ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9207270285 DOC.DATE: 92/02/12 NOTARIZED: NO DOCKET #
FACIL:50-335 St. Lucie Plant, Unit 1, Florida Power & Light Co. 05000335
50-389 St. Lucie Plant, Unit 2, Florida Power & Light Co. 05000389
AUTH.NAME AUTHOR AFFILIATION
* Florida Power & Light Co.
RECIP.NAME RECIPIENT AFFILIATION

See Rpt.

SUBJECT: "St Lucie Nuclear Power Plant 1992 Emergency Preparedness Evaluated Exercise,920212."

DISTRIBUTION CODE: IE35D COPIES RECEIVED:LTR 0 ENCL 1 SIZE: 500
TITLE: Emergency Preparedness-Appraisal/Confirmatory Action Ltr/Exercise Rep

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL
	PD2-2 PD	1	1	NORRIS,J	1	1
INTERNAL:	AEOD/DOA/IRB	1	1	NRR/DREP/PEPB9D	1	1
	NRR/PMAS/ILRB12	1	1	NUDOCS-ABSTRACT	1	1
	OC/LEMB	1	0	OE DIR	1	1
	REG FILE 02	1	1	RGN2 FILE 01	1	1
EXTERNAL:	NRC PDR	1	1	NSIC	1	1

R I D S / A D D S

R I D S / A D D S

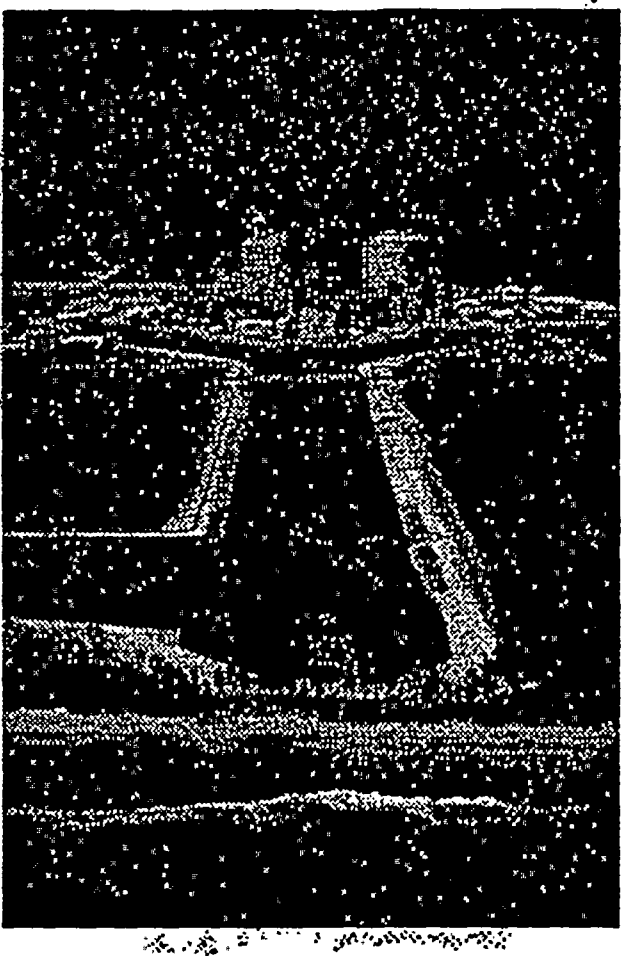
NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK, ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 12 ENCL 11

mAf

FPL



St. Lucie
Nuclear Power Plant

1992 EMERGENCY PREPAREDNESS EVALUATED EXERCISE FEBRUARY 12, 1992

CONFIDENTIAL

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

TABLE OF CONTENTS

TABLE OF CONTENTS (Continued)

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

1.0 INTRODUCTION

This Scenario Section contains the following information:

Section 1.1 FOREWORD - A brief introduction to the basis for the exercise and the layout of the scenario data.

Section 1.2 SCHEDULE OF EVENTS - Date, time, location, purpose and participants for the following meetings and critiques for exercise performance and evaluation.

1.2.1 Player Briefing
1.2.2 Nuclear Regulatory Commission Entrance Briefing
1.2.3 Exercise Scenario Review
1.2.4 Controller/Evaluator Training
1.2.5 Nuclear Regulatory Commission Plant Tour
1.2.6 Exercise
1.2.7 Facility Critiques
1.2.8 Florida Power and Light Controller/Evaluator Critique
1.2.9 Florida Power and Light Management Critique/Nuclear Regulatory Commission Exit

Section 1.3 PARTICIPATING AGENCIES - A listing of the agencies from the following areas that will be participating in the performance of the exercise.

1.3.1 Utility
1.3.2 Federal
1.3.3 State
1.3.4 County
1.3.5 Volunteer Agencies
1.3.6 Support Organizations

Section 1.4 CONTROLLER ORGANIZATION - A listing of the FPL Controllers and Controller assignments for the exercise.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

1.1 FOREWORD

In the interest of assuring that the health and safety of the general public is protected in the event of an accident at St. Lucie Nuclear Plant (PSL), and to meet the requirements of 10 CFR 50, Appendix E, the Florida Power and Light Company (FPL) plans and conducts annual emergency preparedness exercises. This scenario has been written to conduct the 1992 Annual Evaluated Emergency Preparedness Exercise on February 12, 1992.

The exercise scenario will be performed during normal working hours (7:30 A.M. to 4:00 P.M. EST) on February 12, 1992 and will involve partial activation and participation by State and Local Government Agencies. The participation will include the activation of Emergency Response Facilities, mobilization of resources and full communications networks activity in response to an escalating emergency condition at St. Lucie Nuclear Plant resulting in a simulated release of radioactivity to the environment.

In accordance with NUREG 0654, FEMA-REP-1, Rev. 1, *Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants*: "An exercise is an event that tests the integrated capability and a major portion of the basic elements existing within the emergency preparedness plans and organizations." This exercise will include the elements of a Control Room drill, a communication drill, a radiological monitoring drill, a Health Physics drill, a fire and a medical emergency drill. It will require the activation of the Florida Power and Light Emergency Response Organization and Facilities. It will also include the exercise of the response activities of the State of Florida and Local Government Agencies.

The exercise is scheduled to be conducted on Wednesday, February 12, 1992. Exercise participants ("players") will have no prior knowledge of the exercise scenario. The intent of this exercise is to demonstrate that those individuals that are assigned responsibilities in a radiological emergency are adequately trained to perform in accordance with emergency preparedness plans and procedures.

The exercise will be observed, evaluated and critiqued by representatives of the Nuclear Regulatory Commission (NRC). The exercise will be controlled, observed, evaluated and critiqued by an FPL Controller Organization for the St. Lucie Plant portion of the exercise. The exercise will be controlled, observed, evaluated and critiqued by a State and Local Government Controller Organization for the off-site portion of the exercise. Portions of the off-site organization will be observed and evaluated by representatives of the Federal Emergency Management Agency (FEMA).

This scenario manual has been prepared to provide the Controller Organization the information and data necessary to conduct the exercise in an efficient and coordinated manner. It contains the following scenario sections:

Section 2.0, SCOPE, OBJECTIVES AND RULES; describes the exercise scope and objectives and sets forth the guidelines for conducting the exercise to meet those objectives. In addition, the rules for conduct of the exercise are detailed.

Section 3.0, SCENARIO; describes the postulated sequence of events that should require the various onsite and off-site Emergency Response Organizations to respond.

Section 4.0, MESSAGES; includes information in the form of message sheets which are utilized to control scenario activities. Messages will be used to initiate activities and ensure proper progression of the scenario. In addition, this section provides a scenario and messages for conducting the radiological medical emergency drill and mini-scenarios associated with various in-plant response teams.

Section 5.0, PLANT PARAMETERS; includes information concerning the status of plant parameters and equipment to provide simulated operational data to the exercise operations shift in Unit 1 Control Room.

Section 6.0, METEOROLOGICAL PARAMETERS; contains postulated meteorological information to be used for the scenario.

Section 7.0, RADIOLOGICAL DATA; contains time-related information concerning simulated radiological conditions at postulated onsite and off-site monitoring locations. Also included in this section is information concerning primary coolant activity, radiological release data, area radiation monitor readings, process radiation monitor readings and plume data for the teams tracking and monitoring the simulated release of radioactivity.

Section 8.0, MISCELLANEOUS DATA; contains evaluation materials for the FPL Controller/Evaluators, reference data for the plant parameters and a listing of acronyms and abbreviations.

Copies of this manual will be provided to the exercise controllers, evaluators and observers prior to the exercise. **NOTE:** *The scenario sequence of events, timing and data are to be maintained confidential and not available to the participants prior to the exercise.* Following the exercise, copies of this manual should be made available to key exercise participants for their information and review.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

1.2 SCHEDULE OF EVENTS

1.2.1 Player Briefing

Date: February 10, 1992
Time: 2:30 P.M.
Location: Plant Assembly Room, NTC, PSL
Purpose: Exercise play overview. Review of rules for players.
Participants: Players

1.2.2 Nuclear Regulatory Commission Entrance Briefing

Date: February 11, 1992
Time: 10:00 A.M.
Location: North Service Building Conference Room, PSL
Purpose: Exercise conduct. Introduction of Key Players, Controllers and NRC evaluators.
Participants: Plant Management, Key Players, Controllers and Nuclear Regulatory Commission Evaluation Team.

1.2.3 Exercise Scenario Review

Date: February 11, 1992
Time: 1:30 to 2:30 P.M.
Location: Energy Encounter Auditorium, NTC, PSL
Purpose: Exercise scenario review. Scenario walkthrough. Scenario data/play clarification.
Participants: Controllers and Nuclear Regulatory Commission Evaluation Team.

1.2.4 Controller/Evaluator Training

Date: February 11, 1992
Time: 2:30 to 3:00 P.M.
Location: Energy Encounter Auditorium, NTC, PSL
Purpose: Exercise control and evaluation overview. Review of rules for controllers.
Participants: Florida Power and Light Company Controllers, Controller/Evaluators and Evaluators.

