

A. Chronology

Operating

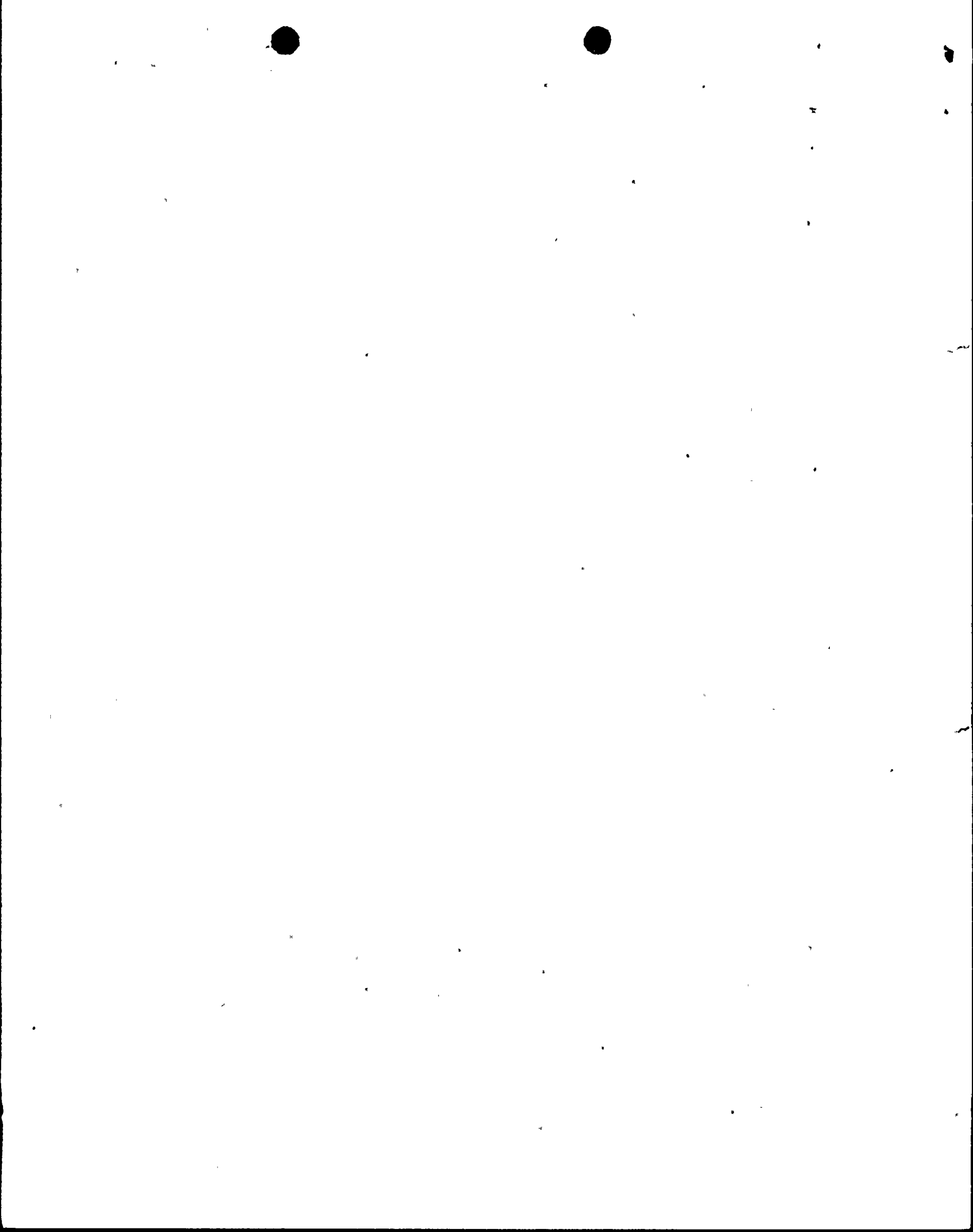
Unit 1: The unit operated at essentially 100% with minor outages and reductions for repairs and surveillance until the Cycle III-IV refueling outage which started April 6, 1979. The outage was extended to perform testing of anchor bolts per IE Bulletin 79-02, and repairs to feedwater pipe cracks discovered on May 19, 1979. The unit was returned to service on July 18th and again operated almost continuously at full power until shutdown on October 27th due to inverter malfunction and reactor coolant pump coupling misalignment. The unit returned to service November 11 and continued at full power until December 24 when shutdown as a result of significant nonconformances identified relative to IEB 79-14. The unit remained shutdown through the end of the year.

