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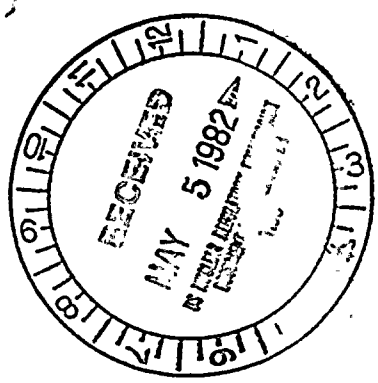

Docket Nos. 50-315
and 50-316

Mr. John Dolan, Vice President
Indiana and Michigan Electric Company
Post Office Box 18
Bowling Green Station
New York, New York 10004

Dear Mr. Dolan:

SUBJECT: UPGRADED SRO AND RO TRAINING AND TRAINING FOR MITIGATING CORE DAMAGE - REQUEST FOR ADDITIONAL INFORMATION

In our review of NUREG 0737 Item Nos. I.A.2.1 and II.B.4 for the D. C. Cook plants, we have identified additional information which we will need in order to complete our review. Science Applications, Inc. under contract to the NRC has developed the enclosed request for additional information. We request that you respond within 30 days of receipt of this letter. Please send a copy of your submittal directly to Dr. R. T. Liner, Science Applications, Inc., 1710 Goodridge Drive, McLean, Virginia 22102. This request for information is in accordance with the OMB Clearance No. 3150-0065, which expires May 31, 1983.

Sincerely,

Original signed by:
S. A. Varga

Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Enclosure:
Request for Additional
Information

cc w/enclosure:
See next page

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DATE	4/14/82	4/13/82	4/ /82				

Mr. John Dolan
Indiana and Michigan Electric Company

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U. S. Nuclear Regulatory Commission
Resident Inspectors Office
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William J. Scanlon, Esquire
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Ann Arbor, Michigan 48103

The Honorable Tom Corcoran
United States House of Representatives
Washington, D. C. 20515

James G. Keppler
Regional Administrator - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

ENCLOSURE

LICENSING ACTION REQUEST FOR ADDITIONAL INFORMATION

The U.S. Nuclear Regulatory Commission and its technical assistance contractor, Science Applications, Inc. (SAI), are performing a review to ascertain the acceptability of your response to certain requirements contained in post-TMI Action Items set forth in NUREG-0660 and NUREG-0737:

I.A.2.1 Immediate Upgrading of Reactor Operator and Senior Reactor Operator Training and Qualifications

II.B.4 Training for Mitigating Core Damage

Specifically, this review addresses the following items from Enclosure 1 of Harold Denton's letter of March 28, 1980, contained in NUREG-0737. Item A.2.c which addresses operator training requirements, item A.2.e which addresses instructor requalification, and Section C which addresses operator requalification. Some of these items are further elaborated in Enclosures 2, 3, and 4 of the Denton letter and in post-TMI Action Item II.B.4 (also in NUREG-0737).

Our review is presently based on your submittal of July 25, 1980, which includes the following:

1. Letter dated July 25, 1980, D. V. Shaller to Paul F. Collins (NRC).

2. Donald C. Cook Nuclear Plant Licensed Operator Requalification Program (11 pages, dated July 1980)

We assume these submittals reflect your current requalification program, but we have no information on your training program. Please submit your Senior Reactor Operator/Reactor Operator training programs. In particular, please address the following questions:

A. Training Program

1. Does your initial training program for Reactor Operator and Senior Reactor Operator cover the subjects of heat transfer, fluid flow, and thermodynamics? If it does, is the coverage to the level of detail spelled out in enclosure 2 of the Denton letter? Please send a course outline if available.

2. Does your initial training program for Reactor Operator and Senior Reactor Operator cover the subject of using installed plant systems to control or mitigate an accident in which the core is severely damaged? If it does, is the coverage to the level of detail spelled out in enclosure 3 of Denton's letter? Please send a course outline if available.

3. Are the lectures and quizzes on the subject of accident mitigation given to shift technical advisors and operating personnel from the plant manager through the operations chain to the licensed operators? If they are, would you please provide the titles of the people who are trained and an organization chart which illustrates their position in the operations chain?

4. Do the training program elements which involve heat transfer, fluid flow, thermodynamics and accident mitigation involve 80 contact hours in each program? (A contact hour of instruction is a one-hour period in which the course instructor is present or available for instructing or assisting students; lectures, seminars, discussions, problem-solving sessions, and examinations are considered contact periods under this definition.)

5. Is there an increased emphasis on reactor and plant transients in the reactor operator and senior rector operator training program as required by item A.2.C.3 of enclosure 1 of Denton's March 28, 1980, letter? If there is, does this training deal with both normal transients and abnormal (accident) transients?

In addition, please clarify your requalification program by providing answers to the following:

B. Requalification Program

1. The requalification program (1.a) for reactor operators and senior reactor operators has a lecture which appear to cover the subjects of heat transfer, fluid flow and thermodynamics as called out in enclosure 1 of Denton's March 28, 1980, letter. Does this lecture in fact cover this material and is the coverage to the level of detail spelled out in enclosure 2 of the Denton letter? Please send a course outline if it is different from that used in the training program.

2. The reactor operator and senior reactor operator qualification program (1.f) has a lecture which appears to address the subject of using installed plant systems to control or mitigate an accident in which the core is severely damaged. Does this lecture address the topic and does it cover the subject to the level of detail spelled out in enclosure 3 of Denton's letter? Please send a course outline if it is different from that used in the training program.

3. Are your instructors enrolled in appropriate requalification programs to assure they are cognizant of current operating history, problems, and changes to procedures and administrative limitations?

4. The reactor operator requalification program identifies twenty-six reactor control manipulations which are nearly the same as those identified in enclosure 4 of the Denton letter. Will this list be expanded to address item (8) as called out in enclosure 4 of Denton's letter? If not, please provide a rationale for the exceptions.

5. Please answer question 3 of Section A with respect to your requalification program.

6. Please answer question 4 of Section A with respect to your requalification program.