

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

March 25, 1997

50-335  
50-389

Mr. T. F. Plunkett  
President - Nuclear Division  
Florida Power and Light Company  
Juno Beach, FL 33408-0420

SUBJECT: ST. LUCIE NUCLEAR PLANTS UNITS 1 AND 2 DESIGN INSPECTION (NRC  
INSPECTION REPORT NOS. 50-335/96-201 AND 50-389/96-201)

Dear Mr. Plunkett:

During the period from November 18, 1996 through January 10, 1997 the U.S. Nuclear Regulatory Commission's (NRCs) Office of Nuclear Reactor Regulation (NRR) performed a design inspection of the St. Lucie Unit 1 Auxiliary Feedwater System (AFW) and the Unit 2 Component Cooling Water (CCW) System. The purpose of the inspection was to evaluate the capability of the systems to perform safety functions required by their design basis, adherence to the design and licensing basis, and consistency of the as-built configuration with the Updated Final Safety Analysis Report.

The results of this inspection are contained in the attached inspection report. Overall, the team found the design of the two selected systems to be good, with adequate design margins. Your staff's understanding of the design basis was good, as was their inspection preparation and ability to resolve team identified concerns. The implementation of the design was found to be adequate with some concerns noted.

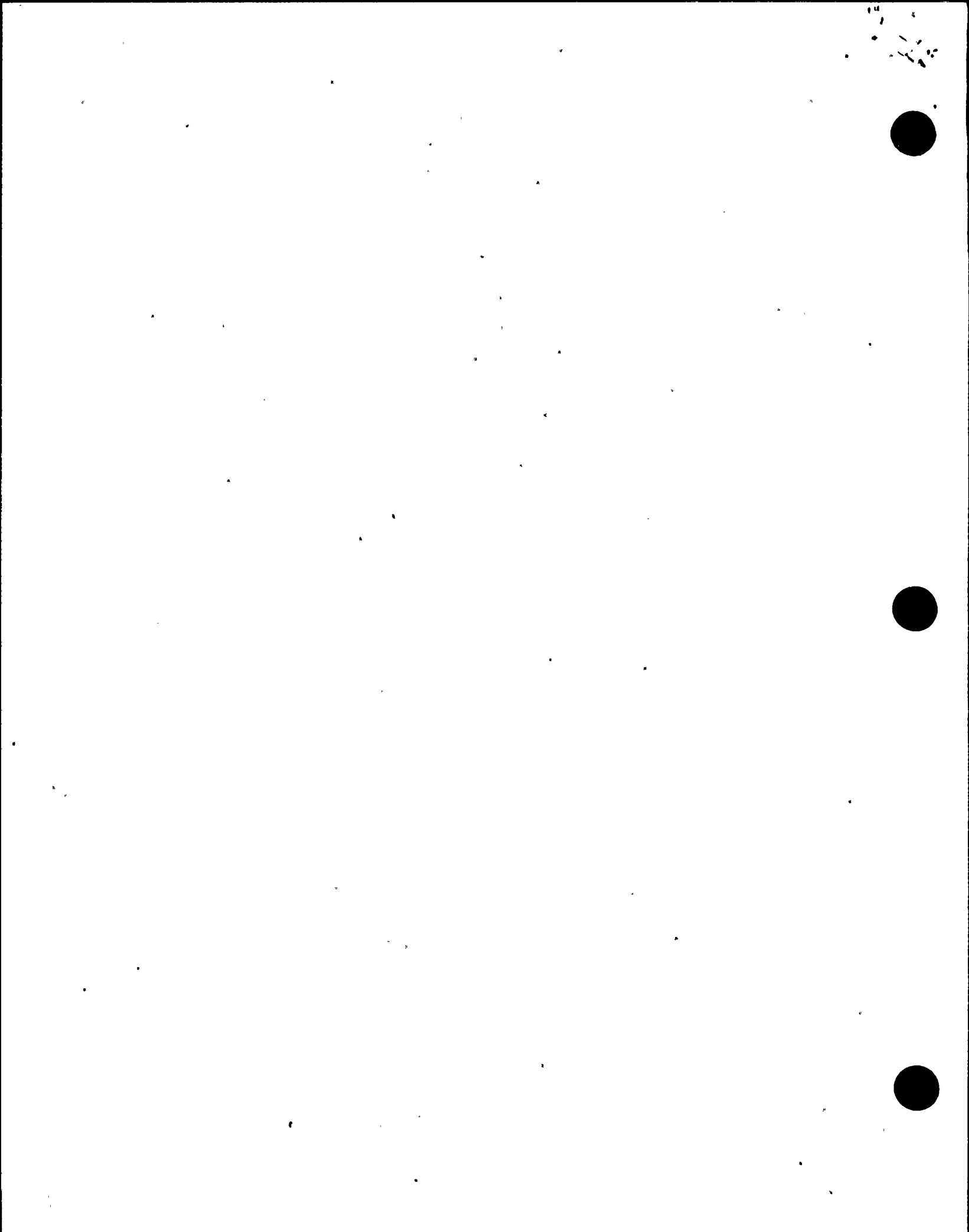
While none of the team's findings resulted in system inoperability, some errors made during the original plant design have reduced system operating margins. Of specific concern are the calculations which support operation of the component cooling water system. The current calculations for determining the temperature limit for the seawater intake to the component cooling water heat exchangers are non-conservative. Your interim actions to establish a 82 °F temperature limit on intake cooling water are adequate for the short term, but plant operation could be challenged by higher intake cooling water temperatures that occur during the warmer months of the year.