1.2.5 Nuclear Regulatory Commission Plant Tour

Date: February 11, 1992
Time: 3:00 to 4:00 P.M.
Location: St. Lucie Unit 1 Control Room (CR), Technical Support Center (TSC), Operational Support Center (OSC), Emergency Operations Facility (EOF) and Emergency News Center (ENC).
Purpose: Plant and Emergency Response Facility (ERF) familiarization.
Participants: Nuclear Regulatory Commission Evaluation Team.

1.2.6 Exercise

Date: February 12, 1992
Time: Normal Working Hours (7:30 A.M.- 4:00 P.M.)
Location: St. Lucie Nuclear Plant, St. Lucie County, Martin County, Indian River County and Palm Beach County.
Purpose: Demonstration of emergency response capabilities.
Participants: All exercise participants.

1.2.7 Facility Critiques

Date: February 12, 1992
Time: Exercise Termination
Location: All Emergency Response Facilities
Purpose: Review and evaluate exercise play.
Participants: All exercise participants

1.2.8 Florida Power and Light Company Controller/Evaluator Critique

Date: February 13, 1992
Time: 8:00 to 11:00 A.M.
Location: Plant Assembly Room "B", NTC, PSL
Purpose: Review and evaluate exercise play. Formal exercise critique. Collect and prepare final critique comments.
Participants: All Florida Power and Light Company Controllers, Controller/Evaluators and Evaluators (Note: mandatory for all Lead Controllers).

1.2.9 Florida Power and Light Company Management Critique/Nuclear Regulatory Commission Exit

Date: February 14, 1992
Time: 9:00 A.M. 8:00
Location: Plant Assembly Room "A", NTC, PSL
Purpose: Presentation of major critique items to Plant Management, Nuclear Regulatory Commission preliminary Exercise findings.
Participants: Plant Management, Key Players, Lead Controllers and Nuclear Regulatory Commission Evaluators.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

1.3 PARTICIPATING AGENCIES

1.3.1 Utility

Florida Power and Light Company (FPL)

St. Lucie Nuclear Plant (PSL)

1.3.2 Federal

United States Nuclear Regulatory Commission (NRC), Region II - Evaluation Team

1.3.3 State

State of Florida Division of Emergency Management (DEM)

State of Florida Department of Health and Rehabilitative Services (DHRS), Office of Radiation Control

1.3.4 County

Martin County

St. Lucie County

Indian River County

Palm Beach County

1.3.5 Volunteer Agencies

American Red Cross (ARC)

1.3.6 Support Organizations

Martin Memorial Hospital

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

1.4 CONTROLLER ORGANIZATION

Exercise Controller:	Rick Walker
Onsite Lead Controller:	Rick Walker
Off-Site Lead Controller:	Don Mothena

Control Room:

Lead Controller:	Roger Walker
NPS/ANPS/Messages:	Roger Walker
Operators/Messages:	Dave Brown
Operators/Messages:	Dennis Borgman
Data:	Tom Ashley
HP:	Lang Jacobus
Plant Operators/SNPO:	Ray Voss

Technical Support Center:

Lead:	Sergio Valdes
EC/Communications:	Sergio Valdes
Dose Assessment/Chemistry:	Tom Ware
HP:	J.R. Smith
ERDADS:	Mitch Williford

Operational Support Center:

Lead:	Thom Coste
Repair Team:	Russell Williams
Repair Team:	Roger Thomas
Fire Team:	Bob McDaniel
First Aid/Decon Team:	Joel Burgess
HP:	Ray McCullers
HP(Medical):	Joel Burgess
Chemistry:	Joel Burgess
Field Monitoring Team:	
Field Monitoring Team:	
Field Monitoring Team:	

Security:

Security Team:	Dick Czarnecki
Security Team:	Peter Plantz

Emergency Operations Facility:

Lead:	Greg Casto
EOF/EC/State/County:	Greg Casto
EOF/HP/Dose Assessment:	Don Mothena
EOF/Medical/County EOC:	Dean Miller
Communications/Engineering:	Phil Green
Recovery Manager/staff:	Dave Sager
EOF/ENC:	Fitch King

Emergency News Center/Mock Media:

Mock Media:	Roger Scott
Mock Media:	Genevieve Blair
Mock Media:	Kathleen Wisniewski
Mock Media:	Jo Magennis
Mock Media:	Phyllis Cordover

Simulated Nuclear Regulatory Commission Operations Center:

NRCOC Simulation:	John Zudans

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

2.0 SCOPE, OBJECTIVES AND RULES

This section contains the following information:

Section 2.1	SCOPE - Describes the overall content and intended activities of the exercise. Details the anticipated responses and the emergency facilities to be activated.
Section 2.2	OBJECTIVES - Describes the objectives that will be evaluated for determination of satisfactory performance during the course of the Plume Exposure Pathway portion of the exercise.
Section 2.3	RULES - Describes the rules for Participants, Controllers, Evaluators and Observers which will be applied to the conduct of the exercise.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

2.1 SCOPE

To assure that the health and safety of the general public is protected in the event of an accident at St. Lucie Nuclear Plant (PSL), it is necessary for the Florida Power and Light Company (FPL) to conduct an annual emergency preparedness exercise. This is the 1992 Evaluated Exercise at St. Lucie Nuclear Plant. This exercise involves mobilization of FPL, State of Florida and Local Government Agency personnel and resources to respond to a simulated accident scenario. The exercise will be evaluated onsite by the Nuclear Regulatory Commission (NRC). An FPL Controller/Evaluator organization will control, observe, evaluate and critique the PSL portion of the exercise so that the emergency response capabilities of the utility may be assessed. A State of Florida and Local Government Agency Controller/Evaluator organization will control, observe, evaluate and critique the off-site portion of the exercise so that the emergency response capabilities of the off-site agencies may be assessed. Representatives of the Federal Emergency Management Agency (FEMA) will be observing and evaluating portions of the off-site organization.

The exercise will postulate conditions necessitating a declaration of a NOTIFICATION OF UNUSUAL EVENT, continue to escalate through an ALERT, a SITE AREA EMERGENCY and eventually result in the declaration of a GENERAL EMERGENCY.

Due to the compressed timeline of the exercise, some portions of the FPL Emergency Response Organization may be prepositioned. All onsite Emergency Response Facilities will be activated in accordance with simulated conditions and appropriate emergency response procedures for the exercise. Exercise participants ("players") will not have any prior knowledge of the simulated accident events, operational sequence, radiological effluents or weather conditions.

A radiological medical emergency will be integrated into the operational and radiological portion of the exercise scenario in order to evaluate the ability of the PSL plant staff to effectively respond to a contaminated/injured individual. The medical emergency will also test the ability of the designated hospital, Martin Memorial Hospital, to treat a contaminated/injured patient.

State and Local Government Agencies will participate partially in the off-site portion of the exercise.

2.1 SCOPE (Continued)

In addition, the exercise incorporates the following:

Radiological Monitoring Drill - both onsite and off-site teams will be dispatched during the exercise to obtain required air samples and measurements associated with a simulated off-site release of radioactivity and communicate these results to the appropriate Emergency Response Facility (ERF). (Field monitoring team protective clothing and respiratory protection will be simulated in the field.)

Health Physics Drill - involves the response to and analysis of simulated elevated activity airborne or liquid samples, radiation exposure control, emergency dosimetry and the use of protective equipment onsite.

Communications Drill - Actual usage and demonstration of the integrity of emergency response communications links and equipment.

Fire Drill - a demonstration of the onsite Fire Team to respond to and extinguish a simulated fire in emergency electrical panels in the Unit 1 Reactor Auxiliary Building (RAB).

Medical Emergency Drill - a demonstration of the response to a simulated medical emergency situation involving radiological considerations including the packaging and transport of the simulated injured person to the designated off-site treatment facility.

The preceding sub-drills are incorporated into the exercise scenario and will be demonstrated concurrently in the course of the exercise.

The overall intent of the exercise is to demonstrate that the FPL staff assigned responsibilities in an emergency situation are adequately trained to perform in accordance with emergency preparedness plans and procedures.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

2.2 OBJECTIVES

The St. Lucie Plant (PSL) 1992 emergency preparedness evaluated exercise objectives are based upon Nuclear Regulatory Commission requirements provided in 10 CFR 50, Appendix E, *Emergency Planning and Preparedness for Production and Utilization Facilities.* Additional guidance provided in NUREG-0654, FEMA-REP-1, Revision 1, *Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants,* was utilized in developing the objectives.

The exercise will be conducted and evaluated using a realistic basis for activities. Scenario events will escalate to core damage and subsequent release of radioactive material to the environment.

The following objectives for the exercise are consistent with the aforementioned documents:

A. Accident Assessment and Classification

1. Demonstrate the ability to identify initiating conditions, determine Emergency Action Level (EAL) parameters and correctly classify the emergency throughout the exercise.

B. Notification

1. Demonstrate the ability to alert, notify and mobilize Florida Power and Light (FPL) emergency response personnel.

2. Demonstrate the capability to promptly notify the U.S. Nuclear Regulatory Commission (NRC), State and Local Authorities of an emergency declaration or change in emergency classification.

3. Demonstrate appropriate procedures for both initial and follow-up notifications.

4. Demonstrate the ability to provide follow-up information to State, Local and Federal Authorities.

B. Notification (Continued)

5. Demonstrate the ability to provide accurate and timely information to State, Local and Federal Authorities concerning radioactive releases in progress.

C. Emergency Response

1. Demonstrate staffing and activation of Emergency Response Facilities (ERF).

2. Demonstrate planning for 24-hour per day emergency response capabilities.

3. Demonstrate the timely activation of the Technical Support Center (TSC) and Operational Support Center (OSC).

4. Demonstrate the functional and operational adequacy of the TSC, OSC, Emergency Operations Facility (EOF) and Emergency News Center (ENC).

5. Demonstrate the adequacy, operability and effective use of designated emergency response equipment.

6. Demonstrate the adequacy, operability and effective use of emergency communications equipment.

7. Demonstrate the ability of each emergency response facility manager to maintain command and control over the emergency response activities conducted within the facility throughout the exercise.

8. Demonstrate the ability of each facility manager to periodically inform facility personnel of the status of the emergency situation and plant conditions.

9. Demonstrate the precise and clear transfer of Emergency Coordinator responsibilities from the Nuclear Plant Supervisor to designated senior plant management and transfer of Emergency Coordinator responsibilities to the Recovery Manager.

