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 50-316 Donald C. Cook Nuclear Power Plant, Unit 2, Indiana & 05000316  
 AUTH. NAME AUTHOR AFFILIATION  
 HERING, R.F. Indiana & Michigan Electric Co.  
 RECIP. NAME RECIPIENT AFFILIATION  
 DENTON, H.R. Office of Nuclear Reactor Regulation, Director.

SUBJECT: Application for amends to Licenses DPR-58 & DPR-74, changing plant heatup & cooldown curves to reflect recent reactor vessel matl surveillance capsule analyses performed by SWRI.

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# INDIANA & MICHIGAN ELECTRIC COMPANY

P.O. BOX 16631  
COLUMBUS, OHIO 43216

February 14, 1985  
AEP:NRC:0894

Donald C. Cook Nuclear Plant Unit Nos. 1 and 2  
Docket Nos. 50-315 and 50-316  
License Nos. DPR-58 and DPR-74  
TECHNICAL SPECIFICATION CHANGE REQUEST  
HEATUP AND COOLDOWN CURVES

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Denton:

This letter and its attachments constitute an application for amendment to the Technical Specifications for the Donald C. Cook Nuclear Plant Unit Nos. 1 and 2. Specifically, we request that the plant heatup and cooldown curves be changed to reflect the recent reactor vessel material surveillance capsule analyses performed by Southwest Research Institute. Our analysis concerning significant hazards considerations are contained in Attachment 1 to this letter. The proposed revised Technical Specification pages are contained in Attachment 2.

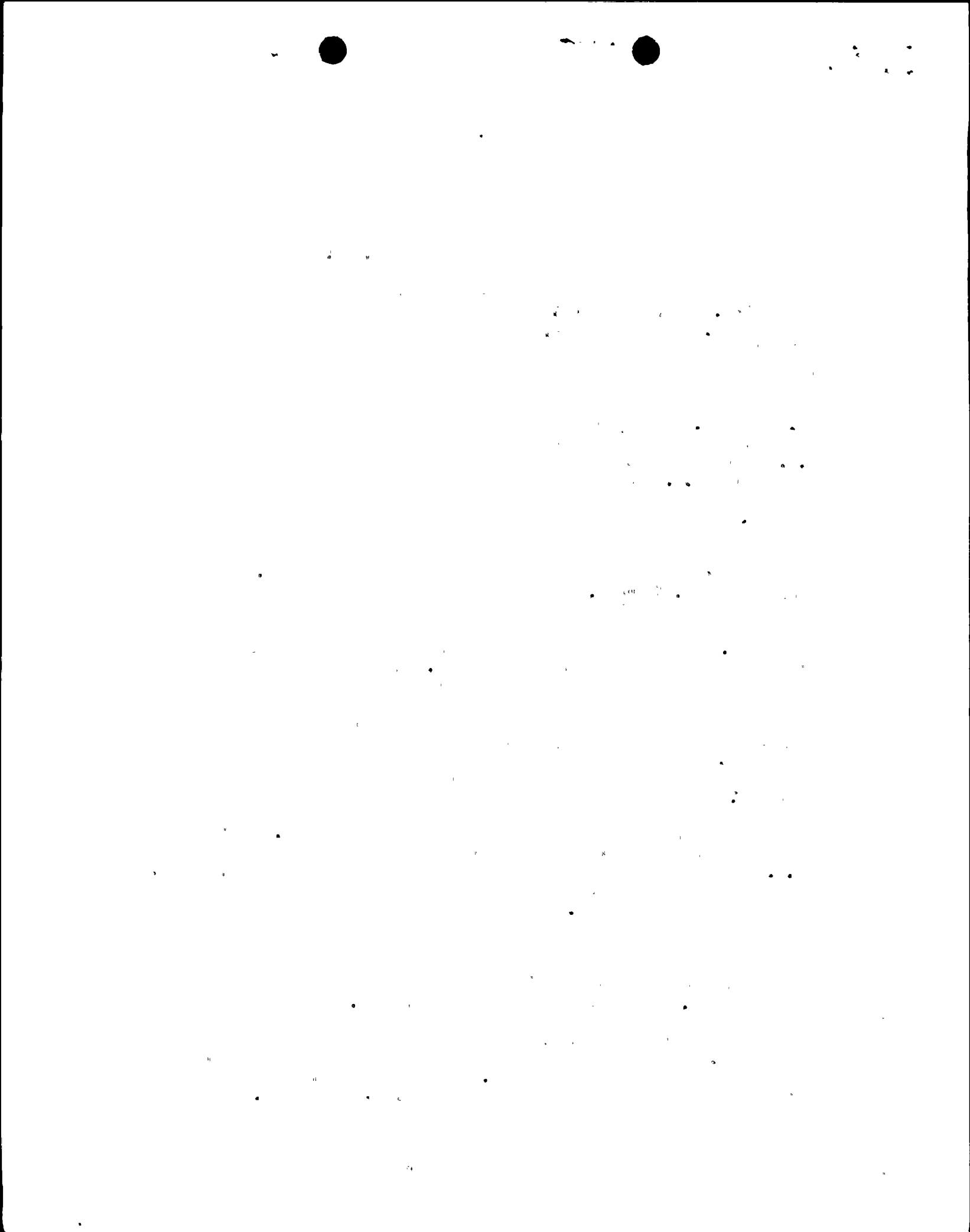
We believe that the proposed changes will not result in (1) a significant change in the types of effluents or a significant increase in the amounts of any effluent that may be released offsite, and (2) a significant increase in individual or cumulative occupational radiation exposure.

The revised heatup-cooldown curves in Attachment No. 2 have been evaluated against the criterion of 10 CFR 50, Appendix G, Paragraph IV.A.2. The proposed revised curve reflects the Appendix G criterion. The impact of this T/S change request on our cold overpressurization limits has been evaluated. This evaluation indicates that the existing limits are still adequate.

The Reactor Vessel Material Surveillance Reports prepared by Southwest Research Institute (SWRI) were transmitted to your office with our letter No. AEP:NRC:0894A dated July 20, 1984. Please note that the curves mentioned in the above reports do not reflect 10 CFR 50, Appendix G criterion, because at that time the revised curves were still under evaluation. We hereby request that the SWRI letter dated June 12, 1984, which is enclosed as Attachment No. 3 to this letter, be included in the earlier SWRI reports sent via our letter No. AEP:NRC:0894A.

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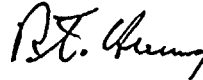
These proposed Technical Specifications have been reviewed by the Plant Nuclear Safety Review Committee and will be reviewed by the Nuclear Safety and Design Review Committee at their next scheduled meeting.

In compliance with the requirements of 10 CFR 50.91(b)(1), a copy of this letter and its attachments have been transmitted to Mr. R. C. Callen of the Michigan Public Service Commission and Mr. G. Bruchmann of the Michigan Department of Public Health.

Pursuant to 10 CFR 170.12(c), we have enclosed an application fee of \$150.00 for the proposed amendments.

This document has been prepared following Corporate procedures which incorporate a reasonable set of controls to insure its accuracy and completeness prior to signature by the undersigned.

Very truly yours,



R. F. Hering  
Vice President

ROK  
2/14/85

cm

Attachments

- cc: John E. Dolan (w/attachments)
- W. G. Smith, Jr. - Bridgman (w/attachments)
- M. P. Alexich (w/o attachments)
- G. Bruchmann (w/attachments)
- R. C. Callen (w/attachments)
- G. Charnoff (w/attachments)
- NRC Resident Inspector - Bridgman (w/attachments)

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