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 AUTH. NAME AUTHOR AFFILIATION
 COLAN, J.E. Indiana & Michigan Electric Co.
 RECIP. NAME RECIPIENT AFFILIATION
 DENTON, H.R. Office of Nuclear Reactor Regulation

SUBJECT: Responds to NRC 790801 request for addl info re
 recirculation sump. Model test to be completed by Nov, 1979.
 Sump location drawings encl.

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INDIANA & MICHIGAN POWER COMPANY

P. O. BOX 18
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NEW YORK, N. Y. 10004

September 24, 1979
AEP:NRC:00260

Donald C. Cook Nuclear Plant Units No. 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
Request for Additional Information Regarding Recirculation Sump

Mr. Harold R. Denton
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

This letter is in response to a Request for Additional Information enclosed in Mr. A. Schwencer's letter of August 1, 1979 concerning the performance of the D.C. Cook Plant recirculation sump under LOCA/jet impingement conditions.

In response to Item 1 of the subject request, the following information is provided. The original sump model testing program did not include provisions for considering ice condenser drain flow or postulated pipe breaks. The final report of the original study was submitted in our letter of December 20, 1978 (AEP:NRC:00112). In our letter of December 29, 1978 (AEP:NRC:00110), we stated that our testing program would be extended to include jet impingement testing. This model testing has been proceeding since February. The current schedule calls for Alden Research Laboratory to complete model testing by November 1979. Based on this schedule a final report would be submitted to the NRC no later than January 20, 1980. Responses to the particular questions included in Item 1 will be provided at that time.

In response to Item 2 of the subject request, Figures 1 and 2 of Attachment 1 are enclosed. The figures are marked to indicate break locations being considered and containment water levels at various switchover points. The piping segments shown in Figure 2 which is an enlargement of Section B-9 of FSAR Figure 5.2-5 represent the only high pressure piping in the vicinity of the recirculation sump.