10. Demonstrate the ability to promptly and accurately transfer information between Emergency Response Facilities.

C. Emergency Response (Continued)

11. Demonstrate the ability of the TSC and OSC to coordinate the deployment of emergency teams.

12. Demonstrate the availability of qualified personnel and timely organization of reentry teams to assist in accident assessment and mitigation.

13. Demonstrate the capability for development of Protective Action Recommendations (PAR) for the general public within the 10 Mile Emergency Planning Zone (EPZ).

14. Demonstrate that PARs can be communicated to State and Local Authorities within the regulatory time constraints.

D. Radiological Assessment and Control

1. Demonstrate the coordinated gathering of radiological and non-radiological (meteorological) data necessary for emergency response, including collection and analysis of in-plant surveys and samples.

2. Demonstrate the capability to perform dose assessment.

3. Demonstrate the ability to compare onsite and off-site dose projections to Protective Action Guidelines (PAGs) and determine and recommend the appropriate protective actions.

4. Demonstrate the ability to provide dosimetry to emergency response personnel as required and adequately track personnel exposure.

5. Demonstrate the capability for onsite contamination control.

6. Demonstrate the ability to adequately control radiation exposure to onsite emergency workers, as appropriate to radiological conditions.

7. Demonstrate the decision making process for authorizing emergency workers to receive radiation doses in excess of St. Lucie Plant administrative limits, as appropriate.

D. Radiological Assessment and Control (Continued)

8. Demonstrate the ability to control and coordinate the flow of information regarding off-site radiological consequences between radiological assessment personnel stationed at the TSC and EOF.

9. Demonstrate the ability of field monitoring teams to respond to and analyze an airborne radiological release through direct radiation measurements in the environment.

10. Demonstrate the collection and analysis of air samples and provisions for effective communications and recordkeeping.

11. Demonstrate the ability to control and coordinate the flow of information regarding off-site radiological consequences with State radiological assessment personnel in the EOF.

E. Public Information Program

1. Demonstrate the timely and accurate response to news inquiries.

2. Demonstrate the ability to brief the media in a clear, accurate and timely manner.

3. Demonstrate the ability to coordinate the preparation, review and release of public information with Federal (NRC), State and Local Government Agencies as appropriate.

4. Demonstrate the ability to establish and operate rumor control.

F. Medical Emergency

1. Demonstrate the ability to respond to a radiation medical emergency in a timely manner.

2. Demonstrate the capability of the First Aid and Personnel Decontamination Team to respond to a medical emergency, administer first aid and survey for contamination on a simulated contaminated injured individual.

3. Demonstrate the capability to arrange for and obtain transportation and off-site medical support for a radiological accident victim.

F. Medical Emergency (Continued)

4. Demonstrate the ability of Martin Memorial Hospital personnel to treat an injured and/or contaminated patient.

5. Demonstrate the administrative means to document and monitor status of a medical emergency victim.

G. Fire Emergency

1. Demonstrate the ability of the Fire Brigade to respond to a simulated fire emergency in a timely and appropriate manner.

H. Evaluation

1. Demonstrate ability to conduct a post-exercise critique to determine areas requiring improvement or corrective action.

I. Exemptions

Areas of the PSL Emergency Plan that will NOT be demonstrated during this exercise include:

1. Site evacuation of non-essential personnel

2. Onsite personnel accountability

3. Actual shift turnover (long term shift assignments will be demonstrated by rosters).

4. Actual drawing of a sample utilizing the Post-Accident Sampling System (PASS)

FLORIDA POWER AND LIGHT COMPANY
-- ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

2.3 RULES

The following general rules have been established to help delineate the extent of play for participants to meet exercise objectives.

1. Five groups of personnel may be in attendance and will function as described below:

   A. **PLAYERS**: Managers, supervisors, operators, health physics technicians, chemistry technicians, maintenance personnel, etc., who have been assigned a "Participant" role during the simulated emergency. These persons serve to take necessary actions to mitigate, terminate, correct and/or recover from the simulated events.

   B. **CONTROLLERS**: Those designated Florida Power and Light Company (FPL) and contracted personnel who serve an active role during the exercise by providing scenario data to participants. **CONTROLLERS** also serve to initiate certain actions (i.e.: contingency messages) in order to assure continuity of the events described in the exercise scenario. **CONTROLLERS** are the only personnel who will provide information to the **PLAYERS**. **CONTROLLERS** will also serve as **FPL EVALUATORS**.

   C. **EVALUATORS** : **FEDERAL EVALUATORS** from the Nuclear Regulatory Commission (NRC) will be evaluating the performance of FPL participants in accordance with the exercise scenario objectives. **FPL EVALUATORS** and **CONTROLLERS** designated by FPL will provide documentation and assessment of the exercise activities for the purpose of the internal self-critique. **EVALUATORS** serve a passive function (when not a Controller/Evaluator) and will only note actions taken by **PLAYERS**. These personnel may have specific areas to consider in their evaluation. **EVALUATORS** may ask questions to clarify actions taken or procedural concerns but should not interfere with the flow of events.

D. **OBSERVERS**: Personnel who serve no evaluation, control or participatory function in the exercise. These people may be from Turkey Point Nuclear Plant (PTN), the FPL Miami General Office, Juno Beach (JNO) office or another utility. **OBSERVERS** should not interfere with **EVALUATORS**, and particularly, **PLAYERS**. Questions from **OBSERVERS** should be directed to a **CONTROLLER**.

E. **VISITORS**: Personnel who serve no function related to the exercise activities. These people only enter a facility for a short time to perform non-exercise related functions. This group of people may be from PTN, JNO, the FPL Miami General Office or other organizations. Visitors shall not interfere with **PLAYERS, CONTROLLERS, EVALUATORS** or **OBSERVERS**.

Identification of the different groups of personnel will be accomplished by the use of badges. These identification badges are worn in addition to normal security badging and will be provided to all participants during the meetings prior to the exercise. Badges are to be marked with an erasable marker to show the participant's name (and position filled, if a Player). The four groups are identified by the following colors:

PLAYERS	Green
CONTROLLERS	Pink
EVALUATORS	Blue
OBSERVERS	White

2. Personnel will be assigned as controllers at all functional areas to monitor and control exercise activities. In addition, they will accompany radiological monitoring teams, plant health physics personnel, maintenance, repair and rescue teams.

3. Message forms will be the mechanisms used to initiate, orchestrate, modify and complete the events comprising the overall scenario. Controllers will use the message forms to initiate scenario events and trigger responses from the involved personnel. Some messages which deal with plant operations activities are used only in the Unit Control Room simulation of operations.

4. Each controller will have time-related plant and radiological parameters for the exercise scenario. This information will be issued when necessary or upon request from the appropriate players.

5. Controllers will **NOT** provide information to the participants regarding scenario development or resolution of problem areas encountered. Participants are expected to obtain information through their organizations and exercise their judgement in determining correct response actions and problem resolution.

6. Scenario data will be provided in at least 15-minute intervals. Data for the time period between the updates and any future times are not available to the participants. Participants should request from the controllers data which they feel is necessary for the performance of their function.

7. Some participants may insist that certain parts of the scenario are unrealistic. The controllers have the authority with the approval from the Exercise Controller to clarify any questions regarding scenario content. In some cases, it may be necessary to exercise "controller's prerogative" of countermanding participant actions to preserve the continuity and objectives of the exercise. Participants must accept the controllers word as final and proceed. Inappropriate actions can delay or speed-up the entire scenario timeline and impact other groups.

8. Scenario events are hypothetical. Any portions of the scenario depicting plant system operational transients are simulated events. **NO** scenario actions shall involve operation of plant systems or affect generation capability. To help delineate such actions, all exercise scenario messages *must* be preceded and followed by the words: **"THIS IS A DRILL"**!

9. Postulated accident conditions will result in a simulated radiological effluent release which may necessitate the consideration of protective actions for the general public.

10. All FPL onsite and off-site emergency response facilities will be manned and perform their prescribed functions as appropriate to the development of the exercise scenario.

11. Participation of FPL onsite personnel directly involved in responding to an emergency should be carried out to the extent necessary to meet the scope and objectives, including the deployment of radiological monitoring teams, emergency repair/damage control teams and other emergency workers. All actions are to be played out, as much as possible, in accordance with the emergency plan and procedures as if it were an actual emergency. Actions should be identified to the controller for guidance as to whether to play them out or simulate them.

12. Certain events and activities may be simulated rather than utilize the actual deployment of the resources.

    - Simulation - Involves identification and utilization of requirements and procedures short of actual deployment.
    - Actual - Involves movement of resources and/or physical implementation for this exercise.

13. Exercise participants, controllers, evaluators and observers should not take any action which would preclude maintaining emergency readiness of the organization and community. If an actual situation occurs that requires a group to terminate its participation in the exercise, they should notify the Exercise Controller. All messages concerning actual events must be preceded with **"THIS IS NOT, I REPEAT, NOT A DRILL MESSAGE"**. In the event of an actual situation during the exercise or other unforeseen contingencies, contact:

    Exercise Controller, pager no. 340-9327.

14. Communications between all exercise participants shall occur in accordance with the procedures of applicable emergency response plans. All communications, including initial telephone conversations, radio transmittals and loudspeaker announcements *must* begin and end with **"THIS IS A DRILL"**!

15. Intentional violation of laws is not permitted during the exercise. Participants, controllers, evaluators and observers should comply with all federal, state and local legal restrictions. All local traffic laws, specifically speed limits, shall be observed.

16. Exercise participants, controllers, evaluators and observers should avoid the endangering of public and private property.

17. It is not the intent to inconvenience or alarm the public during the conduct of the exercise. All communications, particularly in the public relations area, *must* begin and end with **"THIS IS A DRILL"**!

18. At the appropriate point in the exercise scenario, the Exercise Controller will initiate termination of the scenario. The Exercise Controller will ensure notification of off-site points of contact to advise them that the exercise is being terminated.

19. All exercise participants will take part in a critique session in their emergency response facility immediately upon termination of exercise activities.

