

INDIANA & MICHIGAN ELECTRIC COMPANY

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NEW YORK, N. Y. 10004

February 22, 1980
AEP:NRC:00313

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2
Docket Nos. 50-315 and 50-316
License No.s DPR-58 and DPR-74

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

This letter and its attachments request changes in several of the Appendix 'A' Technical Specifications for the Donald C. Cook Nuclear Plant Unit Nos. 1 and 2. Attachment 'A' to this letter contains the description and review of each change. Attachment 'B' contains the appropriate revised pages. A number of these changes are editorial in nature and are noted as such. Attachment 'C' provides supporting information for change No. 2. We request that the NRC file and process this technical specification change package as a single amendment.

All of the proposed technical specification changes contained herein have been reviewed and approved by the Plant Nuclear Safety Committee. The result of these reviews indicates that in no instance will the proposed technical specification change adversely affect the health and safety of the public.

This application for technical specification revision is considered to be a Class II License Amendment as per the provisions of 10 CFR 170.22. As required by Part 170 Subsection 22 a check for \$1,600.00 accompanies this submittal.

Very truly yours,

John E. Dolan
John E. Dolan
Vice President

*Apo1
S/I
AW/CHECK
\$1,600.00*

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Date 2-22-80 of Document
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Mr. Harold R. Denton, Director

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cc: R. S. Hunter
R. W. Jurgensen
G. Charnoff
R. C. Callen
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ATTACHMENT 'A' TO AEP:NRC:00313.

PROPOSED TECHNICAL SPECIFICATION REVISIONS
DONALD C. COOK NUCLEAR PLANT UNIT NOS. 1 AND 2

Change No. 1 - Unit Nos. 1 and 2

Unit No. 2 Technical Specification No. 4.7.1.2.a.2 - Page 3/4 7-5

Unit No. 1 Technical Specification No. 4.7.1.2.a.5 - Page 3/4 7-6

This change deletes the 'exemption' of positively controlled valves (those valves which are locked, sealed, or otherwise secured in position) from the Auxiliary Feedwater System surveillance requirement of a monthly verification of valve position. This change, performed in accordance with NRC Recommendation GS-2, as detailed in the attachment to Mr. D. G. Eisenhut's letter of October 30, 1979, does not represent any additional surveillance requirement and fulfills the commitment contained in our response letter of December 11, 1979, AEP:NRC:00300. Technical Specification 6.8.1.c requires that "written procedures shall be established, implemented and maintained..." "...for the surveillance and test activities of safety related equipment."

Change No. 2 - Unit No. 1 Only

The Technical Specification RCS Heatup and Cooldown curves, Figures 3.4-2 and 3.4-3, are being revised to reflect the results of the analysis of the first material surveillance capsule removed from the Unit No. 1 reactor pressure vessel. A report prepared by Southwest Research Inc. is submitted in support of this change (Attachment 'C').

Change No. 3 - Unit No. 1 Only (Editorial)

Specification 6.9.2.f incorrectly references Specification 4.7.9.1.3. The correct reference is Specification 4.7.7.1.3, "Seal Source Leakage". The revised page reflects the correct reference.

Change No. 4 - Unit No. 2 Only (Editorial)

Specifications 6.9.2.b - "Seismic Monitoring Instrumentation" and 6.9.2.c - "Meteorological Monitoring Instrumentation", should refer to Unit No. 1 Specifications 3.3.3.3 and 3.3.3.4, respectively. The revised Technical Specification page reflects the correct references.

Change No. 5 - Unit Nos. 1 and 2

Technical Specification Tables 2.2-1 and 3.3-4 have been amended to reflect the 6% (of narrow-range instrument span) increase in the "SG Water Level Low-Low" trip and ESF actuation setpoint due to reference leg heatup following a hypothetical high energy line rupture inside containment. The revised setpoints, 15% and 21% for Unit Nos. 1 and 2 respectively, were forwarded to the NRC via our response to IE Bulletin No. 79-21, AEP:NRC:00271, dated November 5, 1979.

Change No. 6 - Unit Nos. 1 and 2

The undervoltage protection and monitoring scheme will be modified as per the proposed modifications detailed in Attachment No. 7 to our AEP:NRC:00268 submittal dated December 17, 1979. The required modifications will be completed prior to startup from the next regularly scheduled refueling outage for each Unit. Subsequent to these modifications, the 'degraded voltage' function will be monitored on the 4 kv safety buses; Unit No. 1 buses T11A and T11D and Unit No. 2 buses T21A and T21D. As shown on revised Table 3.3-3, the 'degraded voltage' function will be monitored by three channels per bus, with a two out of three actuation logic.

Unit No. 1 Technical Specification Table Nos. 3.3-3, 3.3-4, and 4.3-2 have been revised to incorporate the changes discussed in (5) and (6) above. The surveillance requirements of Table 4.3-2, Item Nos. 7 and 8, have been revised to reflect the requirements of Table 3.3-3 for both Units.