Very truly yours,

John E. Dolan
John E. Dolan
Vice President

JED:em

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*A001
5-11*

Mr. Harold R. Denton, Director

-2-

AEP:NRC:00260

cc: R. C. Callen
G. Charnoff
R. W. Jurgensen
R. S. Hunter
D. V. Shaller-Bridgman

ATTACHMENT 1

TO

AEP:NRC:00260



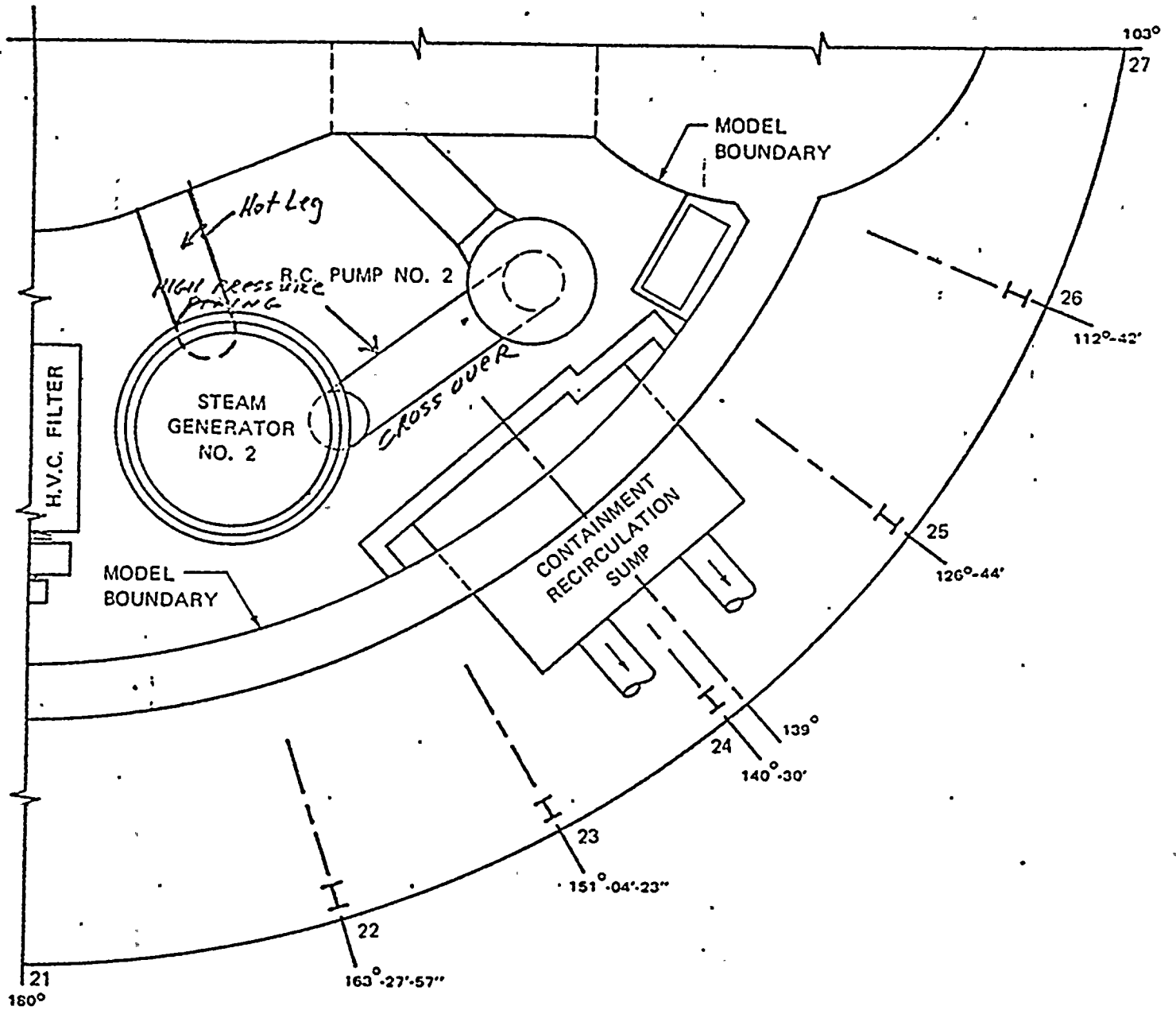
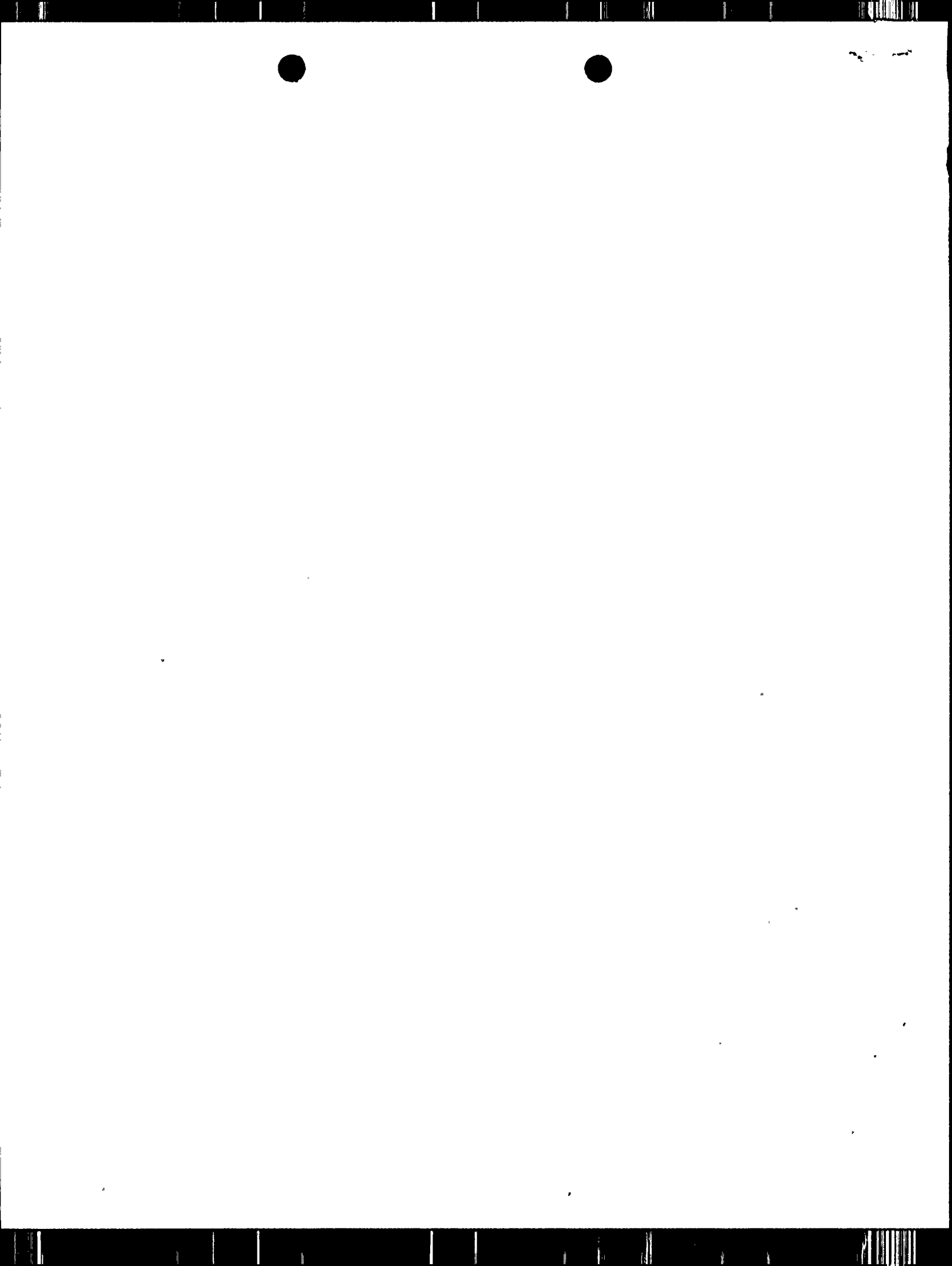
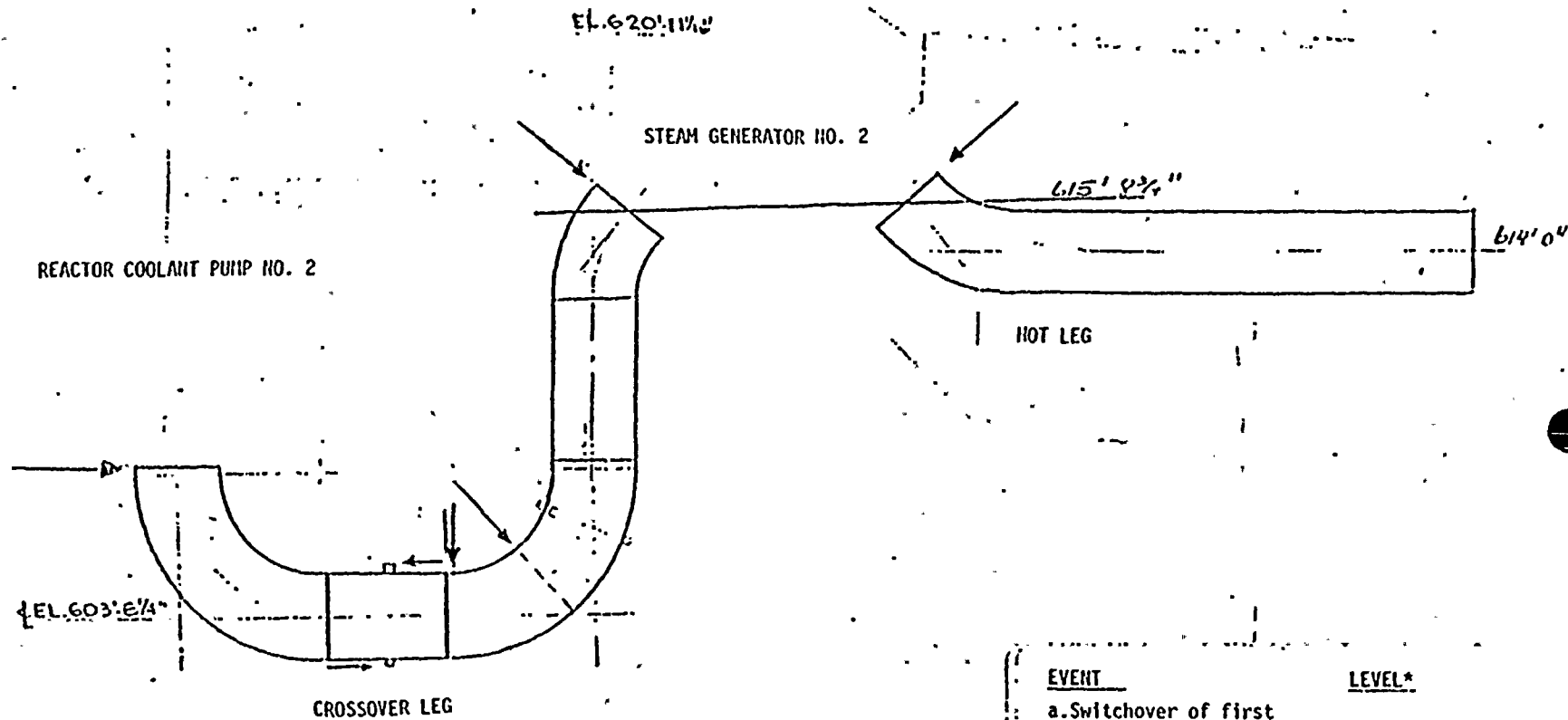