20. All Evaluators and Controller/Evaluators should complete their evaluation materials and provide them to the Emergency Preparedness Department at the FPL Controller/Evaluator Critique following the exercise.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

3.0 SCENARIO

This section contains the following information:

Section 3.1	NARRATIVE SUMMARY - Describes in detail activities and events which occur during the scenario along with appropriate and anticipated corrective actions.
Section 3.2	SCENARIO TIMELINE - Provides a relationship between scenario events, real time and scenario time.
Section 3.3	SEQUENCE OF EVENTS TIMELINE - Provides a relationship between scenario activities and expected actions from the Emergency Response Organizations (ERO)s.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

3.1 NARRATIVE SUMMARY

3.1.1 Brief Narrative

The scenario begins with a fire occurring in the "1B5" 480 Volt Motor Control Center (MCC) in the Unit 1 "B" Switchgear Room (Unit 1 Reactor Auxiliary Building [RAB]). As a result of the fire uncontrolled for greater than 10 minutes, a NOTIFICATION of UNUSUAL EVENT (NUE) should be declared. Due to inoperative equipment and valves associated with the "1B5" MCC, operators begin a reactor power reduction. Difficulty in extinguishing the fire prompts a request for off-site fire assistance (simulated) which should produce a declaration of ALERT. A radwaste worker in the Unit 1 Drumming Room is injured and contaminated. The severity of the injury requires transport to the off-site medical treatment facility (Martin Memorial Hospital). A flexible bellows assembly in the Unit 1 condensate pump suction header fails, dumping condenser hotwell contents and breaking vacuum in the main condenser. The reactor and turbine are tripped and steam is dumped through the Atmospheric Dump Valves (ADVs). Heat damage to the "1B3" 4160 Volt Switchgear caused by the previous fire in the adjacent "1B5" 480 V. MCC causes a loss of the "1B3" 4160 V. Switchgear. Reduced output on the "1C" Auxiliary Feed Water (AFW) Pump causes a decreasing Steam Generator (S/G) level . The "1C" AFW Pump fails, resulting in total loss of feed to the S/Gs. When operators establish once-through cooling on the one operable High Pressure Safety Injection (HPSI) pump and Power Operated Relief Valve (PORV), a SITE AREA EMERGENCY (SAE) should be declared. Failure of the HPSI pump results in loss of once-through cooling and should produce a declaration of GENERAL EMERGENCY (GE). The lack of Reactor Coolant System (RCS) feed and steaming down through the PORV uncovers the core and causes fuel damage. Recovery of the "1A" AFW pump allows restoration of feed to the "1A" S/G. The addition of cold feed water to the dry generator causes tube ruptures and a release of contaminated steam to the environment through a failed open ADV. RCS reflood, cooldown, feed up of the S/G and closure of the ADV isolation valve terminate and contain the radioactive release.

3.1 NARRATIVE SUMMARY (Continued)

3.1.2 Detailed Narrative

Initial conditions establish Unit 1 operating at 100% power, in the middle of core life. Power history has been full power operation for the last 180 days. Unit 2 is in day 12 of a 30 day scheduled maintenance outage. The "1A" Auxiliary Feed Water (AFW) Pump is tagged-out for motor bearing replacement. The pump motor has been lifted from the baseplate and the end bells and bearing housings disassembled. The "1A" AFW Pump was taken out of service at 1600 on February 11, 1992, starting the 72 hour Limiting Condition for Operation (LCO) clock at that time, and is anticipated to be restored to service in another 6 to 8 hours. Demand on the system is moderate with an anticipated peak of 12,000 MWe. Service area conditions are normal. Weather has been sunny and mild for the last week with occasional late afternoon and evening showers. Forecast is for partly cloudy skies, temperatures in the upper 80's and occasional showers for the next four days. Current temperature is 78° with winds from the East at 4-8 mph.

At 0800, fire alarms at Delta 1.07.01 and 1.07.02 (Unit 1 Zone #7 A&B) occur. Trouble alarms occur for the "1B5" 480 Volt Motor Control Center (MCC). The "1B5" MCC trips. The Fire Team responds and fights the fire. The Senior Nuclear Plant Operator (SNPO) is dispatched to investigate the alarms. Within minutes, the SNPO reports verification of fire and smoke in the Unit 1 Reactor Auxiliary Building (RAB) 43' elevation in the "B" Switchgear Room.

The Fire Team responds and sets up to fight the fire in the "1B5" MCC. Due to a lack of fixed fire suppression and fire burning in the overhead cable trays uncontrolled for greater than 10 minutes, a NOTIFICATION of UNUSUAL EVENT (NUE) should be declared in accordance with Emergency Plan Implementing Procedure (EPIP) No. 3100022E, *Classification of Emergencies*,

EVENT/CLASS	UNUSUAL EVENT
3. FIRE	Uncontrolled fire within the plant lasting more than 10 minutes.

Operators should have evaluated the consequences to the electrical system and operational status utilizing Off-Normal Operating Procedure (ONOP) 1-0910054, *Loss of a Safety Related A. C. Bus*, and begun a down-power of the reactor utilizing boric acid feed to the Volume Control Tank (VCT).

3.1 NARRATIVE SUMMARY (Continued)

The Fire Team submits a request for off-site fire assistance (to be simulated). This should result in a declaration of ALERT in accordance with Emergency Plan Implementing Procedure (EPIP) No. 3100022E, *Classification of Emergencies*,

EVENT/CLASS	ALERT
3. FIRE	Uncontrolled fire, 1. Potentially affecting safety systems. AND 2. Requiring off-site support in the opinion of the NPS/EC.

A radwaste operator (simulated medical emergency volunteer victim) working in the Unit 1 Radwaste Building Drumming Room is injured severely when a B25 box lid falls on the victim. A coworker (Medical Controller) carries the injured person out of the Drumming Room and makes a report to the Unit 1 Control Room. The First-Aid Decon Team is dispatched to the scene. The injured individual is verified to be contaminated and the severity of the injury requires rapid transport to the off-site treatment facility (Martin Memorial Hospital). (Due to the levels of simulated contamination and the life-threatening nature of the simulated injuries, the patient will be transferred to the off-site facility still simulated to be contaminated.) The ambulance is ordered, the victim is given initial care and prepared for transport.

The simulated arrival of the off-site fire assistance team will provide the final step to extinguish the fire in the "1B5" 480 V. MCC. Fire cleanup and appraisal of damages in the "B" Switchgear Room will begin.

A sudden failure of a flex joint in the condensate header upstream of the pumps in Unit 1 results in the rapid loss of main condenser hotwell inventory and vacuum. Hot water fills the pump pit area under the Unit 1 Turbine Deck. The turbine and reactor are tripped due to rapidly decreasing vacuum. The condensate and main feed pumps trip on low suction pressure. When the Auxiliary Feed Actuation Signal (AFAS) occurs, the "1B" AFW pump autostarts. Heat damage as a result of the previous fire in the adjacent "1B5" 480 V. MCC causes the "1B3" 4160 Volt switchgear to fail. The instabilities in affected control relays that tripped the "1B3" switchgear prevent it from being reenergized. Loss of the "1B3" switchgear also causes the temporary loss of the "1AB" 4160 Volt and 480 Volt loadcenters until they are realigned to the "A" side 4160 Volt and 480 Volt supplies. When the "1C" AFW pump is started, the controller only brings the pump turbine up to 3000 rpm, providing only about 100 gpm of feed to the S/Gs. Operators stabilize the plant and begin cooling down on the S/G Atmospheric Dump Valves (ADV)s. The injured person should have been loaded and transported off site in the ambulance by this time:

3.1 NARRATIVE SUMMARY (Continued)

The "1C" AFW pump has been losing oil since starting up and the turbine seizes. The lack of feed to the S/Gs being utilized for heat rejection begins to steam down S/G level.

When the S/G levels reach 15% wide range, operators begin once-through cooling of the RCS utilizing the one operable High Pressure Safety Injection (HPSI) Pump and the one available Power Operated Relief Valve (PORV). Upon the opening of the PORV, a SITE AREA EMERGENCY (SAE) should be declared in accordance with Emergency Plan Implementing Procedure (EPIP) No. 3100022E, *Classification of Emergencies*,

EVENT/CLASS	SITE AREA EMERGENCY
1.C. LOSS OF SECONDARY COOLANT	TLOF with once-through cooling initiated 1. No main or auxiliary feedwater flow available. AND 2. PORV(s) have been opened to facilitate core heat removal.

The one operating HPSI Pump bearings fail and the pump locks up, stopping once-through cooling. RCS temperature and pressure increase as a result of the lack of heat removal. The open PORV continues to blow down RCS inventory to the containment through the Quench Tank. A GENERAL EMERGENCY (GE) should be declared based upon Total Loss of Feed (TLOF) followed by failure of once-through cooling in accordance with Emergency Plan Implementing Procedure (EPIP) No. 3100022E, *Classification of Emergencies*,

EVENT/CLASS	GENERAL EMERGENCY
6.A. INCREASED AWARENESS OF POTENTIAL CORE MELT	Emergency Coordinator's judgement that plant conditions exist that make release of large amounts of radioactivity in a short period appear possible or likely. (Any core melt situation.) 3. Total loss of feedwater followed by failure of once-through cooling (ECCS) to adequately cool the core.

Protective Action Recommendations (PARs) should be generated based upon plant conditions.

3.1 NARRATIVE SUMMARY (Continued)

The lack of sufficient heat removal and feed to the RCS causes reactor vessel level to decrease until the core is uncovered as indicated on the Reactor Vessel Level Monitoring System (RVLMS). Uncooled and uncovered fuel begins to release gas gap activity and fuel overheat begins to cause zirconium-water reaction, liberating hydrogen. Containment High Range Radiation Monitors (CHRRMs) increase sharply. When CHRRM readings have exceeded $4.2X10^4$ R/hr , PARs should be upgraded based upon plant conditions.

The "1A" AFW pump is reassembled and restored for feed to the S/Gs. The failure of feed valve controllers caused by the previous electrical casualties necessitates feeding of the "1A" S/G. When the cold auxiliary feedwater is admitted to the hot and dry "1A" S/G, approximately 50 U-tubes rupture. RCS equalizes into the S/G and the release of contaminated steam begins through the open ADV on the "1A" S/G. (NOTE: if operators attempt to close the ADV, the ADV controller has failed "as is" and may not be operated remotely.) The current system availability and the need to remove heat from the RCS necessitates steaming the damaged S/G to the atmosphere and releasing radioactive material.

