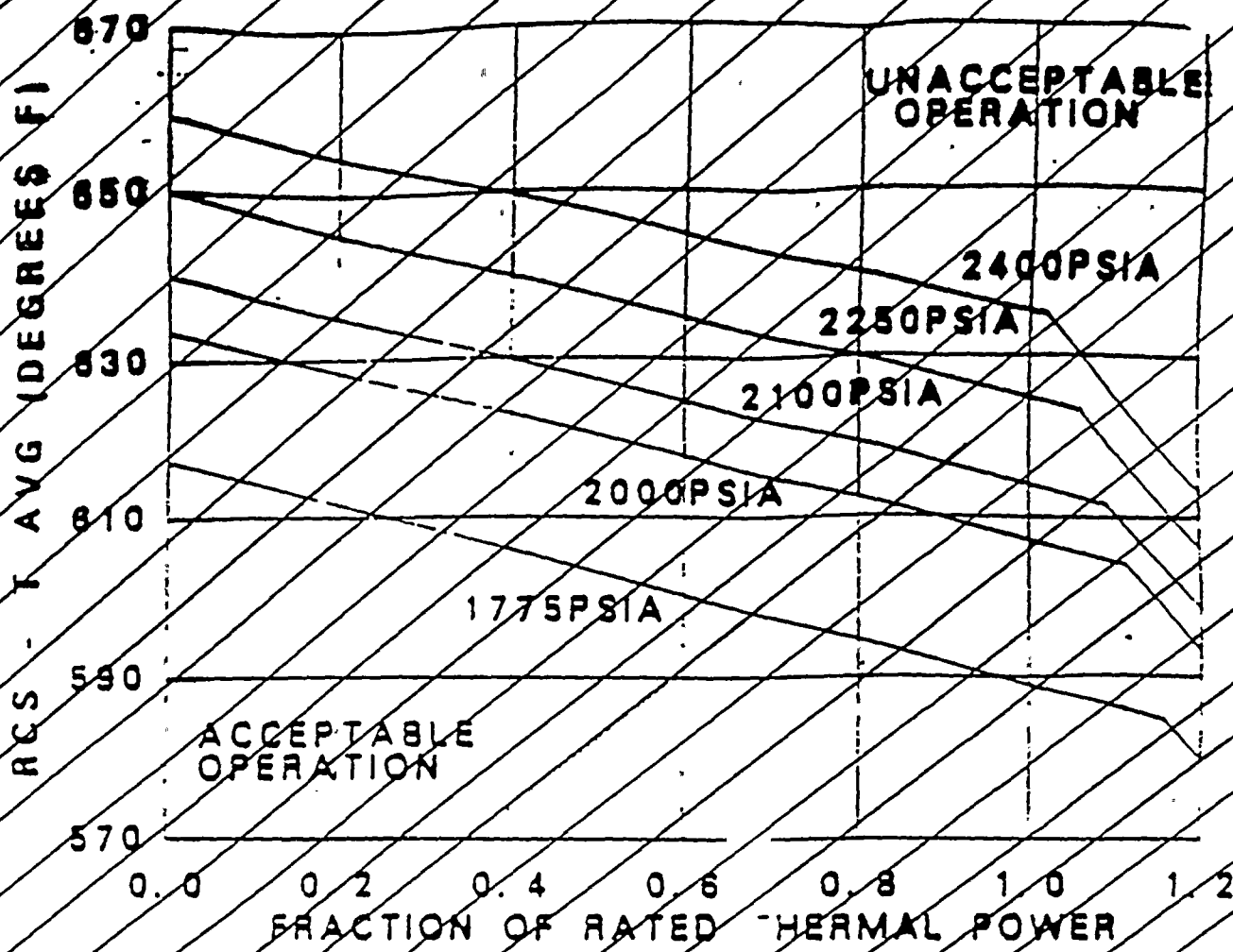
Attachment 1 to AEP-90-231

NS-OPLS-OPL-11-90-463

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Replace with supplied figure