Unit 2: The unit operated at essentially 100% with minor outages and reductions for repairs and surveillance until May 19 when shutdown for repairs to cracks in feedwater piping. The unit was returned to service July 3 and again operated at virtually full power through October 19 when shutdown for Cycle I-II refueling. This major outage was extended through the end of the year as a result of significant nonconformances identified relative to IEB 79-14.

Inspection and Enforcement

March 30, 1979- A corporate management meeting was held to discuss trends in the Licensee's performance.

July 31, 1979- A Corporate management meeting was held to discuss a schedule, techniques, and procedure for replacement of feedwater elbow welds on Unit 2. Also discussed were areas of disagreement and concern regarding welds in question, and results of analysis of the nozzle cracks.



Inspection/Branch	Noncompliance	Sanction Points	Type of Noncompliance
79-02/02 Safeguards	1 Infraction 2 Deficiencies	10 4	Security Security
79-03/03 FF&MS	1 Infraction	10	Failure to conduct drill.
79-04/04 RO & NS	1 Infraction	10	Failure to implement procedures
79-05/05 RO & NS	2 Infractions	20	Failure to follow procedures.
79-07/06 RO & NS	1 Infraction	10	Security
79-11/08 RO & NS	2 Infraction	20	Failure to follow procedures PNSRC Review
79-13/10 RO & NS	4 Infractions	40	Failure to follow procedures PNSRC Review
79-14/11 FF & MS	2 Infractions	20	Inadequate air sampling Inadequate follow on WBC.
79-15/12 RC & ES	1 Infraction	10	Failure to follow procedures.
79-17/13 FF & MS	1 Infraction	10	Chlorine Discharge
79-19/15 Safeguards	1 Infraction 2 Deficiencies	10 4	Security Security
79-23/20 RO & NS	2 Infractions	20	Failure to follow procedures PNSRC failure to review
79-24/21 RO & NS	2 Infractions	20	Failure to follow procedures
79-26/24 FF & MS	1 Infraction	10	Solid radioactive waste handling.
79-28 RO & NS	1 Deficiency	2	Failure to follow procedures

Summary

RO & NS Totals

Infractions	14	
Deficiencies	2	
Sanction Points	142	(1978 Total - 224)

RC & ES Totals

Infractions	1	
Deficiencies	0	
Sanction Points	10	(1978 Total - 10)

FF & MS Totals

Infractions	5	
Deficiencies	0	
Sanction Points	50	(1978 Total - 12)

Safeguards

Infractions	2	
Deficiencies	4	
Sanction Points	28	(1978 Total - 42)

1979 Total	230	(1978 Total - 288)
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C. Inspector's Evaluation of Licensee Performance, Problem Areas, And Recommendations for Next Year.

The licensee has shown improvement this past year in terms of total number of LER's and items of noncompliance. Response to TMI "Lessons Learned" and the resulting upgrading of training, design, etc., is generally favorable. Continued emphasis must be placed on reducing the number and percentage of personnel errors through improved coordination/communication, attention to detail, and better integrated systems training. Administratively, the inspector expects to see vast improvement in the timely closeout of Condition Reports and audit Corrective Action Requests.

D. Significant Personnel Errors

Unit 1

February 18, 1979 (LER 79-007/03L-0); Charging Pump Suction Valve left closed following surveillance test.

February 23, 1979 (LER 79-009/03L-0); Both Diesel-Generators inoperable on two separate occasions, once for 1 hour and once for 7 minutes.

March 4, 1979 (LER 79-016/03L-0); Both Containment Isolation Valves used for instrument sampling left open.

November 15, 1979 (LER 79-058/03L-0); Two of three auxiliary feed-water flow paths made inoperable at same time during maintenance.

Unit 2

October 17, 1979 (LER 79-035/03L-0); Two of three auxiliary feed-water flow paths made inoperable at same time during maintenance.

November 14, 1979 (LER 79-038/03L-0); Boron injection flow path made inoperable when being used as charging pump boration flowpath during maintenance.