The flooding up of the affected S/G with AFW covers the ruptured tubes, dilutes and partitions the release. Release radiation readings decline. Cool down and depressurization of the RCS reduces the primary to secondary flow. Repair and recovery team activities continue to stabilize the plant. Field monitoring teams track and monitor the release from the S/G ADV.

Cool down and the establishment of long-term cooling allow the closure of the affected S/G ADV isolation valve, terminating the release. Field monitoring activities continue on the plume in the environment. Repair and recovery teams and discussions are continuing in the Emergency Response Facilities. The exercise is terminated.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

3.2 SCENARIO TIMELINE

TIME	EVENT
0730	Initial conditions establish Unit 1 operating at 100% power, in the middle of core life. Power history has been full power operation for the last 180 days. Unit 2 is in day 12 of a 30 day scheduled maintenance outage. The "1A" Auxiliary Feed Water (AFW) Pump was placed on a clearance for motor bearing replacement at 1600 on February 11, 1992, starting the 72 hour Limiting Condition for Operation (LCO) clock at that time. Demand on the system is moderate with an anticipated peak of 12,000 MWe. Service area conditions are normal. Weather has been sunny and mild for the last week with occasional late afternoon and evening showers. Forecast is for partly cloudy skies, temperatures in the upper 80's and occasional showers for the next four days. Current temperature is 78° with winds from the East at 4-8 mph.
0800	A fire occurs in the "B" Switchgear Room of the Unit 1 Reactor Auxiliary Building (RAB) in the "1B5" 480 Volt Motor Control Center (MCC).
0805	Senior Nuclear Plant Operator (SNPO) verifies fire. Fire alarm is sounded and Fire Team is dispatched.
0815	Fire fighting efforts are hindered by the lack of fixed suppression system and difficulties experienced by the Fire Team. The fire burns greater than 10 minutes, affecting safety equipment.
0830	Due to difficulties experienced in fighting the fire, a request for off-site fire assistance (*simulated*) is made by the Fire Team. A **NOTIFICATION OF UNUSUAL EVENT** should be declared.

3.2. SCENARIO TIMELINE (Continued)

TIME	EVENT
0835	Operators start a down power by injecting boric acid into the Volume Control Tank (VCT).
0840	The off-site fire assistance is simulated to arrive at the East Gate by Security Controller message.
0845	An **ALERT** should be declared based upon the request (*simulated*) for off-site fire assistance .
0845 (Approx.)	Upon arrival and setup (*simulated*) of the off-site fire assistance team, the fire is out.
0900	The off-site fire assistance team is simulated to have departed site by Security Controller message.
0905	A person is reported to be injured in the Unit 1 RAB Drumming Room, a Player or Controller makes notification of the injury.
0910 (Approx.)	The First-Aid Decon team is dispatched to the injury scene. The injured person is examined by the First-Aid Decon Team, triage and radiological assessment is performed. Off-site medical assistance is required and the individual is contaminated. Off-site medical facility is informed, ambulance ordered and injured individual prepared for movement to the off-site treatment facility.
0930	A flexible joint fails on the suction header of the Unit 1 condensate pumps resulting in a large leak of condensate. A rapid loss of condenser vacuum caused by the damaged suction header and loss of hotwell inventory causes the operators to trip the turbine and reactor. Both main feed pumps trip on low suction. The "1B" AFW Pump autostarts on the Auxiliary Feed Actuation Signal (AFAS) and heat damage causes the "1B3" 4160 Volt switchgear to fault. The "1B3" 4160 Volt switchgear trips as a result of the previous fire in the adjacent "1B5" 480 V. MCC. The "1C" AFW Pump controller only brings the turbine up to reduced rpm, producing minimal feed to the Steam Generators (S/G)s. The severely reduced feed to the operating S/Gs begins to steam down the S/Gs.

3.2 SCENARIO TIMELINE (Continued)

TIME	EVENT
0945 (Approx.)	The injured individual has been transported to the off-site medical facility (Martin Memorial Hospital) by ambulance.
0945	The operators establish cool-down on the Steam Generator Atmospheric Dump Valves (ADV)s. The "1C" AFW Pump turbine experiences a loss of lubricating oil and locks up.
1015	When S/G levels reach 15% Wide range, operators begin once-through cooling on the one available High Pressure Safety Injection (HPSI) pump and the available Power Operated Relief Valves (PORV)s.
1030	A **SITE AREA EMERGENCY** should be declared as a result of the loss of feed and initiation of once-through-cooling by opening of the PORV.
1100	The one operable HPSI pump fails, stopping once-through cooling. The lack of heat rejection causes Reactor Coolant System (RCS) temperature and pressure to increase. The PORVs blow down excess pressure to the Quench Tank and containment.
1115	A **GENERAL EMERGENCY** should be declared based upon total loss of feedwater followed by failure of once-through-cooling. Protective Action Recommendations (PARs) are generated on plant conditions.
1130	The lack of heat removal in the RCS and decay heat causes full voiding and the core uncovers as indicated on the Core Exit Thermocouples (CET)s. Uncooled and uncovered fuel begins to release gas gap activity and fuel overheat begins to cause zirconium-water reaction, liberating hydrogen. Containment High Range Radiation Monitors (CHRRM)s increase sharply. Containment hydrogen increases.
1145	CHRRMS have exceeded $4.2X10^4$ R/hr. Protective Action Recommendations (PARs) are upgraded based upon plant conditions.

3.2. SCENARIO TIMELINE (Continued)

TIME	EVENT
1200	The recovery of the "1A" AFW Pump allows the feeding of the "1A" S/G and resumption of heat rejection through the "1A" S/G and atmospheric dumps. The addition of cold feed water to the hot and dry "1A" S/G results in the rupturing of approximately 50 U-tubes in the "1A" S/G. The necessity of rejecting heat mandates utilizing the one available leaking S/G to release contaminated steam. On the initiation of feed to the dry S/G, the open atmospheric dump valve (ADV) fails as is, preventing the remote closure of the ADV. Off-site release rates increase sharply. PARs are upgraded based upon plant conditions.
1400	Cooldown of the RCS has reduced primary pressure and reduced the primary to secondary flow in the S/G being utilized for cooldown. Auxiliary feed has refilled the "1A" S/G covering and partitioning the tube ruptures. Release radiation readings begin to decline. Field monitoring activities continue. The emergency response teams continue to stabilize the reactor, verify safe shutdown and evaluate system integrity. Discussions of recovery and reentry should begin as the release rate continues to decline.
1500	Termination of Exercise Play.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

3.3 SEQUENCE OF EVENTS TIMELINE

TIME: **0730**

EMERGENCY CLASSIFICATION: NONE

SCENARIO EVENT:

Initial Conditions:

Initial conditions establish Unit 1 operating at 100% power, in the middle of core life. Power history has been full power operation for the last 180 days. Unit 2 is in day 12 of a 30 day scheduled maintenance outage. The "1A" Auxiliary Feed Water (AFW) Pump has been placed on a clearance for motor removal and motor bearing replacement. The motor has been disconnected electrically and mechanically, lifted from the baseplate and end bells removed for bearing removal. The AFW pump work places Unit 1 in a 72 hour Limiting Condition for Operation (LCO) started at 1600 on February 11, 1992. The work is anticipated to require another 6 to 8 hours. Demand on the system is moderate with an anticipated peak of 12,000 MWe. Service area conditions are normal. Weather has been sunny and mild for the last week with occasional late afternoon and evening showers. Forecast is for partly cloudy skies, temperatures in the upper 80's and occasional showers for the next four days. Current temperature is 78° with winds from the East at 4-8 mph.

ANTICIPATED ACTION:

Unit 1 Control Room (Exercise Player) Operations staff receives initial conditions turnover.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 0800

EMERGENCY CLASSIFICATION: NONE

SCENARIO EVENT:

Fire alarms at delta point numbers 1.07.01 and 1.07.02 (Unit 1 Zone #7 A&B) occur.

ANTICIPATED ACTION:

Senior Nuclear Plant Operator (SNPO) is dispatched to investigate the alarms.

TIME: 0805

EMERGENCY CLASSIFICATION: NONE

SCENARIO EVENT:

SNPO verifies the fire alarm.

ANTICIPATED ACTION:

Sound fire alarm, make announcements, dispatch Fire Team. Nuclear Plant Supervisor (NPS) should refer to Emergency Plan Implementing Procedure (EPIP) No. 3100022E, *Classification of Emergencies*, EPIP No. 3100025E, *Fire Emergencies*, and Administrative Procedure No. 1-1800023, *Unit 1 Fire Fighting Strategies.*

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: **0810**

EMERGENCY CLASSIFICATION: NONE

SCENARIO EVENT:

Fire Team responds to the "B" 4160 Volt Switchgear Room on the 43' elevation of the Unit 1 Reactor Auxiliary Building (RAB). The "1B5" 480 Volt MCC is isolated for firefighting purposes.

ANTICIPATED ACTION:

Operators should enter Off-Normal Operating Procedure (ONOP) No. 1-0910054, *Loss of a Safety Related A.C. Bus.*

TIME: **0815**

EMERGENCY CLASSIFICATION: NONE

SCENARIO EVENT:
Fire Team continues firefighting action on the 43' elevation of the Unit 1 RAB.

ANTICIPATED ACTION:
Continue in ONOP No. 2-0910054, evaluate the consequences of the loss of the "1B5" 480 Volt MCC .

NPS/EC: A NOTIFICATION of UNUSUAL EVENT (NUE) should be declared in accordance with EPIP No. 3100022E, *Classification of Emergencies,*

EVENT/CLASS	UNUSUAL EVENT
3. FIRE	Uncontrolled fire within the plant lasting more than 10 minutes.

Complete Fire or Explosion Emergency Checklist, Unusual Event Checklist and perform notifications in accordance with EPIP No. 3100021E, *Duties and Responsibilities of the Emergency Coordinator.*

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 0830

EMERGENCY CLASSIFICATION: NOTIFICATION OF UNUSUAL EVENT (NUE)

SCENARIO EVENT:

Difficulties experienced in fighting the fire in the "1B5" 480 Volt MCC and the overhead cable trays in addition to the lack of fixed suppression systems in that area lead to a request from the Fire Team for off-site fire assistance. (NOTE: Off-site fire assistance will be *simulated.*)

ANTICIPATED ACTION:

NPS/EC: In accordance with EPIP No. 3100021E, *Duties and Responsibilities of the Emergency Coordinator,* complete another Fire or Explosion Emergency Checklist and notify the off-site fire assistance organization (simulated call).