| PRESSURE (PSIA) | BREAKPOINTS FRACTION RATED THERMAL POWER, T AVG IN DEGREES F |
|-----------------|-----------------------------------------------------------------|
| 1775 | (0.0, 617.1), (1.18, 584.5), (1.20, 579.7) |
| 2000 | (0.0, 632.5), (1.11, 603.9), (1.20, 592.1) |
| 2100 | (0.0, 640.3), (1.09, 611.5), (1.20, 598.3) |
| 2250 | (0.0, 650.0), (1.05, 623.2), (1.20, 606.8) |
| 2400 | (0.0, 659.0), (1.02, 634.8), (1.20, 613.0) |

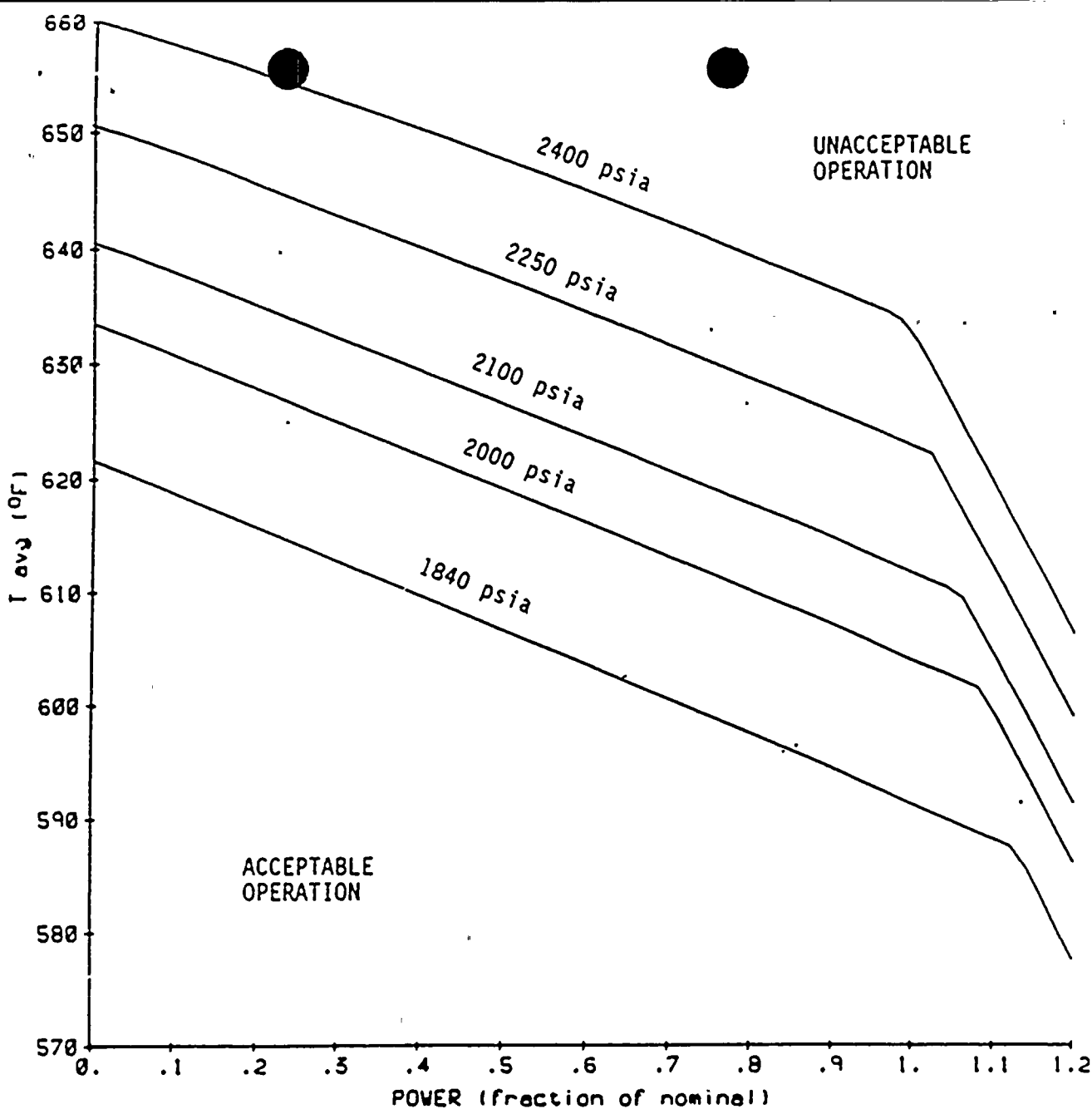
Figure 2.1-1 Reactor Core Safety Limits
Four Loops In Operation