FIGURE 1 LOCATION OF SUMP WITHIN CONTAINMENT BUILDING



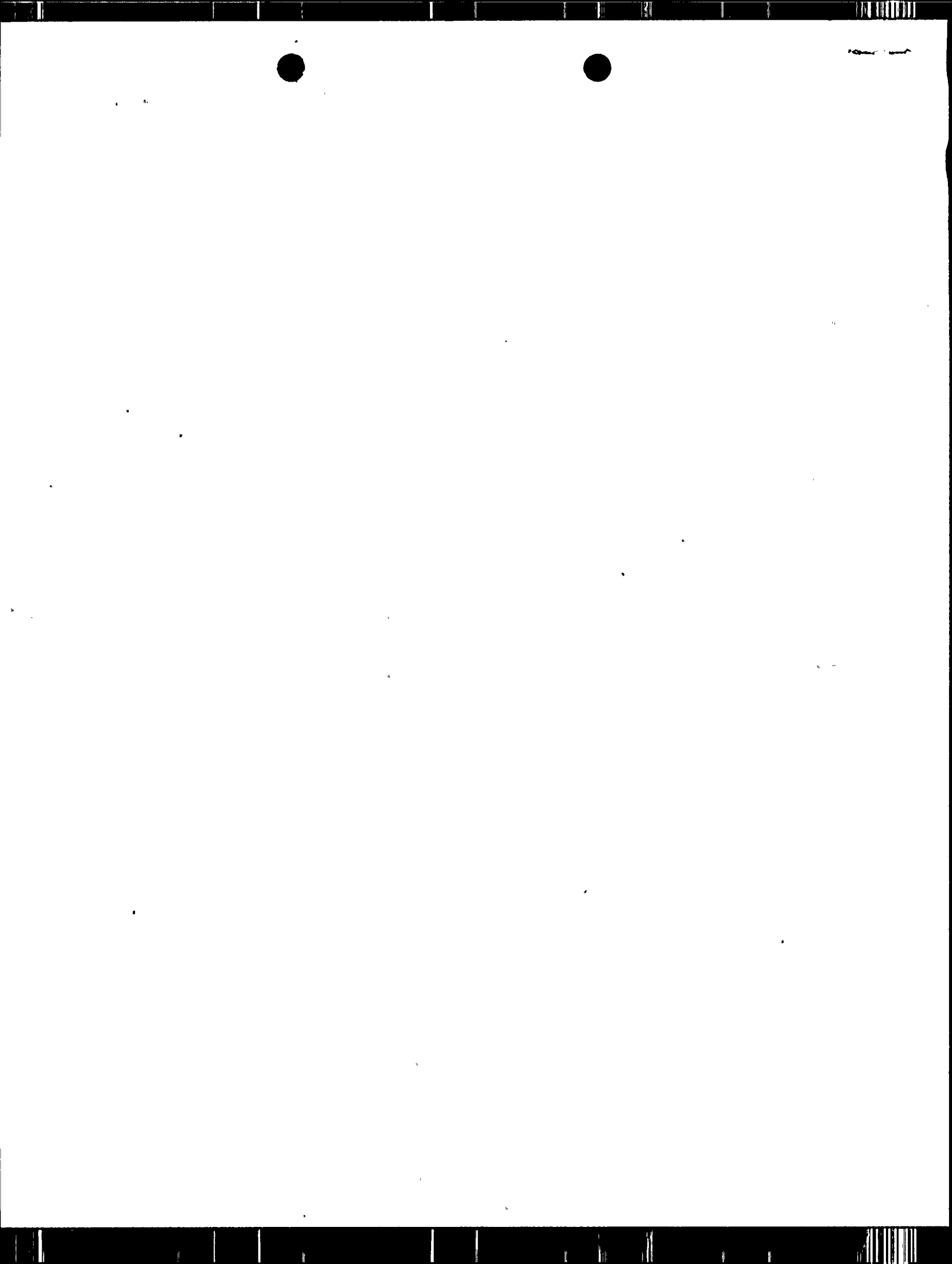


ARROWS DESIGNATE AREAS CONSIDERED FOR
POTENTIAL BREAK LOCATIONS

EVENT	LEVEL*
a. Switchover of first RHR Train	607' 4"
b. Switchover of second RHR Train	610' 0"
c. Water level at time of second switchover considering partial ice condenser melt-out	612' 0"

* WATER LEVELS ARE BASED ON BREAK LOCATIONS SHOWN

FIGURE 2





UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SEPTEMBER 18 1979

Docket Nos. 50-315
and 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Indiana and Michigan Power Company
Post Office Box 18
Bowling Green Station
New York, New York 10004

Dear Mr. Dolan:

RE: REACTOR CAVITY SEAL RING GENERIC ISSUE (PWR)

We have completed our review of your response to our letter of February 2, 1978, in regard to the potential for the reactor cavity annulus seal ring to become a destructive missile in the event of a loss-of-coolant accident pipe break inside the reactor cavity.

Your response acceptably resolves this issue for Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2.

Sincerely,

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

cc: See next page



Mr. John Dolan
Indiana and Michigan Electric Company
Indiana and Michigan Power Company

cc: Mr. Robert W. Jurgensen
Chief Nuclear Engineer
American Electric Power
Service Corporation
2 Broadway
New York, New York 10004

Gerald Charnoff, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, D. C. 20036

Citizens for a Better Environment
59 East Van Buren Street
Chicago, Illinois 60605

Maude Preston Palenske Memorial
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500 Market Street
St. Joseph, Michigan 49085

Mr. D. Shaller, Plant Manager
Donald C. Cook Nuclear Plant
P. O. Box 458
Bridgman, Michigan 49106

Mr. Robert Masse
Donald C. Cook Nuclear Plant
P. O. Box 458
Bridgman, Michigan 49106

SEP 25 1979