Declare an ALERT in accordance with Emergency Plan Implementing Procedure (EPIP) No. 3100022E, *Classification of Emergencies,*

EVENT/CLASS	ALERT
3. FIRE	Uncontrolled fire, 1. Potentially affecting safety systems. AND 2. Requiring off-site support in the opinion of the NPS/EC.

Complete Alert Checklist and perform notifications in accordance with EPIP No. 3100021E, *Duties and Responsibilities of the Emergency Coordinator.*

Technical Support Center (TSC):
Begin activation

Operational Support Center (OSC):
Begin activation

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 0835

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

The loss of AFW feed isolation valves due to degraded electrical condition in conjunction with the inoperative "1A" AFW pump causes operators to begin a power reduction (approximately 5 Megawatts per min) by injecting boric acid into the Volume Control Tank (VCT). (NOTE: VCT injection chosen because Boron Load Control Valve, V-2525 is deenergized by the loss of the "1B5" 480 Volt MCC.)

ANTICIPATED ACTION:

Operators: Commence Reactor power reduction.

TIME: 0840

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

The off-site fire assistance group is simulated to have arrived at the site by a Security Controller Message to the Security Officers at the East Gate.

ANTICIPATED ACTION:

Security makes notification of the arrival of the simulated off-site fire team and simulates escort to the fire scene.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 0845

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

The simulated off-site fire assistance has arrived and joined the Fire Team at the "B" Switchgear Room in the Unit 1 RAB. The fire in the "1B5" 480 Volt MCC and overhead cable trays is out.

ANTICIPATED ACTION:

Fire Team: Complete firefighting activities in the "B" Switchgear Room. Perform initial damage appraisal.

TIME: 0900

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

The off-site fire assistance group is simulated to have departed the site by a Security Controller Message.

ANTICIPATED ACTION:

Security makes notification of the departure of the simulated off-site fire team.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 0905

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

A Radwaste operator (medical drill volunteer victim) working in Drumming Room of the Unit 1 RAB is injured when a B25 box lid falls and strikes the worker. The injured person has sustained a gash on the arm and a blow to the head. Another worker (or exercise medical controller) is simulated to have moved the semi-conscious victim outside of the Drumming Room and makes telephone notification of the injury.

ANTICIPATED ACTION:

NPS/EC: The First-Aid/Decon Team is requested by the Unit 1 Control Room.

TIME: 0910 (Approximate)

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

First-Aid/Decon Team responds to the scene of the injury in the Unit 1 RAB.

ANTICIPATED ACTION:

The First-Aid/Decon Team arrives on the scene. Triage is performed. The victim is determined to be contaminated. Initial medical appraisal indicates serious injuries. Off-site medical assistance is necessary and an ambulance is requested.

NPS/EC: Complete Medical Emergency Checklist in accordance with EPIP No. 3100021E, *Duties and Responsibilities of the Emergency Coordinator.* Call for the off-site ambulance.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: **0930**

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

A sudden failure in a flexible pipe joint in the suction header of the Unit 1 Main Condensate Pumps results in a rapid loss of condenser vacuum and loss of hotwell inventory. Condensate Pumps and Feed Pumps trip. Hot water inundates the pump pit area under the Unit 1 Turbine Deck. When the Auxiliary Feed Actuation Signal (AFAS) autostarts the "1B" AFW pump, the "1B3" 4160 Volt bus faults and trips. Heat damage and instabilities in the "1B3" switchgear caused by the fire in "1B5" MCC prevent it from being utilized for the remainder of the exercise. The phase differential lockout of the "1B3" switchgear also deenergizes the "1AB" 4160 and 480 Volt loadcenters normally aligned to the "B" side until they are realigned to "A" side sources. When the "1C" AFW pump starts, a failure in its turbine controller only brings its turbine up to 3000 rpm, producing only about 100 gpm Steam Generator (S/G) feed. The S/Gs begin to steam down due to lack of feedwater.

The OSC and TSC should have been activated.

ANTICIPATED ACTION:

Operators: Trip the turbine/reactor manually due to low condenser vacuum. Enter Emergency Operating Procedure (EOP) 1-EOP -01, *Standard Post Trip Actions*. Reject heat through the Atmospheric Dump Valves (ADVs). Respond to the loss of the "1B3" 4160 Volt bus in accordance with ONOP No. 1-0910054, *Loss of a Safety Related A.C. Bus*. Transfer the "1AB" 4160 and 480 Volt loadcenters to the "A" side supplies. Investigate the "1C" AFW pump minimal output.

TSC: The Emergency Coordinator in the TSC should have relieved the NPS/EC of his Emergency Coordinator responsibilities. TSC should have assumed notification and communication responsibilities. Off-site treatment Facility (Martin Memorial Hospital) should be notified. Ambulance should be ordered and Security alerted for ingress on arrival. Respond to the loss of 4160 Volt Emergency bus, minimal AFW feed and condensate leak.

OSC: The OSC should have been activated by this time and offered electrical maintenance team support to the damage assessment in the "B" Switchgear Room. Dispatch teams as requested to the "1C" AFW pump, condensate leak and loss of "1B3" 4160 Volt bus.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 0945 (Approximate)

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

The injured person is transferred to the ambulance.

ANTICIPATED ACTION:

Security expedites the ingress of the ambulance. The ambulance is prepared to receive the contaminated victim. Ambulance attendants are given dosimetry and briefed on the condition of the patient. The victim is packaged and moved out of the Radiation Control Area into the ambulance with minimum delay. A Health Physics (HP) Technician accompanies the victim in the ambulance and goes to the hospital to provide Health Physics support.. Security expedites the egress of the ambulance.

TIME: 0945

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

"1C" AFW pump has been losing oil since startup. The "1C" AFW pump turbine experiences bearing lockup due to lack of lubrication.

ANTICIPATED ACTION:

Operators: Stabilize the plant, enter Emergency Operating Procedure (EOP) No. 1-EOP-06, *Total Loss of Feedwater*. Begin to cool down on the ADVs and monitor S/G levels.

TSC: Investigate the loss of the "1C" AFW pump. Evaluate the possibility of expedited recovery of the "1A" and "1B" AFW pumps.

OSC: Field teams as requested to respond to the ongoing equipment casualties.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 1015

EMERGENCY CLASSIFICATION: ALERT

SCENARIO EVENT:

S/Gs have reached 15% wide range level.

ANTICIPATED ACTION:

Operators: Enter 1-EOP-15, *Functional Recovery.* Begin once-through cooling on the one available High Pressure Safety Injection (HPSI) pump ("1A") and the available Power Operated Relief Valves (PORV)s.

TSC: EC should declare a SITE AREA EMERGENCY on the opening of the PORV in accordance with EPIP No. 3100022E, *Classification of Emergencies,*

EVENT/CLASS	SITE AREA EMERGENCY
1.C. LOSS OF SECONDARY COOLANT	TLOF with once-through cooling initiated 1. No main or auxiliary feedwater flow available. AND 2. PORV(s) have been opened to facilitate core heat removal.

EC should complete the Site Area or General Emergency Checklist and the TSC should perform notifications in accordance with EPIP No. 3100021E, *Duties and Responsibilities of the Emergency Coordinator.*

Continue to evaluate plant conditions and respond to the lack of S/G feed.

OSC: Field teams as required to respond to the ongoing equipment casualties.

Emergency Operations Facility (EOF): Begin activation (if the Emergency Control Officer [ECO] had not determined to activate earlier).

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: **1100**

EMERGENCY CLASSIFICATION: SITE AREA EMERGENCY (SAE)

SCENARIO EVENT:

The "1A" HPSI pump begins to experience high vibration and the bearings seize. The loss of once-through cooling causes Reactor Coolant System (RCS) inventory to decrease, temperature and pressure to increase. The open PORVs continue to blow down RCS inventory to the containment through the Quench Tank. Core Exit Thermocouples (CET)s begin to increase.

ANTICIPATED ACTION:

Operators: Evaluate the consequences of the loss of once-through cooling. Monitor RCS inventory and CET temperatures.

TSC: A GENERAL EMERGENCY (GE) should be declared based upon Total Loss of Feed (TLOF) followed by failure of once-through cooling in accordance with Emergency Plan Implementing Procedure (EPIP) No. 3100022E, *Classification of Emergencies,*

EVENT/CLASS	GENERAL EMERGENCY
6.A. INCREASED AWARENESS OR POTENTIAL CORE MELT	Emergency Coordinator's judgement that plant conditions exist that make release of large amounts of radioactivity in a short period appear possible or likely. (Any core melt situation.) 3. Total loss of feedwater followed by failure of once-through cooling (ECCS) to adequately cool the core.

The EC should complete the Site Area or General Emergency Checklist and the TSC should perform notifications in accordance with EPIP No. 3100021E, *Duties and Responsibilities of the Emergency Coordinator.* Protective Action Recommendations (PARs) should be generated based upon plant conditions.

OSC: Continue to provide teams and support as requested to respond to the situation.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 1130

EMERGENCY CLASSIFICATION: GENERAL EMERGENCY (GE)

SCENARIO EVENT:

RCS isolation with no heat removal and no Emergency Core Cooling System (ECCS) feed has caused reactor vessel level to steam down. Core Exit Thermocouples (CET)s indicate core voiding and fuel uncovery. The uncovered upper fuel region of the core begins to release gas gap activity and fuel overheat begins to cause zirconium-water reaction, liberating hydrogen. Containment High Range Radiation Monitors (CHRRMs) increase sharply. Containment hydrogen increases.

ANTICIPATED ACTION:

Operators: Continue efforts to restore ECCS and AFW. Monitor the increasing containment hydrogen and radiation readings. Monitor CET temperatures.

TSC: Evaluate and take precautions against the increasing radiation fields associated with the containment and systems. Monitor and analyze for potential effluent releases. Perform dose projections based upon containment monitor readings.

OSC: Continue to provide teams and support to mitigation efforts. Monitor teams in the field and ensure the changing radiological conditions are disseminated to emergency response personnel.