D. C. COOK - UNIT 1

2 - 2

Amendment No. 7A, 125





| PRESSURE (PSIA) | BREAKPOINTS (FRACTION RATED THERMAL POWER, T-AVG IN DEGREES F) |
|--------------------|-------------------------------------------------------------------|
| 1840 | (0.0, 622.1), (1.13, 587.3), (1.20, 577.5) |
| 2000 | (0.0, 633.8), (1.08, 601.4), (1.20, 586.0) |
| 2100 | (0.0, 640.8), (1.06, 609.8), (1.20, 591.3) |
| 2250 | (0.0, 650.7), (1.02, 621.9), (1.20, 598.9) |
| 2400 | (0.0, 660.1), (0.98, 633.7), (1.20, 606.2) |

RATED THERMAL POWER = 3413 MWT

FIGURE 2.1-1 REACTOR CORE SAFETY LIMITS

O. C. COOK - UNIT 1

2-5

Amendment No. 97, 126

TABLE 2.2-1

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

| <u>FUNCTIONAL UNIT</u> | <u>TRIP SETPOINT</u> | <u>ALLOWABLE VALUES</u> |
|--------------------------------------------------|----------------------------------------------------------------------------------------------------------|----------------------------------------------------------------------------------------------------------|
| 1. Manual Reactor Trip | Not Applicable | Not Applicable |
| 2. Power Range, Neutron Flux | Low Setpoint - $\leq 25\%$ of RATED THERMAL POWER High Setpoint - $\leq 109\%$ of RATED THERMAL POWER | Low Setpoint - $\leq 26\%$ of RATED THERMAL POWER High Setpoint - $\leq 110\%$ of RATED THERMAL POWER |
| 3. Power Range, Neutron Flux, High Positive Rate | $\leq 5\%$ of RATED THERMAL POWER with a time constant ≥ 2 seconds | $\leq 5.5\%$ of RATED THERMAL POWER with a time constant ≥ 2 seconds |
| 4. Power Range, Neutron Flux, High Negative Rate | $\leq 5\%$ of RATED THERMAL POWER with a time constant ≥ 2 seconds | $\leq 5.5\%$ of RATED THERMAL POWER with a time constant ≥ 2 seconds |
| 5. Intermediate Range, Neutron Flux | $\leq 25\%$ of RATED THERMAL POWER | $\leq 30\%$ of RATED THERMAL POWER |
| 6. Source Range, Neutron Flux | $\leq 10^4$ counts per second | $\leq 1.3 \times 10^5$ counts per second |
| 7. Overtemperature ΔT | See Note 1 | See Note 3 |
| 8. Overpower ΔT | See Note 2 | See Note 4 |
| 9. Pressurizer Pressure--Low | ≥ 1875 psig | ≥ 1865 psig |
| 10. Pressurizer Pressure--High | ≤ 2385 psig | ≤ 2395 psig |
| 11. Pressurizer Water Level--High | $\leq 92\%$ of Instrument span | $\leq 93\%$ of Instrument span |
| 12. Loss of Flow | $\geq 90\%$ of design flow per loop* | $\geq 89.1\%$ of design flow per loop* |

*Design flow is 90,400 ~~91,600~~ gpm per loop.

