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 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
 AUTH. NAME AUTHOR AFFILIATION
 WIDAY, J.A. Rochester Gas & Electric Corp.
 RECIP. NAME RECIPIENT AFFILIATION
 VISSING, G.S.

Rev 5/10/97 W

SUBJECT: Forwards revised Emergency Operating Procedures for plant.

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JOSEPH A. WIDAY
Plant Manager
Ginna Nuclear Plant

TELEPHONE
AREA CODE 716 546-2700

January 23, 1997

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I-1
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,

Joseph A. Widay
Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s): 030021

E Index	E-1, Rev. 14	ES-1.3, Rev. 20
ECA Index	E-3, Rev. 19	FR-C.1, Rev. 13
ES Index	ECA-2.1, Rev. 14	FR-C.2, Rev. 12
FR Index	ECA-3.3, Rev. 15	FR-P.1, Rev. 14

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PDR ADDCK 05000244
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ADD 2



REPORT NO. 01
REPORT: NPSPO200
DOC TYPE: PRES

GINNA NUCLEAR POWER PLANT
PROCEDURES INDEX
EQUIPMENT SUB-PROCEDURE

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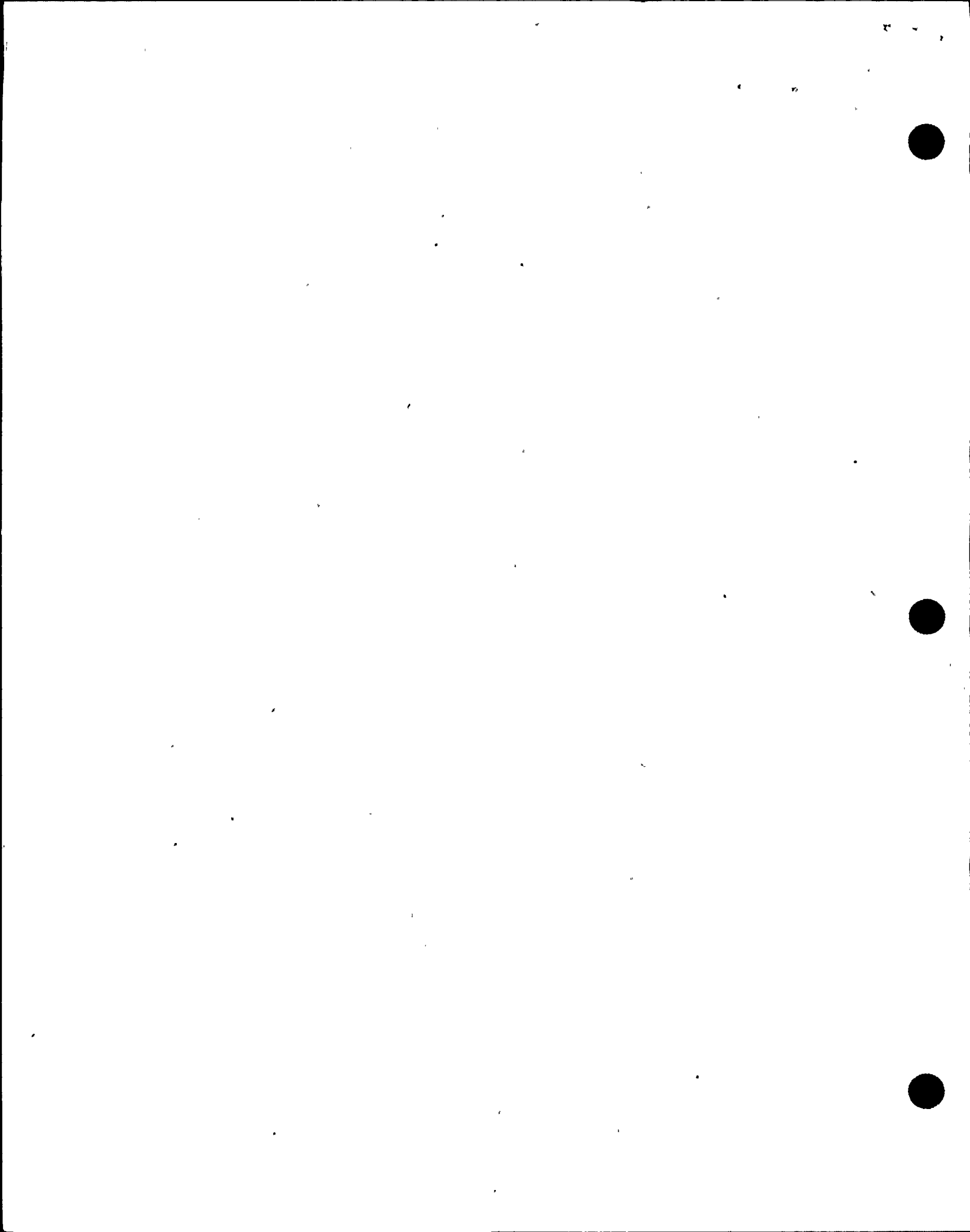
PARAMETERS: DOC TYPES - PRER PRES PRFR

STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
ES-0.0	REDIAGNOSIS	009	04/07/94	04/07/94	04/07/99	EF
ES-0.1	REACTOR TRIP RESPONSE	012	06/03/96	04/07/94	04/07/99	EF
ES-0.2	NATURAL CIRCULATION COOLDOWN	009	06/03/96	04/20/95	04/20/00	EF
ES-0.3	NATURAL CIRCULATION COOLDOWN WITH STEAM VOID IN VESSEL	006	06/03/96	04/07/94	04/07/99	EF
ES-1.1	SI TERMINATION	012	06/03/96	04/20/95	04/20/00	EF
ES-1.2	POST LOCA COOLDOWN AND DEPRESSURIZATION	014	06/03/96	04/20/95	04/20/00	EF
ES-1.3	TRANSFER TO COLD LEG RECIRCULATION	019	01/10/97	03/26/93	03/26/98	EF
ES-3.1	POST-SGTR COOLDOWN USING BACKFILL	010	06/03/96	04/20/95	04/20/00	EF
ES-3.2	POST-SGTR COOLDOWN USING BLOWDOWN	011	06/03/96	04/20/95	04/20/00	EF
ES-3.3	POST-SGTR COOLDOWN USING STEAM DUMP	011	06/03/96	04/20/95	04/20/00	EF
TOTAL FOR PRES	10					

*Superseded P&P PW Rev
to S&P 1/23/97
9702030046
50-244*

23



REPORT NO. 01
REPORT NAME: DOC-P001
DOC TYPE: PRF

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CRITICAL SAFETY FUNCTION STATUS PROC

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PARAMETERS: DOC TYPES - PRES PRF

PRFR PRPT PRPTT STATUS:

5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
F-0.1	SUBCRITICALITY CSFST	001	07/21/89	07/13/94	01/13/97	EF
F-0.2	CORE COOLING CSFST	003	07/21/89	07/13/94	01/13/97	EF
F-0.3	HEAT SINK CSFST	002	01/12/90	07/13/94	01/13/97	EF
F-0.4	INTEGRITY CSFST	001	07/21/89	07/13/94	01/13/97	EF
F-0.5	CONTAINMENT CSFST	002	01/12/90	07/13/94	01/13/97	EF
F-0.6	INVENTORY CSFST	003	01/12/90	07/13/94	01/13/97	EF