EOF: Should have been activated by this time. Perform dose assessment and establish communications with State and Local Government.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 1145

EMERGENCY CLASSIFICATION: GE

SCENARIO EVENT:

Fuel damage has peaked at approximately 80% of gas gap release and 25% of fuel overheat. CHRRMS have exceeded $4.2X10^4$ R/hr.

ANTICIPATED ACTION:

Operators: Continue attempts to recover ECCS and AFW.

TSC: Continue to track increasing radiation levels and take precautionary actions. Perform dose projections on potential releases based upon source term in containment.

OSC: Continue to provide emergency response teams as capable. Monitor and control emergency response personnel exposure.

EOF: Perform dose projections and continue to maintain communications with State and Local Government. Make Protective Action Recommendations (PARs) based upon plant conditions.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 1200

EMERGENCY CLASSIFICATION: GE

SCENARIO EVENT:

Emergency team(s) have succeeded in remounting and aligning the "1A" AFW pump. The "1A" AFW pump is started and the "1A" S/G is fed. The shock of cold water from the Condensate Storage Tank (CST) on the hot and dry S/G tube bundle causes approximately 50 U-tubes to rupture. The open ADV on the affected S/G has failed "as is" and cannot be moved in either direction remotely. The resulting sluice of fuel-damaged primary coolant releases almost pure primary steam from the ADV until the AFW feed can cover and dilute the tube leakage. Main steam line monitor readings, ambient radiation readings around the Steam Trestle and field readings spike as a result of the release of radioactive material to the environment.

ANTICIPATED ACTION:

Operators: Feed up the affected S/G with AFW to cool, partition and dilute the primary to secondary leakage.

TSC: Take precautionary actions onsite for dramatically increased building radiation fields and release rates. Perform dose assessment on the release. Make onsite notifications of changing plant conditions.

OSC: Take precautionary actions for teams in the field. Monitor and evaluate increased radiation readings as a result of the release in progress.

EOF: Perform dose assessment on the release. Upgrade PARs and make off-site notifications of changing plant conditions.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 1400

EMERGENCY CLASSIFICATION: GE

SCENARIO EVENT:

Cooldown, depressurization, Safety Injection Tank (SIT) and Low Pressure Safety Injection (LPSI) feed has reflooded the core. Offsite and Steam Trestle radiation readings have declined as primary to secondary flow has equalized and stopped, AFW has flooded and contained the leakage and RCS depressurization has allowed LPSI floodup. The release of contaminated steam has nearly ceased and manual isolation of the stuck ADV may be performed on the Steam Trestle.

ANTICIPATED ACTION:

Operators: Continue stabilization and cooldown of the primary system.

TSC: Continue dose assessment and projection. Continue core damage estimates, mitigation support activities, long-term safe shutdown and recovery strategy development.

OSC: Continue repair and monitoring team activities as capable.

EOF: Continue dose assessment and projection. Review projections and continue to make PARs to state and local authorities. Continue core damage estimates, mitigation support activities, long-term safe shutdown and recovery strategy development.

3.3 SEQUENCE OF EVENTS TIMELINE (Continued)

TIME: 1500

EMERGENCY CLASSIFICATION: GE

SCENARIO EVENT:

The primary system has been cooled-down, depressurized and stabilized. Radiation readings onsite indicate virtually no release of radiation. The plume has been tracked, monitored and analyzed by field teams.

ANTICIPATED ACTION:

The exercise is terminated. A facility critique will be held in all key facilities with the Lead Controllers and the participants.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

4.0 MESSAGES

This section contains the following information:

Section 4.1 CONTROLLER MESSAGES - This section contains messages which are used to control the scenario.

Section 4.2 MEDICAL SCENARIO and MESSAGES - This section contains all information needed for the medical exercise.

Section 4.3 MINI-SCENARIOS - This section contains detailed mini-scenarios for specific activity areas within the main scenario.

CONTROLLER MESSAGE

To: Unit 1 CR Operations Crew

Number: 001

From: CR Controller

Time: 0730

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Initial Conditions:

Initial conditions establish Unit 1 operating at 100% power, in the middle of core life. Power history has been full power operation for the last 180 days. Unit 2 is in day 12 of a 30 day scheduled maintenance outage. The "1A" Auxiliary Feed Water (AFW) Pump has been placed on a clearance for motor removal and motor bearing replacement. The motor has been disconnected electrically and mechanically, lifted from the baseplate and end bells removed for bearing removal. The pump was taken out of service at 1600 on February 11, 1992 and the work is anticipated to require another 6 to 8 hours. The AFW pump work places Unit 1 in a 72 hour Limiting Condition for Operation (LCO) started when the pump was tagged out. Demand on the system is moderate with an anticipated peak of 12,000 MWe. Service area conditions are normal.

Radiological and Radiochemical Conditions:

Normal

Meteorological Conditions:

Weather has been sunny and mild for the last week with occasional late afternoon and evening showers. Forecast is for partly cloudy skies, temperatures in the upper 80's and occasional showers for the next four days. Current temperature is 78° with winds from the East at 4-8 mph.

NOTE:

For purposes of the exercise only, utilize the following contact numbers:

Emergency Control Officer (ECO)	**R.E. Grazio**	**(407)694-3180**
		(407)854-9092(beeper)
Nuclear Energy Duty Officer (NEDO)	**H.N. Paduano**	**(407)694-4190**
		(407)854-5528(beeper)
NRC Operations Center (ENS Call)	**Drill Simulation**	**(407)694-4218.**

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Unit 1 CR RCO/NPS

Number: 002

From: CR Controller

Time: 0800

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

The following alarm(s) have just come in on the Master Fire Panel:

Delta point number 1.07.01, Unit 1 Zone 7 A

Delta point number 1.07.02, Unit 1 Zone 7 B

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Senior Nuclear Operator (SNPO)

Number: 003

From: Fire Controller

Time: 0805 [Approximate]

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

You find the following conditions on the 43' elevation of the Unit 1 Reactor Auxiliary Building (RAB):

Thick black smoke is emanating from the door to the 4160V "B" Switchgear Room.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Fire Team Leader

Number: 004

From: Fire Controller

Time: 0810 [Approximate]

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Provide the following information only to the Fire Team Leader as an entry is performed on the 43' elevation of the RAB in the 4160V "B" switchgear (SWGR) and simulated firefighting activities performed.

Inside the 4160V "B" SWGR Room:

There is a thick layer of black smoke on the ceiling.

There is a very strong odor of burnt insulation.

There are several cubicles in the "1B5" 480 Volt Motor Control Center (MCC) that have their access panels blackened and soot covered.

The top portions of the "1B5" MCC panels are blackened with soot and thin streams of black smoke are escaping from several areas.

There is a smoldering fire in the cable tray over the "1B5" MCC still producing black smoke.

****

THIS IS A DRILL

****

CONTINGENCY MESSAGE

To: Unit 1 CR Nuclear Plant Supervisor (NPS)

Number: C-001

From: CR Controller

Time: 0830

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

If not already done so, declare a NOTIFICATION of UNUSUAL EVENT (NUE) at this time in accordance with EPIP No. 3100022E, *Classification of Emergencies*,

EVENT/CLASS	UNUSUAL EVENT
3. FIRE	Uncontrolled fire within the plant lasting more than 10 minutes.

****

THIS IS A DRILL

****

CONTINGENCY MESSAGE

To: Fire Team Leader

Number: C-002

From: Fire Controller

Time: 0830

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

If not already decided to do so, request off-site fire assistance at this time .

****

THIS IS A DRILL

****

CONTINGENCY MESSAGE

To: Unit 1 CR Nuclear Plant Supervisor (NPS)

Number: C-003

From: CR Controller

Time: 0830

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Off-site fire assistance is to be ***SIMULATED***. **DO NOT CALL OFF-SITE FIRE.** Make all other responses as if off-site fire assistance has been called at this time.

****

THIS IS A DRILL

****

CONTINGENCY MESSAGE

To: Unit 1 CR Nuclear Plant Supervisor (NPS)

Number: C-004

From: CR Controller

Time: 0835

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

If not already done so, you have begun a controlled reactor shutdown at 5 MWe/min by injecting Boric Acid to the Volume Control Tank (VCT) at approximately 10 gpm due to the loss of 2 out of three AFW paths.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: East Gate Security

Number: 005

From: Security Controller

Time: 0840

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

The off-site fire company has arrived at the East Security Gate. Make all notifications as if the off-site fire company were actually here.

****

THIS IS A DRILL

****

CONTINGENCY MESSAGE

To: Unit 1 CR Nuclear Plant Supervisor (NPS)

Number: C-005

From: CR Controller

Time: 0845

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

If not already done so, declare an ALERT at this time in accordance with Emergency Plan Implementing Procedure (EPIP) No. 3100022E, *Classification of Emergencies*,

EVENT/CLASS	ALERT
3. FIRE	Uncontrolled fire, 1. Potentially affecting safety systems. AND 2. Requiring off-site support in the opinion of the NPS/EC.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Fire Team Leader/SNPO

Number: 006

From: Fire Controller

Time: 0845

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

The remainder of information for the fire and recovery actions in the "B" Switchgear Room will be provided from the Mini-Scenarios, Section 4.3.

The off-site fire assistance team has arrived on the scene. The fire is out.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Security

Number: 007

From: Security Controller

Time: 0900

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

The off-site fire company has departed through the East Security Gate. Make all notifications as if the off-site fire company had actually left through the gate.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Unit 1 Control Room

Number: 008

From: Conscripted Player/Medical Controller

Time: 0905

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

The remainder of information for the medical emergency scenario will be provided from the Medical Emergency Mini-Scenario, Section 4.2.

A worker has been injured in the Drumming Room of the Unit 1 Radwaste Building. The victim is semi-conscious and bleeding outside the Drumming Room. We need immediate medical assistance.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Unit 1 Control Room

Number: 009

From: OSC Lead Controller

Time: 0930

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Provide the following information to any and all personnel in the immediate vicinity of the Unit 1 Turbine Mezzanine and Ground Floor at this time. If there are no players in the immediate vicinity, phone the information in to the Unit 1 Control Room yourself and provide the message to any responding or passing players.