TABLE 3.2-1
DNB PARAMETERS

LIMITS

| <u>PARAMETER</u> | <u>4 Loops in Operation at RATED THERMAL POWER</u> |
|-------------------------------------------|---------------------------------------------------------------------------------------------------------------------|
| Reactor Coolant System Tavg | ≤ 570.9°F* |
| Pressurizer Pressure | ≥ 2050 psig** |
| Reactor Coolant System Total Flow Rate | ≥ 361,000 266,400 gpm*** |

Indicated average of at least three OPERABLE instrument loops.

** Limit not applicable during either a THERMAL POWER ramp increase in excess of 5 percent RATED THERMAL POWER per minute or a THERMAL POWER step increase in excess of 10 percent RATED THERMAL POWER.

*** Indicated value.

COOK NUCLEAR PLANT UNIT 1

3/4 2-14

AMENDMENT NO. 97,720, 126



WESTINGHOUSE NUCLEAR SAFETY
SAFETY EVALUATION CHECK LIST

- 1.) NUCLEAR PLANT(S): Donald C. Cook Nuclear Plant Unit 1
- 2.) SUBJECT (TITLE): Reduced Minimum Measured Flow
- 3.) The written safety evaluation of the revised procedure, design change or modification required by 10CFR50.59 (b) has been prepared to the extent required and is attached. If a safety evaluation is not required or is incomplete for any reason, explain on Page 2.

Parts A and B of this Safety Evaluation Check List are to be completed only on the basis of the safety evaluation performed.

CHECK LIST - PART A - 10CFR50.59(a) (1)

- 3.1) Yes No A change to the plant as described in the FSAR?
- 3.2) Yes No A change to procedures as described in the FSAR?
- 3.3) Yes No A test or experiment not described in the FSAR?
- 3.4) Yes No A change to the plant technical specifications?
(See Note on Page 2.)

4) CHECK LIST - PART B - 10CFR50.59(a) (2) (Justification for Part B answers must be included on Page 2.)

- 4.1) Yes No Will the probability of an accident previously evaluated in the FSAR be increased?
- 4.2) Yes No Will the consequences of an accident previously evaluated in the FSAR be increased?
- 4.3) Yes No May the possibility of an accident which is different than any already evaluated in the FSAR be created?
- 4.4) Yes No Will the probability of a malfunction of equipment important to safety previously evaluated in the FSAR be increased?
- 4.5) Yes No Will the consequences of a malfunction of equipment important to safety previously evaluated in the FSAR be increased?
- 4.6) Yes No May the possibility of a malfunction of equipment important to safety different than any already evaluated in the FSAR be created?
- 4.7) Yes No Will the margin of safety as described in the bases to any technical specification be reduced?

NOTES:

If the answers to any of the above questions are unknown, indicated under 5.) REMARKS and explain below.

If the answer to any of the above questions in Part A (3.4) or Part B cannot be answered in the negative, based on written safety evaluation, the change would require an application for license amendment as required by 10CFR50.59(c) and submitted to the NRC pursuant to 10CFR50.90

5.) REMARKS:

The following summarizes the justification upon the written safety evaluation (1) for answers given in Part A (3.4) and Part B of this SECL

See Safety Evaluation

(1)Reference to document(s) containing written safety evaluation:

FOR FSAR UPDATE

Section: * Page(s): * Table(s): * Figure(s): *

Reason for/Description of Change:

* See Safety Evaluation

SAFETY EVALUATION APPROVAL LADDER:

Prepared by (Nuclear Safety): D. H. Behnke Date: 6/25/90
Coordinated with Engineer(s): G. H. Heberle Date: 6/25/90
Coordinating Group Manager(s): _____ Date: _____
Nuclear Safety Group Manager: S. D. Rupprecht Date: 6/25/90

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