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 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316  
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 MURLEY,T.E.                      Document Control Branch (Document Control Desk)

SUBJECT: Application for amends to Licenses DPR-58 & DPR-74, revising  
 submittal of 870326.

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AEP:NRC:0916AL

Donald C. Cook Nuclear Plant Units 1 and 2  
License Nos. 50-315 and 50-316  
Docket Nos. DPR-58 and DPR-74  
CHANGES TO TECHNICAL SPECIFICATION CHANGE SUBMITTAL  
OF MARCH 26, 1987 (AEP:NRC:0916W)

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

Attn: T. E. Murley

August 31, 1988

Dear Dr. Murley:

Pursuant to a request by your staff on August 9, 1988, attached are Technical Specification (T/S) pages that revise our submittal of March 26, 1987 (AEP:NRC:0916W). The changes are consistent with your staff's request, and in all circumstances are more restrictive than our previous request or are administrative in nature. Therefore, we believe the revisions do not require notification in the Federal Register per 10 CFR 50.91. The changes are described below.

1) Pages 3/4 3-9 (Unit 1) and 3/4 3-8 (Unit 2)

The NRC staff requested that the description of the P-7 and P-8 interlock in Table 3.3-1 be revised to provide greater detail on the interlock functions. The logic for reactor trip on low flow in any loop can be actuated by either of two signals. These are (1) two of three loop low flow indications and (2) loop reactor coolant pump (RCP) breaker position open. The output of the "OR" logic of these two signals is input to the loop logic, one of four loops indicating low flow above P-8 and two of four loops indicating low flow below P-8 and above P-7. In principle, a trip below P-8 and above P-7 could result from an RCP breaker open in one loop and two of three low flow indications in a different loop. The interlock descriptions were revised to reflect the fact that RCP breaker and loop low flow are treated as equivalent indications of low flow. This change is purely administrative in nature, intended only to clarify the T/S description of the existing P-7 and P-8 interlocks.

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The changes described above were unrelated to our original AEP:NRC:0916W request, which proposed to conservatively change the Unit 1 P-8 setpoint from greater than or equal to 51% of rated thermal power to greater than or equal to 31% of rated thermal power, to achieve consistency with the Unit 2 T/S. No changes to the corresponding Unit 2 page were requested in AEP:NRC:0916W. However, since the staff's comment was equally applicable to the Unit 2 T/S, we have included the change for both unit's T/Ss in the attachment to this letter.

2) Pages 3/4 3-12, 14 (Unit 1) and 3/4 3-11, 13 (Unit 2)

AEP:NRC:0916W proposed to add T/S 4.0.4 exemptions for the channel calibrations and channel functional tests for the Unit 1 source range, neutron flux reactor trip (Table 4.3-1 Functional Unit 6). Identical changes had been granted for the Unit 2 source range surveillances in Amendment 82, dated May 21, 1986.

The surveillances are required, per T/S Table 4.3-1, to be current in Modes 2, 3, 4, and 5 (below the P-6 setpoint). The exemption for the 18-month channel calibration was proposed to address the situation in which the calibration may expire before the reactor leaves Mode 1 and enters Mode 2, since the calibration cannot be performed in its entirety in Mode 1. Similarly, the T/S 4.0.4 exemption for the monthly channel functional test was proposed to address the situation that results from a reactor trip after continuous power operation of more than 31 days. The surveillance cannot be performed at power without damaging the source range detectors.

Rather than grant a 4.0.4 exemption for Unit 1 that would be applicable regardless of what modes the reactor had been in prior to mode changes, the NRC staff requested us to modify our proposed exemption to require that the surveillances be completed within 24 hours after leaving Mode 1. This change, which is more restrictive than our original request, has been incorporated into the pages found in the attachment to this letter. Because the NRC staff's request was equally applicable to the Unit 2 T/Ss, we have also included the revised T/S 4.0.4 conditions for the Unit 2 T/Ss in the attachment to this letter.

10 CFR 50.92 Evaluation

As discussed previously, the changes proposed in this submittal are either more restrictive than previously proposed in our letter AEP:NRC:0916W, or are administrative in nature. Thus, the

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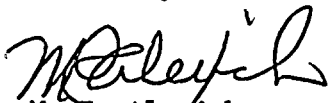
conclusions of the 10 CFR 50.92 evaluation submitted in AEP:NRC:0916W remain valid, and therefore the modified changes should not 1) involve a significant increase in the probability or consequences of a previously analyzed accident, 2) create the possibility of a new or different kind of accident from any previously analyzed or evaluated, or 3) involve a significant reduction in a margin of safety.

Other Licensing Issues

A check in the amount of \$150.00 was enclosed with our original submittal (AEP:NRC:0916W). Because this submittal only modifies the original submittal, at the NRC staff's request, we believe no additional fees per 10 CFR 170.12 are required.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Sincerely,



M. P. Alexich  
Vice President

ldp

Attachments

cc: D. H. Williams, Jr.  
W. G. Smith, Jr. - Bridgman  
G. Bruchmann  
R. C. Callen  
G. Charnoff  
NRC Resident Inspector - Bridgman  
A. B. Davis - Region III

Attachment to AEP:NRC:0916AL  
Revised Technical Specification Pages