November 1, 1979 (LER 79-042/03L-0); Both diesel-generators made inoperable at same time during modification and termination of cable.

November 13, 1979 (LER 79-045/03L-0); Containment integrity breached during refueling operations due to maintenance activities.

E. Summary of Unplanned Radioactive Releases

Unit 1 - None

Unit 2 - .

Event Date: February 24, 1979

Event Number: 79-001/04X-0

Event Description: Unplanned release of low level gaseous activity from a moisture separator reheater (MSR) safety valve, which lifted at a setpoint lower than normal for no apparent reason. The release lasted for approximately 35 minutes and involved Sodium - 24 contaminated steam. The N_a^{24} was being used at the time as a tracer in the secondary system to identify leaks. Actual release concentration was calculated to be 1.15×10^{-11} Ci/cc N_a^{24} . The applicable Technical Specification limit for N_a^{24} in air to an unrestricted area is 5×10^{-9} Ci/cc. The total release was 6.99×10^{-6} Ci at a rate which was 0.23% of allowed Technical Specification limits.

F. Number and Character of Reportable Events

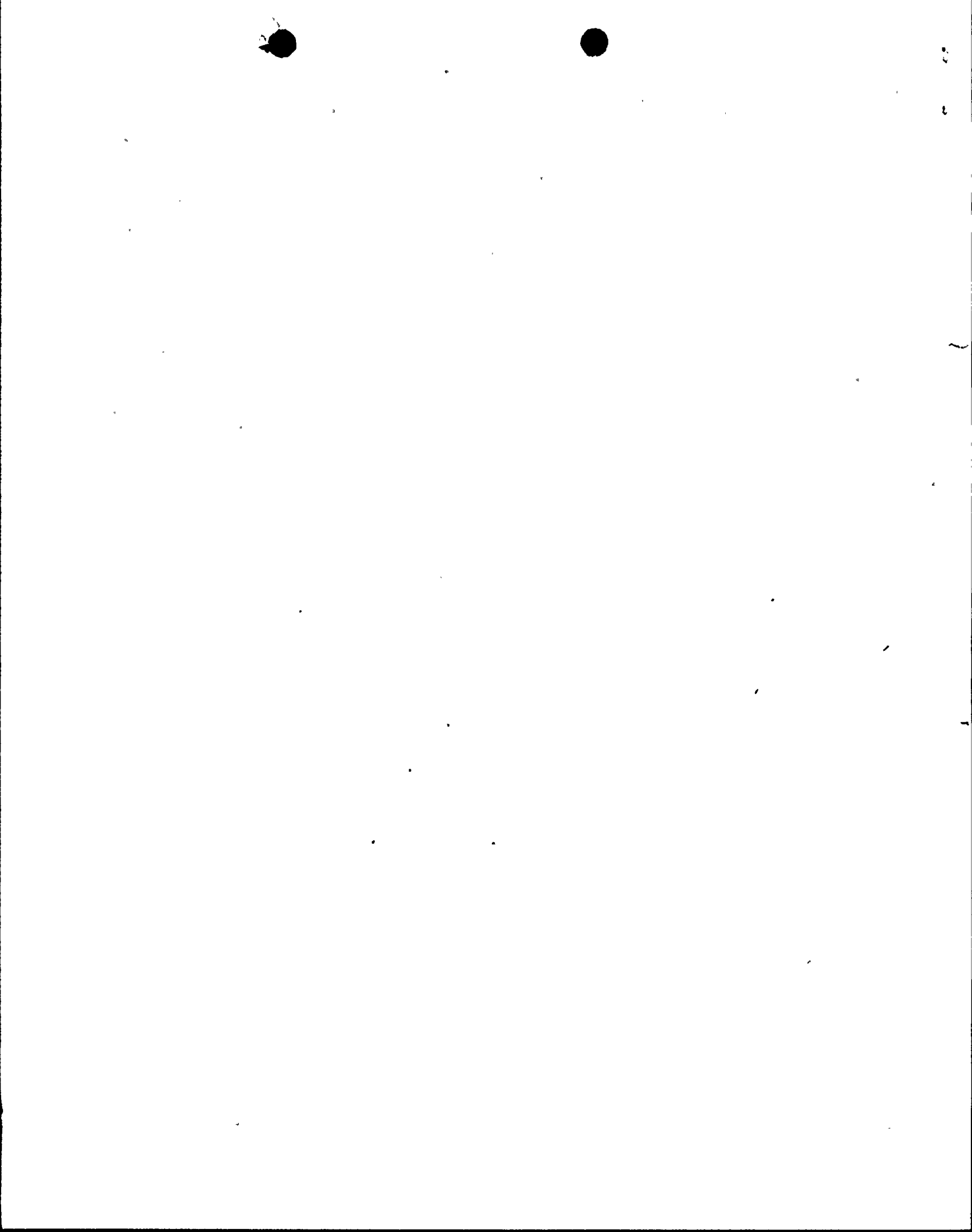
1. 1979 Totals: Unit 1 - 67
 Unit 2 - 54
 Total - 121

