

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8905250071 DOC. DATE: 89/05/19 NOTARIZED: NO DOCKET #
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
 AUTH. NAME AUTHOR AFFILIATION
 BEILMAN, T.P. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 SMITH, W.G. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 89-008-00: on 890420, procedure inadequacy results in TS
 required monthly calibr checks being performed quarterly.
 W/8 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
PD3-1 LA	1 1	PD3-1 PD	1 1
STANG, J	1 1		
INTERNAL: ACRS MICHELSON	1 1	ACRS MOELLER	2 2
ACRS WYLIE	1 1	AEOD/DOA	1 1
AEOD/DSP/TPAB	1 1	AEOD/ROAB/DSP	2 2
DEDRO	1 1	IRM/DCTS/DAB	1 1
NRR/DEST/ADE 8H	1 1	NRR/DEST/ADS 7E	1 0
NRR/DEST/CEB 8H	1 1	NRR/DEST/ESB 8D	1 1
NRR/DEST/ICSB 7	1 1	NRR/DEST/MEB 9H	1 1
NRR/DEST/MTB 9H	1 1	NRR/DEST/PSB 8D	1 1
NRR/DEST/RSB 8E	1 1	NRR/DEST/SGB 8D	1 1
NRR/DLPQ/HFB 10	1 1	NRR/DLPQ/QAB 10	1 1
NRR/DOEA/EAB 11	1 1	NRR/DREP/RPB 10	2 2
NUDOCS-ABSTRACT	1 1	REG FILE 02	1 1
RES/DSIR/EIB	1 1	RES/DSR/PRAB	1 1
RGN3 FILE 01	1 1		
EXTERNAL: EG&G WILLIAMS, S	4 4	FORD BLDG HOY, A	1 1
L ST LOBBY WARD	1 1	LPDR	1 1
NRC PDR	1 1	NSIC MAYS, G	1 1
NSIC MURPHY, G.A	1 1		

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK,
 ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION
 LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 43 ENCL 42

Indiana Michigan
Power Company
Cook Nuclear Plant
P.O. Box 458
Bridgman, MI 49106
616 465 5901



May 18, 1989

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Operating License DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73
entitled Licensee Event Reporting System, the following
report is being submitted:

89-008-00

Sincerely,

W. G. Smith, Jr.
Plant Manager

WGS:clw

Attachment

cc: D. H. Williams, Jr.
A. B. Davis, Region III
M. P. Alexich
P. A. Barrett
J. E. Borggren
R. F. Kroeger
NRC Resident Inspector
J. F. Stang, NRC
R. C. Callen
G. Charnoff, Esq.
Dottie Sherman, ANI Library
D. Hahn
INPO
PNSRC
A. A. Blind
S. J. Brewer/B. P. Lauzau

8905250071 890519
PDR ADOCK 05000315
S PIC

IE22
11

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) D. C. COOK NUCLEAR PLANT - UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 3 1 5					PAGE (3) 1 OF 0 3		
TITLE (4) PROCEDURE INADEQUACY RESULTS IN TECHNICAL SPECIFICATION REQUIRED MONTHLY CALIBRATION CHECKS BEING PERFORMED QUARTERLY																	
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)				
0 4	2 0	8 9	8 9	0 0 8	0 0	0 5	1 9	8 9	D.C. COOK - UNIT 2				0 5 0 0 0 3 1 6				
OPERATING MODE (9) 6			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
POWER LEVEL (10) 0 0 0			20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)		
			20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)		
			20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
			20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
			20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
			20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																	
NAME T. P. BEILMAN INSTRUMENTATION AND CONTROL DEPARTMENT SUPERINTENDENT										TELEPHONE NUMBER							
										AREA CODE 6 1 6 4 6 5 - 5 9 0 1							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																	
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs							
SUPPLEMENTAL REPORT EXPECTED (14)																	
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO		EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 20, 1989 it was determined that the Power Range Neutron Flux instrumentation monthly channel functional surveillance test procedures for Units 1 and 2 do not satisfy the intent of Technical Specification 3.3.1.1, Table 4.3-1, Item 2, Notation (3). This determination was made based on a generic Westinghouse letter entitled "Calibration of AFD Instrumentation" dated December 1, 1988. The stated intent of the Westinghouse document is to specify how the comparison of excore indicated AFD for 1) the Delta-I meters on the main control board, 2) the plant process computer, and 3) the value of AFD input to the F(Delta-I) penalty function generator, to the incore measured Axial Flux Difference should be evaluated. The plant surveillance test procedure complied with these recommendations except for the monthly comparison of the excore indicated value of the AFD input to the F(Delta-I) penalty function generator to the incore measured AFD. The comparison has been performed as part of the quarterly channel calibration procedure.