TOTAL FOR PRF 6

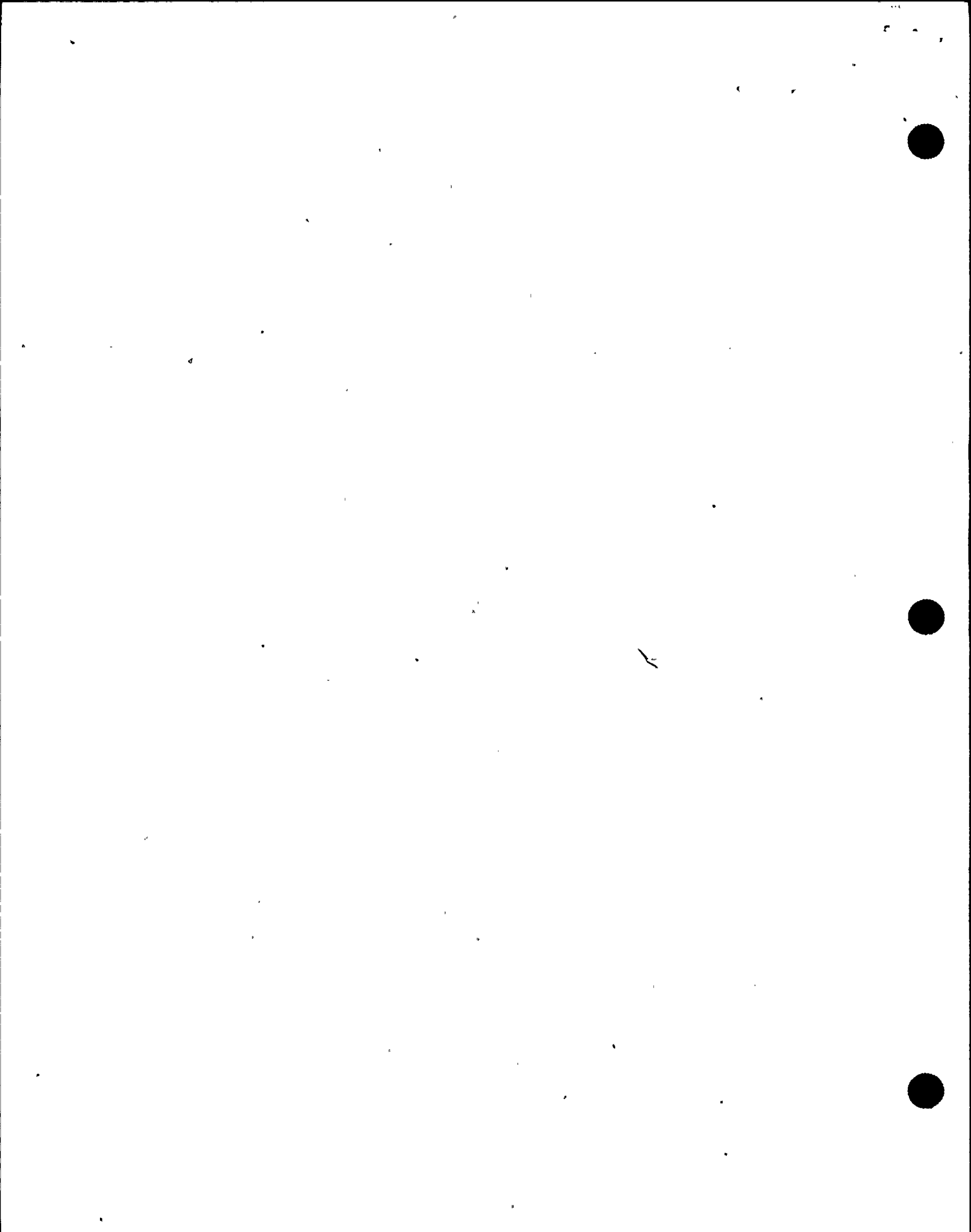


PARAMETERS: DOC TYPES - PRER PRES PRFR STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
FR-C.1	RESPONSE TO INADEQUATE CORE COOLING	012	01/10/97	04/07/94	04/07/99	EF
FR-C.2	RESPONSE TO DEGRADED CORE COOLING	011	01/10/97	04/07/94	04/07/99	EF
FR-C.3	RESPONSE TO SATURATED CORE COOLING	007	01/10/97	03/02/95	03/02/00	EF
FR-H.1	RESPONSE TO LOSS OF SECONDARY HEAT SINK	015	11/08/96	04/20/95	04/20/00	EF
FR-H.2	RESPONSE TO STEAM GENERATOR OVERPRESSURE	003	05/31/96	02/22/95	02/22/00	EF
FR-H.3	RESPONSE TO STEAM GENERATOR HIGH LEVEL	004	05/31/96	02/22/95	02/22/00	EF
FR-H.4	RESPONSE TO LOSS OF NORMAL STEAM RELEASE CAPABILITIES	003	05/31/96	04/21/93	04/21/98	EF
FR-H.5	RESPONSE TO STEAM GENERATOR LOW LEVEL	003	09/26/96	02/22/95	02/22/00	EF
FR-I.1	RESPONSE TO HIGH PRESSURIZER LEVEL	007	05/31/96	04/20/95	04/20/00	EF
FR-I.2	RESPONSE TO LOW PRESSURIZER LEVEL	004	05/03/91	07/28/93	07/28/98	EF