2. <u>Type</u>	<u>Personnel Error</u>	<u>Procedure Deficiency</u>	<u>MGMt Control</u>	<u>Total</u>
Exceeding Safety Limit	0	0	0	0
Significant Release or Overexposure	0	0	0	0
Loss of Entire Engineered Safeguards	0	0	0	0
Major Breakdown of Management Controls Resulting in Degraded Plant Safety	0	0	0	0

Control Rod Mismanipulations	0	0	0	0
Release or Overexposure Slightly Above Limits	0	0	0	0
Does Not Exceed LCO but Results in Potential Degradation of Reactor Systems or Structures	0	0	0	0
Loss of Portion of Safety System or Engineered Safeguards	6	0	0	6
Reduction in Effectiveness of Safety System or Engineered Safeguard	4	0	0	4
Unscheduled Release Below Limits:	0	0	0	0
1. Potential to exceed Limit	0	0	0	0
2. No Potential to exceed Limit	0	0	0	0

G. Summary of Inspections

<u>Report Number</u>	<u>Manhours Onsite</u>	<u>Type Inspection</u>	<u>Unannounced</u>
50-315/79-01 50-316/79-01	20	Radiological Environmental Protection	Yes
50-315/79-02 50-316/79-02	118	Security	No
50-315/79-03 50-315/79-03	27	Emergency Plan	No
50-315/79-04 50-315/79-04	159	Resident Inspection	Yes
50-315/79-05 50-316/79-05	197	Resident Inspection	Yes
50-315/79-06	28	Inservice Inspection	Yes
50-315/79-07 50-315/79-06	2	Security	Yes



50-315/79-08	10	Snubbers	Yes
50-315/79-09	32	Investigation of Allegations	No
50-315/79-10 50-316/79-07	28	Followup on Feedwater Cracks	No
50-315/79-11 50-316/79-08	96	Resident Inspection	Yes
50-315/79-12	15	IEB 79-02 Followup	Yes
50-315/79-13 50-316/79-10	230	Resident Inspection TMI Followup	Yes
50-315/79-14 50-316/79-11	43	Radiation Protection Program	Yes
50-315/79-15 50-316/79-12	168	Followup on Feedwater Cracks	No
50-315/79-16 50-316/79-09	5	Followup on IEC 78-08	Yes
50-315/79-17 50-316/79-13	19	Nonradiological Environmental Protection Program	Yes
50-315/79-18 50-316/79-14	70	Resident Inspection	Yes
50-315/79-19 50-316/79-15	138	Security	Yes
50-316/79-16	27	Management Meeting on Feedwater Cracks	No
50-315/79-20 50-316/79-17	16	Followup on IEB 79-02, 79-04, 79-14	Yes
50-315/79-21 50-316/79-18	87	Resident Inspection	Yes
50-315/79-22 50-316/79-19	7	Followup on IEB 79-04, 79-14	Yes
50-315/79-23 50-316/79-20	98	Resident Inspection	Yes

50-315/79-24 50-316/79-21	134	Resident Inspection	Yes
50-315/79-25 50-316/79-23	36	Investigation of Allegations.	Yes
50-316/79-22	32	Inservice Inspection	No
50-315/79-26 50-316/79-24	34	Rad Waste System	No
50-315/79-27 50-316/79-27	32	Investigation of Allegations	No
50-315/79-28	22	Reactor Physics	Yes

Inspection numbers 79-25 and 79-26 on Docket 50-316 were not used. Inspection number 79-27 was the final inspection for this Docket.

