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SUEJECT: Forwards insp repts 50-315/99-02 & 50-316/99-02 on 990202-19.No violations noted.

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Mr. R. P. Powers Senior Vice President Nuclear Generation Group American Electric Power Company 500 Circle Drive Buchanan, MI 49107-1395

SUBJECT: NRC INSPECTION REPORT 50-315/99002(DRS); 50-316/99002(DRS)

Dear Mr. Powers:

On February 19, 1999, the NRC completed a special inspection conducted at your Buchanan Michigan Corporate facility. This inspection was an examination of activities under your license as they relate to your implementation of the Expanded System Readiness Review program at your D. C. Cook Units 1 and 2 reactor facilities. The NRC understands that these reviews are intended to provide assurance that safety-related plant systems fulfill their design basis safety functions and to determine system restart readiness. The NRC will continue to monitor and assess the effectiveness of these efforts. The enclosed report documents the results of the inspection.

Areas examined during the inspection are identified in the report. Within these areas, the inspection consisted of a selective examination of procedures and representative records, and interviews with personnel. At the conclusion of the inspection, the findings were discussed with you and members of your staff.

No cited violations of NRC requirements were identified. In accordance with 10 CFR 2.790 of the NRC'S "Rules of Practice," a copy of this letter, the enclosures, and your response to this letter, if you choose to provide one, will be placed in the NRC Public Document Room (PDR).

We will gladly discuss any questions you have concerning this inspection.

Sincerely,

John A. Grobe, Director Division of Reactor Safety

Docket Nos.: 50-315; 316 License No.: DPR-58; DPR-74

Enclosure: Inspection Report 50-315/99002(DRS); 50-316/99002(DRS)

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R. Powers

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Sincerely,

John A. Grobe, Director Division of Reactor Safety

Docket Nos.: 50-315; 316 License No.: DPR-58; DPR-74

Enclosure: Inspection Report 50-315/99002(DRS); 50-316/99002(DRS)

cc w/encl: M. Rencheck, Vice President, Nuclear Engineering D. Cooper, Plant Manager R. Whale, Michigan Public Service Commission Michigan Department of Environmental Quality Emergency Management Division MI Department of State Police D. Lochbaum, Union of Concerned Scientists

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R. Powers

In accordance with 10 CFR 2.790 of the NRC'S "Rules of Practice," a copy of this letter, the enclosure, and your response to this letter, if you choose to provide one, will be placed in the NRC Public Document Room.

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Sincerely,

Original /s/ John A. Grobe

John A. Grobe, Director **Division of Reactor Safety**

Docket Nos.: 50-315; 316 License No.: DPR-58; DPR-74

Enclosure: Inspection Report 50-315/99002(DRS); 50-316/99002(DRS)

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EA No. 99-056

Mr. R. P. Powers Senior Vice President Nuclear Generation Group American Electric Power Company 500 Circle Drive Buchanan, MI 49107-1395

SUBJECT: NRC INSPECTION REPORT 50-315/99002(DRS); 50-316/99002(DRS)

Dear Mr. Powers:

On February 19, 1999, the NRC completed a special inspection conducted at your Buchanan Michigan Corporate facility. This inspection was an examination of activities under your license as they relate to your implementation of the Expanded System Readiness Review program at your D. C. Cook Units 1 and 2 reactor facilities. The NRC understands that these reviews are intended to provide assurance that safety-related plant systems fulfill their design basis safety functions and to determine system restart readiness. The NRC will continue to monitor and assess the effectiveness of these efforts. The enclosed report documents the results of the inspection.

Areas examined during the inspection are identified in the report. Within these areas, the inspection consisted of a selective examination of procedures and representative records, and interviews with personnel. At the conclusion of the inspection, the findings were discussed with you and members of your staff.

Based on the results of this inspection, the NRC has determined that a violation of NRC requirements occurred. The violation pertained to the failure of the quality assurance program to assure receipt of all technical information for safety related components provided by nuclear steam system supply vendors. This violation appears to be another manifestation of the design control breakdown that contributed to the extended shutdown and improvement initiatives under way at D. C. Cook. This Severity Level IV violation is being treated as a Non-Cited Violation (NCV). Appendix C of the Enforcement Policy requires that for Severity Level IV violations to be dispositioned as NCVs, they be appropriately placed in a licensee corrective action program. Implicit in that requirement is that the corrective action program be fully acceptable.

The D. C. Cook Plant corrective action program was not adequate and has been the focus of significant attention by your staff to improve the program. While your staff and the NRC have not yet concluded that the corrective action program is fully effective, the corrective action and design control program improvement efforts are underway and captured in the D. C. Cook Plant Restart Plan which is under the formal oversight of the NRC through the NRC Manual Chapter 0350 process, "Staff Guidelines for Restart Approval." Consequently, this issue will be dispositioned as an NCV.