The reason for the omission could not be determined. It is believed that a contributing factor was the lack of specific guidance of when and how the calibration of the separate circuits of the excore AFD channel should be verified. The F(Delta I) penalty function generators have been incorporated into the Unit 2 Power Range Nuclear Instrumentation Channel Functional Test procedures. Procedures for Unit 1 will be completed prior to entry into a mode in which the surveillance is required following the current refueling outage.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1) D. C. COOK NUCLEAR PLANT - UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 9	0 0 8	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 308A's) (17)

Conditions Prior To Occurrence

Unit One in Mode 6 (refueling).

Unit Two in Mode 1 at 100 percent Rated Thermal Power.

Description of Event

On April 20, 1989 it was determined that the Power Range Neutron Flux instrumentation (EIIS/IG) monthly channel functional surveillance test procedures for Units 1 and 2 do not satisfy the intent of Technical Specification 3.3.1.1, Table 4.3-1, Item 2, Notation (3). This determination was made based on a generic Westinghouse letter entitled "Calibration of AFD Instrumentation" dated December 1, 1988. The stated intent of the Westinghouse document is to specify how the comparison of excore (EIIS/IG-JS) indicated AFD for 1) the Delta-I meters (EIIS/IG-XI) on the main control board, 2) the plant process computer (EIIS/ID), and 3) the value of AFD input to the F(Delta-I) penalty function generator, to the incore (EIIS/IG-RI) measured Axial Flux Difference should be evaluated. The plant surveillance test procedure complied with these recommendations except for the monthly comparison of the excore indicated value of the AFD input to the F(Delta-I) penalty function generator to the incore measured AFD.

Cause of the Event

The reason for not including the checking of the value of AFD input to the F(Delta-I) penalty function generator in the monthly Power Range Neutron Flux Instrumentation channel functional test procedures could not be determined. It is believed that a contributing factor was the lack of specific guidance of when and how the calibration of the separate circuits of the excore AFD channel should be verified.

Analysis of Event

This event is being reported per 10CFR50.73 (a)(2)(v) as the plant was being operated outside the requirements of Technical Specifications due to the procedure inadequacy.

The input to the F(Delta-I) penalty function generator has always been checked and calibrated by a quarterly channel calibration procedure for Technical Specification Table 4.3-1, Item 2, but have not been checked by the monthly channel functional test. A review of past calibration test results showed the input values to have remained stable with infrequent minor adjustments required. Based on this fact, failing to check this input on the monthly basis did not constitute a significant hazard to the health and safety of the public.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
D. C. COOK NUCLEAR PLANT - UNIT 1	0 5 0 0 0 3 1 5	8 9	0 0 8	0 0	0 3	OF 0 3

TEXT (If more space is required, use additional NRC Form 368A's) (17)

Corrective Action

On April 20, 1989 the inputs to the Unit 2 F(Delta-I) penalty function generators were checked for proper function and were found to be within specifications. The monthly channel functional test procedures for Unit 2 have been revised. The Unit 1 procedures will be revised prior to the unit entering a mode in which the surveillance is required following the current refueling outage.

Failed Component Identification

None.

Previous Similar Events

LER 50-315/88-003-0
LER 50-315/87-014-0
LER 50-315/86-006-0
LER 50-315/86-007-0
LER 50-315/86-013-0