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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

. ACCESSION NBR: 9010010274 DOC. DATE: 90/09/17 NOTARIZED: NO DOCKET #
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
 AUTH. NAME AUTHOR AFFILIATION
 . WOJCIK, J.T. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 BLIND, A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 90-009-00: on 900821, access to extreme high radiation
 area not controlled per Tech Specs. W/900917 ltr.
W/9 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 5
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

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	COLBURN, T.	1		1				
INTERNAL:	ACNW	2		2	AEOD/DOA	1		1
	AEOD/DSP/TPAB	1		1	AEOD/ROAB/DSP	2		2
	NRR/DET/ECMB 9H	1		1	NRR/DET/EMEB 7E	1		1
	NRR/DLPQ/LHFB11	1		1	NRR/DLPQ/LPEB10	1		1
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	REG-FILE 02	1		1	RES/DSIR/EIB	1		1
	RGN3 FILE 01	1		1				
EXTERNAL:	EG&G BRYCE, J.H	3		3	L ST LOBBY WARD	1		1
	NRC PDR	1		1	NSIC MAYS, G	1		1
	NSIC MURPHY, G.A	1		1	NUDOCS FULL TXT	1		1

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Indiana Michigan
Power Company
Cook Nuclear Plant
P.O. Box 458
Bridgman MI 49106
616 465 5901



September 17, 1990

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73
entitled Licensee Event Reporting System, the following
report is being submitted:

90-009-00

Sincerely,



A.A. Blind
Plant Manager

AAB:clj

Attachment

cc: D.H. Williams, Jr.
A.B. Davis, Region III
M.P. Alexich
P.A. Barrett
J.E. Borggren
R.F. Kroeger
B. Walters - Ft. Wayne
NRC Resident Inspector
J.G. Gitter, NRC
J.G. Keppler
M.R. Padgett
G. Charnoff, Esq.
Dottie Sherman, ANI Library
D. Hahn
INPO
S.J. Brewer/B.P. Lauzau
R.A. Svensson

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) D. C. COOK NUCLEAR PLANT - UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5 1	PAGE (3) 1 OF 03
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TITLE (4) **ACCESS TO AN EXTREME HIGH RADIATION AREA NOT CONTROLLED IN ACCORDANCE WITH TECHNICAL SPECIFICATIONS**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)											
0	8	2	1	9	0	9	0	0	0	0	9	0	0	9	1	7	9	0	0	9	D. C. COOK PLANT UNIT 2	0 5 0 0 0 3 1 6
0	8	2	1	9	0	9	0	0	0	0	9	0	5	0	0	0	0	0	3	1	0	6

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 1 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)

NAME J. T. WOJCIK - TECHNICAL PHYSICAL SCIENCES DEPARTMENT SUPERINTENDENT	TELEPHONE NUMBER
	AREA CODE 6 1 6
	4 6 5 - 5 9 0 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH: DAY: YEAR:
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On August 21, 1990 at 1315 hours it was discovered that the outer door to the 587' elevation Drumming Room was unlatched and the lock on the internal extreme high radiation area gate was found unlocked. The subject gate controls access to the high level waste storage area. There were no inoperable structures, components or systems that contributed to this event.

The cause of this event is attributed to a cognitive error by an unknown individual who left the gate unlocked.

Upon discovery of the deviant condition the gate was immediately locked to prevent unauthorized entry.

The NRC was notified via the Emergency Notification System (ENS) at approximately 1315 hours on August 21, 1990.



✓

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) D. C. COOK NUCLEAR PLANT UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 0	— 0 0 9	— 0 0	0 2	OF	0 3

TEXT (if more space is required, use additional NRC Form 366A's) (17)

Conditions Prior to Occurrence

Unit 1 reactor (EIIS/AB-RCT) in Mode 1 (Power Operation) at 100 percent rated thermal power. Unit 2 reactor in Mode 6 (Refueling).

Description of Event

On August 21, 1990 at 1315 hours it was discovered that the outer door to the 587' elevation Drumming Room was unlatched and the lock on the internal extreme high radiation area gate (EIIS/NE-GATE) was found open. The subject gate controls access to the high level waste storage area inside the 587' Drumming Room.

The area was found open by the Radiation Protection Manager while on a management tour. Dose rates in the general area were found to be 1200 mrem/hr posing violation of Technical Specification 6.12.2. There were no inoperable structures, components, or system that contributed to this event.

Cause of Event

The cause of this event is attributed to a cognitive error by an unknown individual who left the lock in an unlocked condition. Additionally, existing procedure controls did not preclude occurrence of this event.

Analysis of Event

This event is being reported in accordance with 10 CFR 50.73(a)(2)(i) as an event that is a deviation from the plant's Technical Specifications. Technical Specification 6.12.2 requires that in addition to the requirements of 6.12.1 that areas in which the intensity of radiation is greater than 1000 mrem/hr that locked doors shall be provided to prevent unauthorized entry into such areas.