A few of the team's findings were attributed to weaknesses in testing or surveillance activities, including an issue identified by your staff just prior to the inspection concerning a lack of operating procedures and testing of certain safety related DC circuit breakers. A review of troubleshooting activities related to the above circuit breakers revealed that written documentation was inadequate to ensure an appropriate level of plant and personnel safety.

The team noted that your staff has implemented corrective action for many of the specific findings identified in the report. Please provide a schedule, within 60 days of the date of this letter, for completion of your corrective actions for the items listed in Attachment 1 to the enclosed report, so that we can plan for re-inspection of these items.

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Mr. T. F. Plunkett

-2-

As with all NRC inspections, we expect that you will evaluate the results of this inspection, and where applicable, apply the specific findings to other systems and components.

In accordance with 10 CFR 2.790 of the Commission's regulations, a copy of this letter and inspection report will be placed in the NRC Public Document Room. Any enforcement action resulting from this inspection will be issued by the NRC Region II office via a separate correspondence. Should you have any questions concerning the attached inspection report, please contact the inspection team leader Mr. Jeffrey B. Jacobson at (301) 415-2977.

Sincerely,



Robert M. Gallo, Chief  
Special Inspection Branch  
Division of Inspection and Support Programs  
Office of Nuclear Reactor Regulation

Enclosure: Inspection Report No. 50-335/96-201  
and 50-389/96-201

cc: See next page

Mr. T. F. Plunkett  
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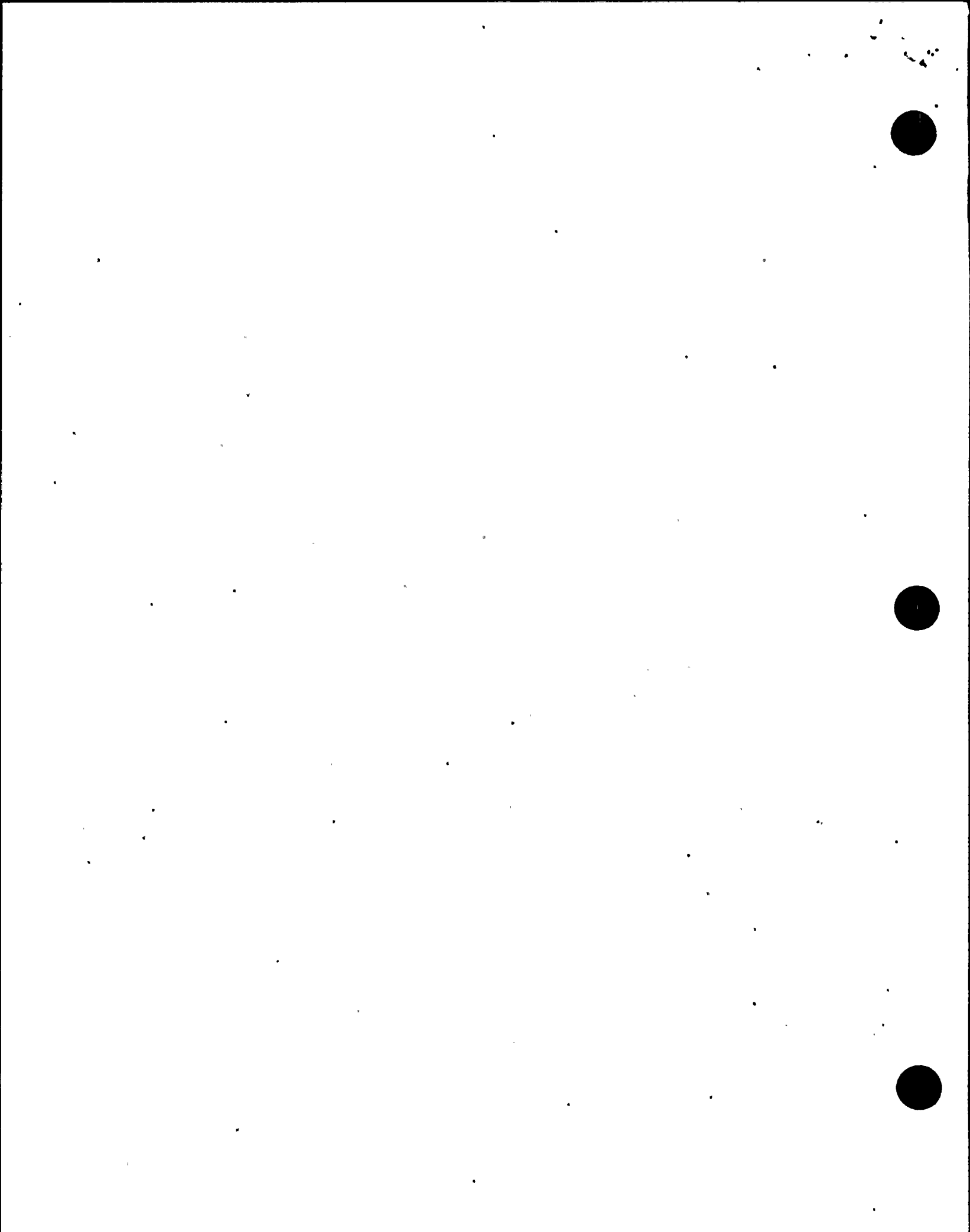
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