FR-I.3	RESPONSE TO VOIDS IN REACTOR VESSEL	008	06/05/96	10/19/94	10/19/99	EF
FR-P.1	RESPONSE TO IMMINENT PRESSURIZED THERMAL SHOCK CONDITION	013	11/08/96	04/20/95	04/20/00	EF
FR-P.2	RESPONSE TO ANTICIPATED PRESSURIZED THERMAL SHOCK CONDITION	005	06/05/96	11/08/94	11/08/99	EF
FR-S.1	RESPONSE TO REACTOR RESTART/ATWS	010	05/31/96	04/20/95	04/20/00	EF
FR-S.2	RESPONSE TO LOSS OF CORE SHUTDOWN	006	05/31/96	04/20/95	04/20/00	EF
FR-Z.1	RESPONSE TO HIGH CONTAINMENT PRESSURE	003	10/16/92	02/22/95	02/22/00	EF
FR-Z.2	RESPONSE TO CONTAINMENT FLOODING	002	04/09/90	02/22/95	02/22/00	EF
FR-Z.3	RESPONSE TO HIGH CONTAINMENT RADIATION LEVEL	003	09/30/92	02/22/95	02/22/00	EF

TOTAL FOR PRFR

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REPORT NO. 01
REPORT NAME: DOC-P001
DOC TYPE: PRE

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EMERGENCY PROCEDURE

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PARAMETERS: DOC TYPES - PRAP

PRATT

PRE

PRECA

PRER

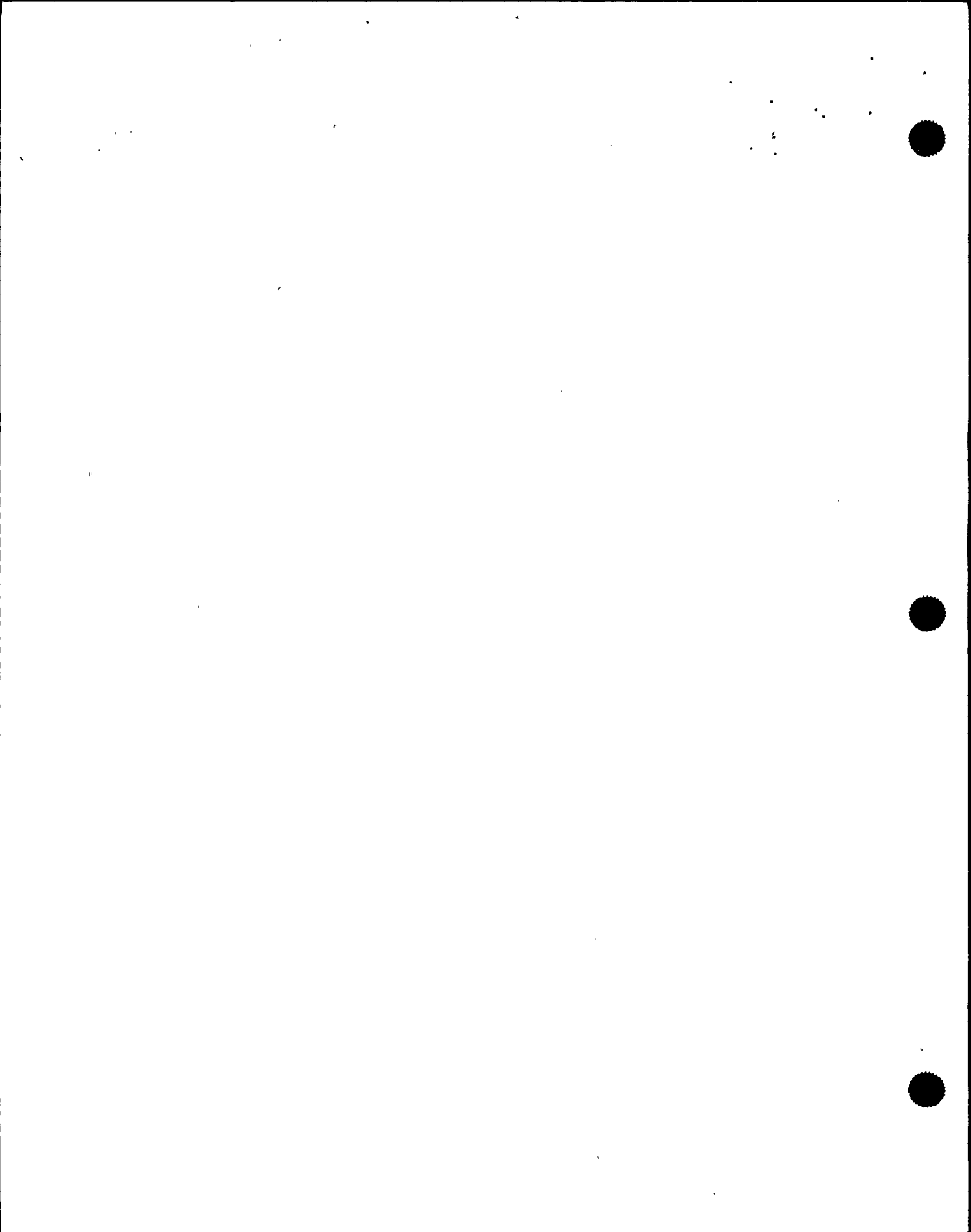
STATUS:

5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
E-0	REACTOR TRIP OR SAFETY INJECTION	019	04/07/94	04/07/94	10/07/96	EF
E-1	LOSS OF REACTOR OR SECONDARY COOLANT	011	03/26/93	03/26/93	09/26/95	EF
E-2	FAULTED STEAM GENERATOR ISOLATION	006	07/08/93	07/08/93	01/08/96	EF
E-3	STEAM GENERATOR TUBE RUPTURE	015	06/09/94	04/07/94	10/07/96	EF

TOTAL FOR PRE

4



REPORT NO. 01
REPORT NAME: DOC-P001
DOC TYPE: PRECA

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PARAMETERS: DOC TYPES - PRAP	PRATT	PRE	PRECA	PRER	STATUS:	5 YEARS ONLY:				
PROCEDURE NUMBER	PROCEDURE TITLE					REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
ECA-0.0	LOSS OF ALL AC POWER					016	11/08/94			SS
ECA-0.1	LOSS OF ALL AC POWER RECOVERY WITHOUT SI REQUIRED					010	06/09/94	04/07/94	10/07/96	EF
ECA-0.2	LOSS OF ALL AC POWER RECOVERY WITH SI REQUIRED					008	04/07/94	04/07/94	10/07/96	EF
ECA-1.1	LOSS OF EMERGENCY COOLANT RECIRCULATION					009	04/07/94	04/07/94	10/07/96	EF
ECA-1.2	LOCA OUTSIDE CONTAINMENT					002	04/09/90	09/23/92	03/23/95	EF
ECA-2.1	UNCONTROLLED DEPRESSURIZATION OF BOTH STEAM GENERATORS					010	06/09/94	04/07/94	10/07/96	EF
ECA-3.1	SGTR WITH LOSS OF REACTOR COOLANT-SUBCOOLED RECOVERY DESIRED					010	04/07/94	04/07/94	10/07/96	EF
ECA-3.2	SGTR WITH LOSS OF REACTOR COOLANT-SATURATED RECOVERY DESIRED					013	04/07/94	04/07/94	04/07/96	EF
ECA-3.3	SGTR WITHOUT PRESSURIZER PRESSURE CONTROL					011	06/09/94	04/07/94	10/07/96	EF
TOTAL FOR PRECA	9									