Investigation into this event did not reveal any unauthorized entries into this area. Additionally, August thermoluminescent dosimeter (TLD) results did not reveal any exposures over administrative limits. Based on the above it is concluded that the event did not adversely impact the health and safety of the public.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 0	- 0 0 9	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Corrective Action

Upon discovery of the deviant condition, the gate was immediately locked to prevent unauthorized entry.

To preclude recurrence the importance of extreme high radiation area door and key control was stressed to all Radiation Protection personnel. Additionally the following actions will be taken:

1. The program for traceability and accountability of extreme high radiation area keys will be enhanced to include information regarding location to be entered and time key returned when using these keys.
2. The Technical Specification Radiation Protection Technician's responsibilities will be revised to require a twelve hour verification of all accessible extreme high radiation area gates.
3. A design change request will be submitted to install Automated Control Access Device (ACAD) readers for the following extreme high radiation area doors: (It should be noted that entry into the general area is already controlled via the ACAD readers. The design change will add additional controls to entry into the actual extreme high radiation area).
 - Spent Resin Storage Tank
 - Volume Control Tank Rooms
 - Seal Water Injection Filter Cubicles
 - Reactor Coolant System Filter Cubicles
 - 617' Elevation Demineralizers
 - 587' Elevation High Level Waste Storage Area

Failed Component Identification

None.

Previous Similar Events

Unit 2 89-001-00

LICENSEE EVENT REPORT REVIEW/EVALUATION FORM

CONDITION/PROBLEM REPORT NO.: 12-09-90-142190-1090 LER NO.: 90 1 109 1 00

RESPONSIBLE DEPARTMENT: Technical Process Service DOCKET/SEQUENTIAL NO./REVISION

The following checklist (Items 1-7) outlines the required contents of an LER and is intended to be used as an aid to the reviewer (see Section 4.0 of PMSO.098 for additional details/guidance).

- ✓ 1. A brief abstract describing the major occurrences during the event, including: a) all component or system failures that contributed to the event and b) significant corrective action taken or planned to prevent recurrence.
- ✓ 2. A clear, specific, narrative description within the text of what occurred so that knowledgeable readers conversant with the design of commercial nuclear power plants, but not familiar with the details of a particular plant, can understand the complete event. The narrative description must include the following specific information as appropriate for the particular event:
 - ✓ (A) Plant operating conditions before the event.
 - NA (B) Status of structures, components, or systems that were inoperable at the start of the event and that contributed to the event.
 - ✓ (C) Dates and approximate times of occurrences.
 - NA (D) The cause of each component or system failure or personnel error, if known.
 - NA (E) The failure mode, mechanism, and effect of each failed component, if known.
 - ✓ (F) The Energy Industry Identification System component function identifier and system name of each component or system referred to in the LER. The Energy Industry Identification System is defined in: IEEE Std. 803-1983 (May 16, 1983) Recommended Practices for Unique Identification Plants and Related Facilities - Principles and Definition.
 - NA (G) For failure of components with multiple functions, include a list of systems or secondary functions that were also affected.
 - NA (H) For failure that rendered a train of a safety system inoperable, an estimate of the elapsed time from the discovery of the failure until the train was returned to service.
 - ✓ (I) The method of discovery of each component or system failure or procedural error.
 - NA (J) (1) Operator actions that affected the course of the event, including operator errors, procedural deficiencies, or both, that contributed to the event.
 - ✓ (2) For each personnel error, the licensee shall discuss:
 - NA (a) Whether the error was a cognitive error (e.g., failure to recognize the actual plant condition, failure to realize which systems should be functioning, failure to recognize the true nature of the event or a procedural error;
 - NA (b) Whether the error was contrary to an approved procedure, was a direct result of an error in an unapproved procedure, or was associated with an activity or task that was not covered by an approved procedure;
 - NA (c) Any unusual characteristics of the work location (e.g., heat, noise) that directly contributed to the error; and
 - NA (d) The type of personnel involved (i.e., contractor personnel, utility-licensed operator, utility non-licensed operator, other utility personnel).
 - NA (K) Automatically and manually initiated safety system responses.
 - NA (L) The manufacturer and model number (or other identification) of each component that failed during the event.
- ✓ 3. An assessment of the safety consequences and implications of the event. This assessment must include the availability of other systems or components that could have performed the same function as the components and systems that failed during the event.
- ✓ 4. A description of any corrective/preventive actions planned as a result of the event, including those to reduce the probability of similar events occurring in the future.
- ✓ 5. Reference to any previous similar events at the same plant that are known to the licensee.
- ✓ 6. The name and telephone number of a person within the licensee's organization who is knowledgeable about the event and can provide additional information concerning the event and the plant's characteristics.

X indicates that the item was included in the LER.

N/A indicates that the item is not applicable to the LER.