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 MURLEY, T. E. Document Control Branch (Document Control-Desk)

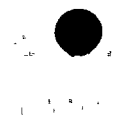
SUBJECT: Requests temporary code relief from testing containment spray heat exchangers be granted until next outage of sufficient duration, Apr 1988 for Unit 1 & May 1988 for Unit 2, to allow testing. Fee paid.

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REC'D w/check \$ 150.00



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AEP:NRC:0969J

Donald C. Cook Nuclear Plant Units 1 and 2
Docket Nos. 50-315 and 50-316
License Nos. DPR-58 and DPR-74
INSERVICE TEST (IST) - REQUEST FOR CODE RELIEF
OF CONTAINMENT SPRAY HEAT EXCHANGER

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Attn: T. E. Murley

October 29, 1987

Dear Dr. Murley:

The purpose of this letter, submitted in accordance with 10 CFR 50.55a(g)(6)(i), is to request temporary code relief for testing the D. C. Cook Units 1 and 2 containment spray heat exchangers.

Following the installation of two flushing ports on the heat exchangers' shell side, pressure testing in accordance with the ASME Boiler and Pressure Vessel Code Section XI, 1983 Edition, Subsection IWD Article IWD-5000 was required.

The heat exchangers were pressure tested to 118 psig, 110% of design pressure (105 psig) rather than the 1983 Code required value of 165 psig, 110% of the lowest safety valve setting (150 psig). The 118 psig test pressure meets the requirements of the 1974 Code edition which was in effect at the Cook Plant until July 1, 1986, and it exceeds the expected pressure on the shell side of the containment spray heat exchanger.

The source of the cooling water for the shell side of the heat exchangers is the essential service water (ESW) system. The highest pressure sources for the ESW system are the ESW pumps which have a maximum discharge pressure of 105 psig. Pressurization above this value could only occur if the cooling source were isolated while the heated tube side fluid continued to flow through the heat exchanger or if a tube leak allowed pressurization from the containment spray system. Either of these would require an unlikely combination of equipment malfunction and operator errors.

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Because of the foregoing, we consider the containment spray system to be operable and capable of performing its intended function.

Our alternatives are to pressure test the containment spray heat exchangers while the reactor is in operation or to reset the relief valve liftpoint so that the test already conducted is in compliance with the code. Both of these alternatives will require entering technical specification action statements and could potentially cause a reactor shutdown if the time limits were not complied with.

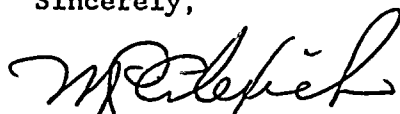
We are planning to test these heat exchangers to the correct hydrostatic pressure at the earliest opportunity. We request that this temporary relief be granted until the next outage of sufficient duration to allow testing. Currently, the next scheduled outages are April 1988 for Unit 1 and May 1988 for Unit 2.

A check in the amount of \$150.00 is attached with this letter for the NRC processing of the aforementioned request.

This letter has been reviewed by the PNSRC and will be reviewed by the NSDRG at the next regularly scheduled meeting. We will inform you if any changes are made as a result of the NSDRG review.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to ensure its accuracy and completeness prior to signature by the undersigned.

Sincerely,



M. P. Alexich
Vice President

cm

Attachment

cc: John E. Dolan
W. G. Smith, Jr. - Bridgman
R. C. Callen
G. Bruchmann
G. Charnoff
NRC Resident Inspector - Bridgman
A. B. Davis - Region III