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 RECIP. NAME: DENTON, H.R. RECIPIENT AFFILIATION: Office of Nuclear Reactor Regulation, Director

SUBJECT: Application for amend to Licenses DPR-58 & DPR-74. Amend provides required NUREG-0737 Tech Specs, per Generic Ltr 82-16.

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 AUTH: DATE
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SUBJECT: Application for Agency to License TRS-25 & TRS-25-A
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INDIANA & MICHIGAN ELECTRIC COMPANY

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April 8, 1983
AEP:NRC:0744

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
PROPOSED NUREG-0737 TECHNICAL SPECIFICATION CHANGES

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Denton:

This letter and its Attachments respond to the Generic Letter No. 82-16 request for the submittal of required NUREG-0737 Technical Specifications (T/S).

Attachment 1 to this letter presents the disposition of those NUREG-0737 items listed in Generic Letter No. 82-16, which are applicable to the D. C. Cook Plant. Attachments 2 and 3 present the proposed T/S changes for Unit Nos. 1 and 2, respectively. The changes are indicated by a bar on the right hand margin of the page.

The proposed Technical Specification changes have been reviewed and approved by both the Plant Nuclear Safety Review Committee (PNSRC) and the Nuclear Safety Design Review Committee (NSDRC).

AEPSC considers the Technical Specification changes contained in Attachment 2 to constitute a Class III Amendment as defined in 10 CFR 170.22. The Class I duplicate fee is applicable to the Technical Specification changes contained in Attachment 3. Enclosed, therefore, is a check in the amount of \$4,400 for the processing of the aforementioned request.

*A046
w/check
\$ 4,400*

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This document has been prepared following Corporate Procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,



R. S. Hunter

Vice President

/os

cc: John E. Dolan - Columbus
M. P. Alexich
R. W. Jurgensen
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Charnoff
NRC Resident Inspector at Cook Plant - Bridgman
Tom Tambling - NRC Region III

ATTACHMENT 1 TO AEP:NRC:0744

(1) STA Training (I.A.1.1.3)

The Shift Technical Advisor's (STA) training requirements are currently given in Technical Specification (T/S) 6.3. The requirement that one STA be onsite during operation in modes 1, 2, 3 and 4 is given in Table 6.2-1 of the T/S. These requirements were made part of the T/S by way of Amendments 49 and 34 to the Units No. 1 and No. 2 T/S, respectively. The Safety Evaluation Report concerning this item contained in Mr. Steven A. Varga's letter of August 25, 1981, finds "that the licensee's submittal satisfies our requirements and is acceptable". As such, no new T/S concerning this item are required for the Cook Plant.

(2) Limit Overtime (I.A.1.3)

The proposed Technical Specification 6.2.2.g, concerning this item for Units Nos. 1 and 2 is contained in Attachments 2 and 3 respectively. The proposed Technical Specification references the limitations promulgated by the NRC Policy Statement on working hours.

(3) Short Term Auxiliary Feedwater System Evaluation (II.E.1.1)

The currently existing Technical Specification 3/4 7.1.2 requires the verification of each valve in the flow path, including valves which are locked, sealed, or otherwise secured in position, being in its correct position. The Safety Evaluation Report enclosed with Mr. S. A. Varga's October 6, 1980, letter concludes that Technical Specification 3/4 7.1.2 adequately meets Generic Recommendation GS-2, thereby confirming the reliability of AFW System. As such, no T/S concerning this item are required for the Cook Plant.

(4) Safety Grade AFW System Initiation and Flow Indication (II.E.1.2)

Mr. S. A. Varga's letter of June 16, 1981, and references therein, concluded that Item II.E.1.2 of NUREG-0737 was a closed item for the Cook Plant pending the installation of environmentally qualified auxiliary feedwater flow rate transmitters. As such no T/S are required for the Cook Plant.

For the record, new transmitters were installed by June 5, 1981. These transmitters were purchased as part of the Utility Transmitter Qualification Program sponsored by Wisconsin Electric Power Company in which AEP participated. The transmitters successfully completed testing on August 10, 1982, however the final test report has not been received yet.

(5) Dedicated Hydrogen Penetrations (II.E.4.1)

Mr. Steven A. Varga's letter to Mr. John Dolan dated September 11, 1981, notes that the NRC acceptance of the D. C. Cook Nuclear Plant Units Nos. 1 and 2 design with respect to NUREG-0578 Item 2.1.5, Dedicated Hydrogen Control Penetrations, also satisfies the requirements of NUREG-0737 Item II.E.4.1. As such, no T/S concerning this item are required for the Cook Plant.

(6) Containment Pressure Setpoint (II.E.4.2.5)

Currently, the appropriate setpoint is incorporated in the T/S for both Units and no modifications to the setpoint are necessary. This item has been closed out as per Enclosure 6 of the SER included in Mr. Steven A. Varga's letter to Mr. John E. Dolan dated September 9, 1981. As such, no new T/S concerning this item are required for the Cook Plant.

(7) Containment Purge Valves (II.E.4.2.6)

Our letter No. AEP:NRC:0370B dated August 30, 1982, proposed T/S changes to allow containment purging while in operational modes 1, 2, 3 and 4. These proposed T/S were approved by the NRC in Amendments No. 66 (Unit 1) and 47 (Unit 2) to the Cook Plant Operating Licenses.

(8) Radiation Signal on Purge Valves (II.E.4.2.7)

The intent of the model T/S concerning this item is currently incorporated in the D. C. Cook T/S. However, the entry into Table 3.3-5 suggested by Generic Letter 82-16, has not been incorporated since credit is not taken for the containment radioactivity signal to close the purge valves in the safety analysis. Containment isolation is caused at the Cook Plant by means of a variety of ESF signals which are already in the existing Table 3.3-5. Furthermore, the response time of the purge valves upon the receipt of the isolation signal is governed by T/S 3/4.6.3. For these reasons, no new T/S concerning this item are submitted for the Cook Plant.

(12) Reporting SV and RV Failures and Challenges (II.K.3.3)

In our letter No. AEP:NRC:0419A dated May 28, 1982, we committed to report all pressurizer safety and relief valve failures within 24 hours by telephone and to submit a written followup report within 14 days. Also, that all safety and relief valve challenges occurring after April 1, 1980, will be reported in the annual operating report. We are proposing Technical Specifications 6.9.1.5 (c) and 6.9.1.8 (j) in Attachments 2 and 3 which are identical to those model T/S presented in Generic Letter No. 82-16.

(13) Anticipatory Trip on Turbine Trip (II.K.3.12)

The logic network of D. C. Cook Nuclear Plant Unit Nos. 1 and 2 presently employs an anticipatory reactor trip on turbine trip, in conformance with NUREG-0737, Item II.K.3.12. This item is considered to be resolved for the D. C. Cook Plant, as per Mr. Steven A. Varga's letter of September 8, 1981. As such, no new T/S concerning this item are required for the Cook Plant.

Attachment 2 to AEP:NRC:0744
Changes to Unit 1 Technical Specifications