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 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana M 05000316

AUTH.NAME AUTHOR AFFILIATION
 GROBE, J.A. Region 3 (Post 820201)
 RECIP.NAME RECIPIENT AFFILIATION
 POWERS, R.P. American Electric Power Co., Inc.

SUBJECT: Forwards insp repts 50-315/99-10 & 50-316/99-10 on
 990417-0527. No violations noted. Two violations of NRC
 requirements occurred & being treated as non-cited
 violations.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
801 WARRENVILLE ROAD
LISLE, ILLINOIS 60532-4351

June 11, 1999

Mr. R. P. Powers
Senior Vice President
Nuclear Generation Group
American Electric Power Company
500 Circle Drive
Buchanan, MI 49107-1395

SUBJECT: D. C. COOK INSPECTION REPORT 50-315/99010(DRP); 50-316/99010(DRP)

Dear Mr. Powers:

On May 27, 1999, the NRC completed an inspection at your D. C. Cook Units 1 and 2 reactor facilities. The inspection was an examination of activities conducted under your license as they relate to compliance with the Commission rules and regulations and with the conditions of your license. Areas reviewed included Operations, Maintenance, Engineering, and Plant Support. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observations of activities in progress. The enclosed report presents the results of that inspection.

Your staff successfully restored the Unit 2 AB diesel generator to an operable condition along with the associated train of boration control resulting in an increased ability to address postulated reactor coolant dilution events. In addition, the continued effort to place both units on electrical back feed indicated an increased focus on reactor core safety issues. We also noted that during the conduct of the Expanded System Readiness Review process, your staff demonstrated a critical questioning attitude by effectively identifying technical issues related to the Containment Spray System.

While an increased focus on reactor core safety issues was evident, several examples were identified by the inspectors where your staff failed to consistently respond in a manner commensurate with the potential impact of the degraded condition on current plant configuration. The examples included your staffs' response to degraded and inoperable Essential Service Water pump discharge strainers, degraded 4 kV breakers, and source range instrument issues. In response to the inspectors' concerns, your staff took prompt and appropriate actions, including the initiation of compensatory measures when warranted. These actions ensured that equipment which could be required to support maintaining the reactor in a cold shutdown condition were available or returned to service in an expeditious manner.

During the inspectors review of previously identified regulatory issues, two violations of NRC requirements were identified. The first violation, identified by the inspectors in 1998, was due to an inappropriate surveillance procedure for the engineered safeguards ventilation system. The surveillance procedure did not direct the operators to restore the system to an operable configuration. The second violation, identified by your staff in 1995, involved the failure to

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restore several main steam safety valves to operable status prior to exceeding the Technical Specification Limiting Condition for Operation time limit.

These Severity Level IV violations are being treated as Non-Cited Violations (NCVs). Appendix C of the Enforcement Policy requires that for Severity Level IV violations to be dispositioned as NCVs, they be appropriately placed in the licensee's corrective action program. Implicit in that requirement is that the corrective action program be fully acceptable. The plant corrective action program was not adequate and has been the focus of significant attention by your staff to improve the program. While your staff and the NRC have not yet concluded that the corrective action program is fully effective, the corrective action program improvement efforts are underway and captured in the Restart Plan which is under the formal oversight of the NRC through the NRC Manual Chapter 0350 Process, "Staff Guidelines for Restart Approval." Consequently, these issues will be dispositioned as NCVs.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be placed in the NRC Public Document Room.

Sincerely,

/s/ J. A. Grobe

John A. Grobe, Director
Division of Reactor Safety

Docket Nos. 50-315; 50-316
License Nos. DPR-58; DPR-74

Enclosure: Inspection Report 50-315/99010(DRP);
50-316/99010(DRP)

cc w/encl: A. C. Bakken III, Site Vice President
T. Noonan, Acting Plant Manager
M. Rencheck, Vice President, Nuclear Engineering
R. Whale, Michigan Public Service Commission
Michigan Department of Environmental Quality
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MI Department of State Police
D. Lochbaum, Union of Concerned Scientists

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