



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

December 4, 2017

Mr. Anthony L. Williams IV  
Site Vice President  
Sequoyah Nuclear Plant  
Tennessee Valley Authority  
P.O. Box 2000, OPS 4A-SQN  
Soddy Daisy, TN 37384-2000

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNIT 1 - REVIEW OF THE STEAM  
GENERATOR TUBE INSERVICE INSPECTIONS FOR CYCLE 21 REFUELING  
OUTAGE IN THE FALL OF 2016 (CAC NO. MF9287; EPID L-2017-LRO-0005)

Dear Mr. Williams:

By letters dated February 13, 2017 (Agencywide Documents Access and Management System Accession No. ML17045A145), Tennessee Valley Authority (TVA) submitted information summarizing the results of the fall 2016 steam generator tube inspections performed at the Sequoyah Nuclear Plant (Sequoyah), Unit 1. These inspections were performed during Refueling Outage 21.

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the information TVA provided and concludes it contains the information required by the Sequoyah, Unit 1, technical specifications and that no additional followup is required at this time. The NRC staff review of the steam generator portions of the report is enclosed.

If you have any questions, please contact me at (301) 415-8480 or [Andrew.Hon@nrc.gov](mailto:Andrew.Hon@nrc.gov).

Sincerely,

A handwritten signature in black ink, appearing to read "Andrew Hon".

Andrew Hon, Project Manager  
Licensing Project Branch II-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-327

Enclosure:  
NRC Review of Report

cc: Listserv

OFFICE OF NUCLEAR REACTOR REGULATION

REVIEW OF STEAM GENERATOR TUBE INSPECTION REPORT FOR CYCLE 21

SEQUOYAH NUCLEAR PLANT, UNIT 1

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

By letter dated February 13, 2017 (Agencywide Documents Access and Management System Accession No. ML17045A145), Tennessee Valley Authority (the licensee), submitted information summarizing the results of the fall 2016 steam generator (SG) tube inspections performed at Sequoyah Nuclear Plant, Unit 1, during Refueling Outage 21.

Sequoyah Unit 1 has four Model 57AG replacement SGs that were installed in 2003. Each SG contains 4,983 thermally treated Alloy 690 tubes, which have a nominal outside diameter of 0.75 inches and a nominal wall thickness of 0.043 inches. The tubes were hydraulically expanded into the tubesheet and are supported by several Type 409 stainless steel, lattice grid tube supports. The U-bend portion of the tubes is supported by diagonal and vertical straps.

The licensee provided the scope, extent, methods, and results of their SG tube inspections in the document referenced above. In addition, the licensee described corrective actions (i.e., tube plugging) taken in response to the inspection findings.

Based on a review of the information provided, the staff concludes that the licensee provided the information required by the applicable Technical Specifications 5.5.7, "Steam Generator Program," and Section 5.6.6, "Steam Generator Tube Inspection Report." In addition, the staff concludes that there are no technical issues that warrant followup action at this time since the inspections appear to be consistent with the objective of detecting potential tube degradation and the inspection results appear to be consistent with industry operating experience at similarly designed and operated units.

Enclosure

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNIT 1 - REVIEW OF THE STEAM  
GENERATOR TUBE INSERVICE INSPECTIONS FOR CYCLE 21 REFUELING  
OUTAGE IN THE FALL OF 2016 (CAC NO. MF9287; EPID L-2017-LRO-0005)  
DATED DECEMBER 4, 2017

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\*by memo ML13235A037

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