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 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana & 05000315
 AUTH. NAME: ALEXICH, M.P. AUTHOR AFFILIATION: Indiana & Michigan Electric Co.
 RECIPIENT NAME: DENTON, H.R. RECIPIENT AFFILIATION: Office of Nuclear Reactor Regulation, Director

SUBJECT: Requests approval for use of encl new peaking factor limit
 rept. for Cycle 8. Rept presumes target band of greater than
 or equal to 5% target axial flux difference. Rept withheld.

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INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631
COLUMBUS, OHIO 43216

March 20, 1984

AEP:NRC:0745L

Donald C. Cook Nuclear Plant Unit No. 1
Docket No. 50-315
License No. DPR-58

UNIT 1 CYCLE 8
PEAKING FACTOR LIMIT REPORT UPDATE

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

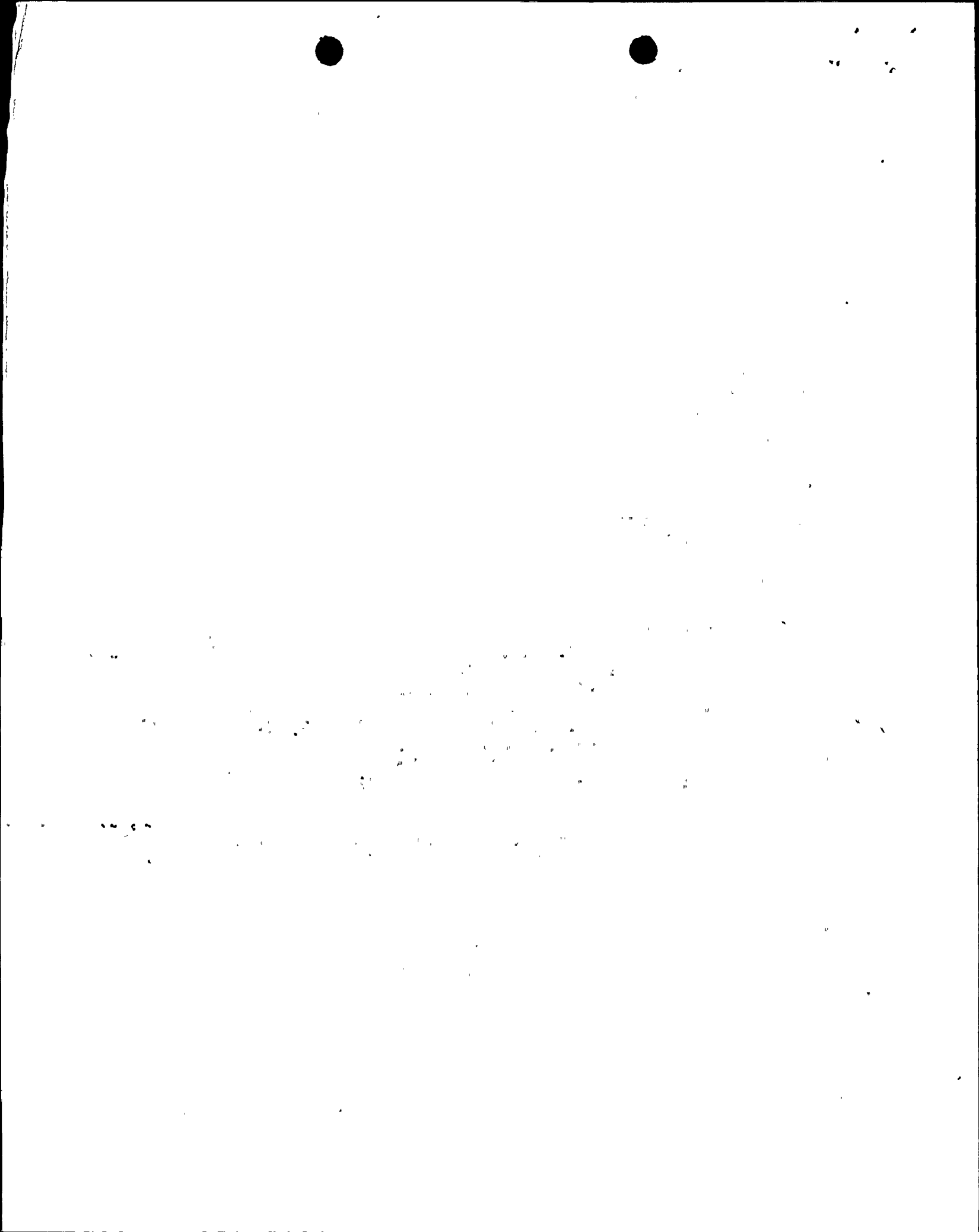
Dear Mr. Denton:

By this letter and its attachment, we request your approval for use of a new Peaking Factor Limit Report for Donald C. Cook Unit 1 Cycle 8, as described in Technical Specification 4.2.2.2.c. The V(Z) data in this new Peaking Factor Limit Report are not based on a new calculation. Rather, the existing results from the previous calculation are specified in the form of a V(Z) for each of several burnup steps. By comparison, the previous Peaking Factor Limit Report, submitted in letter AEP:NRC:0745H, dated August 31, 1983, showed one V(Z) function over the whole cycle; incorporating the most limiting values from the individual burnup V(Z)'s. The new Peaking Factor Limit Report presumes a target band of $\pm 5\%$ about the target axial flux difference.

This use of the more detailed Peaking Factor Limit Report is required, as quickly as possible, for Unit 1 to obtain full power operation. The need to use the burnup dependent V(Z) data was discovered only after some time of Cycle 8 power operation, when we found a slight increase in the F_0 peaking factor above that which had been predicted. The use of burnup dependent V(Z) data has been reviewed by our reload fuel supplier, Westinghouse Electric Corporation, and has been shown not to exceed any safety limit, including those limits specified in 10 CFR 50.46 and 10 CFR 50 Appendix K. On this basis we have concluded that its use will not constitute an unreviewed safety question as defined in 10 CFR 50.59.

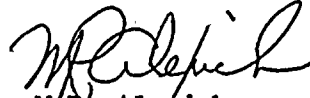
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This document has been prepared following Corporate procedures which incorporated a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,



4/24/81
3-12-81

M.P. Alexich
Vice President

cc: John E. Dolan - Columbus.
W.G. Smith, Jr. - Bridgman
R.C. Callen
G. Charnoff
E.R. Swanson, NRC Resident Inspector - Bridgman

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