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AUTH.NAME AUTHOR AFFILIATION
ALEXICH,M.P. Indiana & Michigan Electric Co.
RECIP.NAME RECIPIENT AFFILIATION
DENTON,H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Application for amend to License DPR-58,revising plant heatup & cooldown curves to reflect reactor vessel matl surveillance capsule analyses,Fee paid.

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July 18, 1985
AEP:NRC:0894E

Donald C. Cook Nuclear Plant Unit No. 1
Docket No. 50-315
License No. DPR-58
TECHNICAL SPECIFICATION CHANGE REQUEST
HEATUP AND COOLDOWN CURVES

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Reference: Our letter No. AEP:NRC:0894C dated July 3, 1985.

Dear Mr. Denton:

This letter and its attachments constitute an application for amendment to the Technical Specifications for the Donald C. Cook Nuclear Plant Unit No. 1. Specifically, we request that the plant heatup and cooldown curves be changed to reflect the recent reactor vessel material surveillance capsule analyses performed by Southwest Research Institute (SWRI). These revised curves incorporate the reactor vessel beltline region weld chemistry data transmitted to your office by the above referenced letter. Our analysis concerning significant hazards considerations are contained in Attachment 1 to this letter. The proposed revised Technical Specification pages are contained in Attachment 2. These proposed changes supersede the changes requested for Unit No. 1 in our letter No. AEP:NRC:0894 dated February 14, 1985.

We believe that the proposed changes will not result in (1) a significant change in the types of effluents or a significant increase in the amounts of any effluent that may be released offsite, or (2) a significant increase in individual or cumulative occupational radiation exposure.

The revised heatup cooldown curves in Attachment No. 2 have been evaluated against the criterion of 10 CFR 50, Appendix G, Paragraph IV.A.2. The proposed revised curve reflects the Appendix G criterion.

The impact of this T/S change request on the cold overpressurization limits is currently being evaluated. As a result of the above evaluation, if there is any need to revise the cold overpressurizer T/S limits, it will be submitted for approval in the near future.

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The Reactor Vessel Material Surveillance Report prepared by SWRI for Unit No. 1 was transmitted to your office with our letter No. AEP:NRC:0894A dated July 20, 1984.

These proposed Technical Specifications will be reviewed by the Plant Nuclear Safety Review Committee and by the Nuclear Safety and Design Review Committee at their next scheduled meetings.

In compliance with the requirements of 10 CFR 50.91(b)(1), a copy of this letter and its attachments have been transmitted to Mr. R. C. Callen of the Michigan Public Service Commission and Mr. G. Bruchmann of the Michigan Department of Public Health.

Pursuant to 10 CFR 170.12(c), we have enclosed an application fee of $150.00 for the proposed amendments. We are requesting expedited review of the Technical Specification since it is required for initial plant heatup following the current refueling outage which is tentatively scheduled for the first week of August.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,

M. P. Alexich 985
Vice President 7/18/85

cm

Attachments

cc: John E. Dolan
W. G. Smith, Jr. - Bridgman
G. Bruchmann
R. C. Callen
G. Charnoff
NRC Resident Inspector - Bridgman