Docket Nos.: 50-315 and 50-316

MAR 27 1987

Mr. John Dolan, Vice President Indiana and Michigan Electric Company c/o American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43216

Dear Mr. Dolan:

By letters dated January 22, 1986 and February 27, 1987, the Indiana and Michigan Electric Company submitted information on the reactor vessel materials properties and analyses on the protection against pressurized thermal shock for the Donald C. Cook Nuclear Plant, Unit Nos. 1 and 2. We have completed our review of the information and analysis, and our Safety Evaluation is enclosed.

In view of:

- (a) the Pressure-Temperature updating requirements for the fracture toughness of the beltline material in 10 CFR 50 Appendix G, and
- (b) the fact that the RT_{PTS} value is readily available from the calculation of the Pressure-Temperature limits, and
- (c) the staff desires to be informed on the current value of the RT_{PTS} for all PWRs,

we request that IMEC submit a re-evlauation of the RT_{PTS} and a comparison to the prediction of Reference 1 along with the future Pressure-Temperature operating limits which are required by 10 CFR 50 Appendix G. It should be noted that this re-evaluation is a requirement by 10 CFR 50.61, whenever core loadings, surveillance measurements, or other information indicate a significant change in projected values.

This request for information is covered under OMB clearance No. 3150-0011.

Sincerely,

しんちん B. J. Youngblood, Director PWR Project Directorate #4 Division of PWR Licensing-A

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Enclosure: As stated

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

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Sincere} B. J. Houngblood, Director

PWR Project Directorate #4 Division of PWR Licensing-A

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Mr. John Dolan Indiana and Michigan Electric Company

cc: Mr. M. P. Alexich Vice President Nuclear Operations American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43215

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The Honorable John E. Grotberg United States House of Representatives Washington, DC 20515

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