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ACCESSION NBR: 9103280137 DOC. DATE: 91/03/22 NOTARIZED: NO DOCKET #
 FACIL: 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316
 AUTH. NAME AUTHOR AFFILIATION
 CARTEAUX, P.F. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 BLIND, A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 91-002-00: on 910220, discovered that previous procedure rev omitted fire damper from insp list & damper not included in surveillance. Cause not conclusively determined. Damper visually & functionally inspected. W/910322 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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	NRR/DLPQ/LHFB11	1 1	NRR/DLPQ/LPEB10	1 1
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EXTERNAL:	EG&G BRYCE, J.H	3 3	L ST LOBBY WARD	1 1
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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616 465 5901



**INDIANA
MICHIGAN
POWER**

March 22, 1991

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-74
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by
10 CFR 50.73 entitled Licensee Event Reporting System,
the following report is being submitted:

91-002-00

Sincerely,

A.A. Blind
Plant Manager

AAB:sb

Attachment

c: D.H. Williams, Jr.
A.B. Davis, Region III
M.P. Alexich
P.A. Barrett
B.F. Henderson
R.F. Kroeger
B. Walters - Ft. Wayne
NRC Resident Inspector
T. Colburn - NRC
J.G. Keppler
M.R. Padgett
G. Charnoff, Esq.
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D. Hahn
INPO
S.J. Brewer/B.P. Lauzau
B.A. Svensson

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PDR ADLCK 05000314
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) D. C. Cook Nuclear Plant, Unit 2		DOCKET NUMBER (2) 0 5 0 0 0 3 1 6	PAGE (3) 1 OF 0 3
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TITLE (4) Missed Surveillance of a Single Fire Damper Due to Inadvertent Omission of Damper from Procedure During Procedure Revision

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)	
0	2	0 9	1	9	1	0	0	0	0	0 3 2 2 9 1	0 5 0 0 0

OPERATING MODE (8)

POWER LEVEL (10) 1 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: P. F. Carteaux, Safety and Assessment Superintendent

TELEPHONE NUMBER: AREA CODE 6 1 6 4 6 5 - 5 9 0 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 2-20-91, while revising the inspection procedure for fire dampers protecting safety-related areas, it was discovered that a previous procedure revision had inadvertently omitted a fire damper from the inspection list. Due to the omission, subsequent surveillance performance using the procedure did not include the omitted damper. The Plant's Technical Specifications require the performance of a visual inspection of each fire damper and its associated hardware at least once per 18 months. Prior to the omission, the damper was part of the procedure and had been properly surveilled.

At the time of discovery, the omitted damper was visually and functionally inspected and accepted. The damper showed no adverse effects due to the extended time interval and remained within specification during the unsurveyed period. Current Administrative Controls governing procedure revision review requirements will reduce the probability of similar events.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) D. C. Cook Nuclear Plant, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 1 6	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 1	0 0 2	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 368A's) (17)

Condition Prior to Occurrence

Unit Two in Mode One (power operation) at 100% power.

Description of Event

While revising the inspection procedure for fire dampers (EIIS/VJ-DMP) protecting safety-related areas, it was discovered that a previous procedure revision had inadvertently omitted fire damper 2-HV-SGPA-1. 2-HV-SGPA-1 is located in the switchgear cable spreading room. Due to the omission, subsequent surveillance performance using the procedure did not include the omitted damper. The Plant's Technical Specifications require the performance of a visual inspection of each fire damper and its associated hardware at least once per 18 months. Prior to the omission, the damper was part of the procedure and had been properly surveilled.

At the time of discovery (2-20-91), the omitted damper was outside the grace period of the surveillance cycle by 107 days. The damper was immediately visually and functionally inspected and accepted. The damper showed no adverse effects due to the extended time interval and remained within specification during the unsurveyed period.

Cause of Event

The cause of the event was the inadvertent omission of the fire damper from the surveillance procedure during a procedure revision. At what stage in the procedure revision process or how the omission occurred is not conclusively known.

Analysis of Event

The event is being reported under 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the Plant's Technical Specifications. At the time of discovery (2-20-91), the unsurveyed fire damper was outside the grace period of the surveillance cycle by 107 days. The damper was immediately visually and functionally tested and accepted. The damper showed no adverse effects due to the extended time interval and remained within specifications during this time frame. Based on the above, it has been concluded that the Event had no impact on public health and safety.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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FACILITY NAME (1) D. C. Cook Nuclear Plant, Unit 2	DOCKET NUMBER (2) 05000316	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		91	002	00	03	OF 03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Corrective Actions

At the time of discovery (2-20-91), the omitted damper was immediately visually and functionally inspected and accepted. The omitted damper was also added to the inspection procedure to ensure damper inspection in the next surveillance period. Current Administrative Controls governing procedure revision review requirements (which identified this discrepancy) will reduce the probability of similar events in the future; thus, no further action will be taken.

Failed Component Identification

None.

Previous Similar Events

None.