



NRC 2017-0051
10 CFR 50.73

November 16, 2017

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Unit 1 and 2
Docket 50-266 and 50-301
Renewed License Nos. DPR-24 and DPR-27

Licensee Event Report 266/2017-001-00
Control Room Barrier Inadvertently Disabled

Enclosed is Licensee Event Report (LER) 266/2017-001-00 for Point Beach Nuclear Plant, Unit 1 and 2. NextEra Energy Point Beach, LLC is providing this LER to report the event as an operation or condition prohibited by Technical Specifications, degraded or unanalyzed condition, and event or condition that could have prevented fulfillment of a safety function.

This letter contains no new regulatory commitments.

If you have any questions please contact Mr. Eric Schultz, Licensing Manager, at 920/755-7854.

Sincerely,

NextEra Energy Point Beach, LLC

A handwritten signature in blue ink, appearing to read "Bob Coffey".

Bob Coffey
Site Vice President

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW



LICENSEE EVENT REPORT (LER)

(See Page 2 for required number of digits/characters for each block)

(See NUREG-1022, R.3 for instruction and guidance for completing this form
<http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/>)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Information Services Branch (T-2 F43), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME Point Beach Nuclear Plant Unit 1	2. DOCKET NUMBER 05000266	3. PAGE 1 of 3
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4. TITLE
Control Room Barrier Inadvertently Disabled

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	18	2017	2017	001	00	11	16	2017	Point Beach Nuclear Plant Unit 2	05000301
									FACILITY NAME	DOCKET NUMBER
									NA	NA

9. OPERATING MODE	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)							
MODE 1	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)				
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input checked="" type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)				
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)				
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)				
10. POWER LEVEL Unit 1 / Unit 2 98% / 100%	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)				
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)				
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> 73.77(a)(1)				
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)(D)	<input type="checkbox"/> 73.77(a)(2)(i)				
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input checked="" type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> 73.77(a)(2)(ii)				
		<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> OTHER	Specify in Abstract below or in NRC Form 366A				

12. LICENSEE CONTACT FOR THIS LER

LICENSEE CONTACT Thomas P. Schneider, Senior Licensing Engineer	TELEPHONE NUMBER (Include Area Code) 920-755-7797
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX
X	NA	NA	NA	Y	NA	NA	NA	NA	NA

14. SUPPLEMENTAL REPORT EXPECTED	<input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	15. EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR
				NA	NA	NA

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

At 1724 [CDT] on 9/18/17 Door-061, South Control Room Door (DR) was inadvertently disabled. The door became wedged open against its backstop during control room ventilation testing. Door-061 is a barrier that functions to maintain the control room envelope (NA). The barrier was subsequently disengaged from the backstop allowing it to close. The door was inspected and returned to operable status at 1750 [CDT]. While the door was stuck open, the control room was in an unanalyzed condition, a condition that could have prevented fulfillment of a safety function, and a common cause inoperability of independent trains or channels.

This event is being reported pursuant to 10 CFR 50.73(a)(2)(ii)(B), 10 CFR 50.73(a)(2)(v)(A), 10 CFR 50.73(a)(2)(v)(D), and 10 CFR 50.73(a)(2)(vii) for a degraded barrier that affected the control room envelope.



**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

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Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Information Services Branch (T-2 F43), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

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		YEAR	SEQUENTIAL NUMBER	REV NO.
Point Beach Nuclear Plant Unit 1	05000266	2017	001	00

NARRATIVE

Description of the Event:

At 1724 [CDT] on 9/18/17 Door-061, South Control Room Door was inadvertently disabled. Door-061 is a barrier that functions to maintain the control room envelope during design basis accidents, events, and conditions (Safe Shutdown / High Energy Line Break / Fire / Flood). The barrier became wedged open against its backstop during control room ventilation testing. The barrier was subsequently disengaged from the backstop allowing it to close. The door was inspected and returned to operable status at 1750 [CDT]. While the door was stuck open, the control room was in an unanalyzed condition, a condition that could have prevented fulfillment of a safety function, and a common cause inoperability of independent trains or channels.

This event is being reported pursuant to 10 CFR 50.73(a)(2)(ii)(B), 10 CFR 50.73(a)(2)(v)(A), 10 CFR 50.73(a)(2)(v)(D), and 10 CFR 50.73(a)(2)(vii) for a degraded barrier that affected the control room envelope.

This 60 day licensee event report is being submitted in accordance with the following requirements;

10 CFR 50.73(a)(2)(ii)(B), any event or condition that results in the condition of the nuclear power plant being in an unanalyzed condition;

10 CFR 50.73(a)(2)(v)(A) and 10 CFR 50.73(a)(2)(v)(D), any event or condition that could have prevented fulfillment of a safety function of structures or systems needed to shut down the reactor and maintain it in a safe shutdown condition or mitigate the consequences of an accident;

10 CFR 50.73(a)(2)(vii), any event where a single cause or condition caused at least one independent train or channel to become inoperable in multiple systems or two independent trains or channels to become inoperable in a single system;

Where, the functions supported by the disabled barrier could not be found acceptable under all design basis accidents, events or conditions.

Cause of the Event:

The cause of the inadvertent degraded barrier was a less than adequate door stop design to control the barrier during ingress/egress activities of the control room envelope with a simultaneous differential pressure transient of the control room ventilation during testing.

Analysis of the Event:

Technical Specifications permit intermittent opening of the control room envelope boundaries under administrative controls. While the barrier was disabled, the boundary was unable to be rapidly closed. An engineering evaluation of the inadvertently disabled barrier determined the functions supported by the barrier could not be found acceptable under all design basis accidents, events and conditions. The less than adequate door stop design led to inadequate administrative control of the barrier during ingress/egress activities of the control room envelop with the simultaneous differential pressure transient of the control room ventilation during testing. The disabled barrier was promptly enabled, restoring the control room envelope support function.

Corrective Actions:

The disabled barrier was promptly enabled, restoring the control room envelope support function, and the door stop was modified with a more robust design. Additionally, an extent of condition review was completed and



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corrective actions initiated for other doors with susceptible conditions.

Safety Significance:

The event was determined to be of very low safety significance. The probability of an event occurring during the short time period is very low. While the condition was present, there was no loss of any safety systems, structures or components needed to shut down the reactor, maintain safe shutdown conditions, remove residual heat, control the release of radioactive material or mitigate the consequences of an accident. There was no impact on the health and safety of the public as a result of this condition.

Similar Events:

There have not been similar events of this condition in the past three years.

Component Failure Data:

None