There is hot steamy water flowing in the pits under the Unit 1 Turbine Deck.

The condensate piping and pump pits are filled with hot water.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Repair Team Leader

Number: 010

From: Repair Team Controller

Time: 0930

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Provide the following information to any and all personnel in the immediate vicinity of the Unit 1 RAB 43' elevation "B" Switchgear Room at this time. If there are no players in the immediate vicinity, phone the information in to the Unit 1 Control Room yourself and provide odor and visible portion of the information in the message to any responding or passing players.

Several breakers and relays slammed in the "1B3" 4160 Volt Switchgear.

There was a loud arcing noise in the vicinity of the "1B3" 4160 Volt Switchgear.

There is the strong odor of burnt rubber, ozone and hot metal.

There is no visible smoke or fire.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Repair Team Leader

Number: 011

From: Repair Team Controller

Time: 0945 (Approximate)

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Provide the following information to any player personnel dispatched to investigate the loss of the "1C" AFW pump.

There is a hazy cloud of oily vapor in the "1C" AFW pump area.

The pump base is shiny and wet with oil.

There is a strong odor of burnt oil and hot metal.

The turbine bearing housings are smoking slightly and are too hot to be touched.

****

THIS IS A DRILL

****

CONTINGENCY MESSAGE

To: Emergency Coordinator

Number: C-006

From: TSC Controller

Time: 1030

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Deliver the following message if a SITE AREA EMERGENCY has not been declared by this time and with the prior approval of the Exercise Controller.

A SITE AREA EMERGENCY should be declared IAW EPIP No. 3100022E, *Classification of Emergencies*,

EVENT/CLASS
1.C. LOSS OF SECONDARY COOLANT

SITE AREA EMERGENCY
TLOF with once-through cooling initiated

1. No main or auxiliary feedwater flow available.
   AND
2. PORV(s) have been opened to facilitate core heat removal.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: RCO

Number: 012

From: CR Controller

Time: 1100

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Provide the following information to operations players in the Unit 1 Control Room to indicate the loss of the "1A" HPSI pump.

The "1A" HPSI pump has a green light.

There is "0" amps and "0" flow.

"1A" HPSI pump has tripped.

The "1A" HPSI pump will not restart from the Control Room.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: Repair Team Leader/SNPO

Number: 013

From: Repair Team Controller

Time: 1100 (Approximate)

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Provide the following information to any player personnel dispatched to investigate the loss of the "1A" HPSI pump.

At the Pump Breaker:

The instantaneous overcurrent flags are showing on all three phases.

In the HPSI Pumproom:

There is a strong odor of burnt oil and hot metal.

The pump bearing housings are smoking slightly and are too hot to be touched.

****

THIS IS A DRILL

****

CONTINGENCY MESSAGE

To: Emergency Coordinator

Number: C-007

From: TSC Controller

Time: 1115

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Deliver the following message if a GENERAL EMERGENCY has not been declared by this time and with the prior approval of the Exercise Controller.

A GENERAL EMERGENCY should be declared IAW EPIP No. 3100022E, *Classification of Emergencies*,

EVENT/CLASS	GENERAL EMERGENCY
6.A. INCREASED AWARENESS OR POTENTIAL CORE MELT	Emergency Coordinator's judgement that plant conditions exist that make release of large amounts of radioactivity in a short period appear possible or likely. (Any core melt situation.) 3. Total loss of feedwater followed by failure of once-through cooling (ECCS) to adequately cool the core.

****

THIS IS A DRILL

****

CONTINGENCY MESSAGE

To: OSC Supervisor

Number: C-008

From: OSC Lead Controller

Time: 1130

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Deliver the following message if team play at the "1A" AFW pump has not occurred or is not proceeding at sufficient pace to simulate recovery with the prior approval of the Exercise Controller.

The team(s) working on the "1A" AFW pump have remounted the motor and reconnected the motor leads. After "bumping" to test rotation, the pump will be ready to run in about ten minutes.

****

THIS IS A DRILL

****

CONTINGENCY MESSAGE

To: RCO

Number: C-009

From: CR Controller

Time: 1155

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

Controller Note:

Deliver the following message if team play at the "1A" AFW pump has not occurred or is not proceeding at sufficient pace to simulate recovery with the prior approval of the Exercise Controller.

The team(s) working on the "1A" AFW pump have remounted the motor and reconnected the motor leads. After "bumping" to test rotation, the pump will be ready to run in five minutes.

****

THIS IS A DRILL

****

CONTROLLER MESSAGE

To: ALL PARTICIPANTS

Number: 014

From: ALL CONTROLLERS

Time: 1500

****

THIS IS A DRILL

DO NOT initiate actions affecting normal plant operations

****

Message:

The Emergency Preparedness Evaluated Exercise is Terminated.

Restore all emergency facilities and equipment to a readiness condition.

Collect all logs, data sheets and notification forms and ensure they are given to the facility lead controllers.

Lead Controllers hold an in-place critique with all participants in all facilities.

Facility Managers and Key Participants ensure that all participants have completed attendance and critique documentation.

****

THIS IS A DRILL

****

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR POWER PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

4.2 MEDICAL SCENARIO AND MESSAGES

-- NOTE --

This Section contains data on the simulated contaminated injury including: inplant, transport and treatment at the off-site medical treatment facility (Martin Memorial Hospital).

CONTENTS

4.2.1 SCENARIO AND ANTICIPATED ACTIONS:

GENERAL SCENARIO:

A Radwaste Operator (simulated by a volunteer victim) was working in the Drumming Room of the Unit 1 RAB when a lid on a B25 box slipped from the top of the box where it was sitting loosely and knocked the worker to the floor. As the lid fell, it knocked the victim over and the corner of the lid ripped open the right sleeve of the victim's protective clothing, gashing the right outer forearm between elbow and wrist. The victim was bowled over by the falling lid, striking their head soundly on the floor and was knocked semi-conscious. The victim is stunned and bleeding and a coworker (simulated by the Medical Controller) helped the worker out of the Drumming Room where the victim slumped to the floor and the medical emergency scenario begins with a notification telephone call to the Unit 1 Control Room.

DETAILED SCENARIO AND ACTIONS:

SCENARIO	ACTIONS
The medical scenario begins with the volunteer victim lying on their side in the Unit 1 RAB outside the Drumming Room. The victim is wearing protective clothing. The victim is semi-conscious and bleeding from the right forearm. Bleeding is apparent through the PC sleeve.	The Medical Controller or a conscripted Player will call in the injury information to the Unit 1 Control Room, initiating the medical emergency scenario.
Initial condition	The victim is semi-conscious and unresponsive to inquiry.
Respiration: 24	
Pulse: 85	
Skin: Pale, and wet with perspiration.	
Pupils: Dilated	

DETAILED SCENARIO AND ACTIONS: (Continued)

SCENARIO	ACTIONS
Plant First-Aid/Decon Team Arrival Respiration: 24 Pulse: 85 B/P: 140/96 Skin: Pale and wet with perspiration. Pupils: Equal. The Patient is conscious, but is weak, dizzy and unable to sit up. The victim complains of headache, a feeling of vertigo and nausea. There is copious bleeding from the torn right forearm.	Communications are established. Vital signs and patient condition are assessed. Protective clothing is removed to facilitate treatment of the wound. A pressure dressing is applied to the laceration. The victim is kept in a supine condition and remaining protective clothing is cut away. Psychological support is given.

DETAILED SCENARIO AND ACTIONS: (Continued)

Radiological evaluation:

General area radiation levels:

At Drumming Room door threshold (door open):

2 mr/hr

airborne:

<MDA

smearable:

5000 cpm >Bkg

(door closed):

1 mr/hr

airborne:

<MDA

smearable:

5000 cpm >Bkg

Patient's SRD:

10 mr

While triage is being administered to the patient, Health Physics Technician should be assessing the general area radiological conditions and evaluating the contamination levels of the victim.

DETAILED SCENARIO AND ACTIONS: (Continued)

SCENARIO	ACTIONS
Radiological Evaluation:(Continued) When moved to a lower background area: **Patient:** PC's (general): 5000 cpm >Bkg right sleeve: 3000 cpm >Bkg Gloves: 5000 cpm >Bkg **After PC removal:** Right forearm, wound area: 2500 cpm >Bkg Face, left cheek: 1000 cpm >Bkg Right Hand: 5000 cpm >Bkg Left Knee: 2000 cpm >Bkg	In general, radiation, contamination and airborne activity outside of the immediate pumproom area will be AS READ.

DETAILED SCENARIO AND ACTIONS: (Continued)

SCENARIO	ACTIONS
Initial Treatment: (Continued) During the initial treatment and preparation for transport: Respiration: 36, shallow, irregular. Pulse: 110 B/P: 148/98 Skin: Pale, cool diaphoretic Pupils: Equal Patient is still complaining of headache and dizziness, is becoming less lucid, less responsive.	Notification of patient condition and contamination should be made to the Control Room and the Emergency Coordinator. Due to the possibility of a cranial injury, notification to off-site treatment facility should be made and preparations for ambulance transport begun. The patient should be transported by stretcher to meet the ambulance and contamination control appropriate to the medical priorities should be exercised as the patient crosses the Radiation Control Area (RCA) line.

DETAILED SCENARIO AND ACTIONS: (Continued)

SCENARIO	ACTIONS
Radiological Evaluation: (Continued) After initial decon attempts: Right forearm, wound area: 1500 cpm >Bkg Face, left cheek: 100 cpm >Bkg Right Hand: 100 cpm >Bkg Left Knee: Background	The initial decontamination attempts performed either in the RAB or in the ambulance will reduce the patient contamination levels to those indicated. These levels will remain unchanged until the patient reaches the off-site treatment facility.
Secondary Treatment: (Martin Memorial Hospital) Respiration: 38, irregular. Pulse: 110 B/P: 186/100 Skin: Pale, clammy, moist. Pupils: Equal, responsive. Patient is still disoriented, is getting stronger and more responsive.	Medical evaluation finds the forearm laceration to be approximately 18 cm in length and not deep. There is considerable venous bleeding, no apparent tendon/muscle damage. The patient is becoming more reactive to inquiry and stimuli, there is the possibility of a subdural hematoma as a result of the head impact upon falling. Blood loss is controlled.

DETAILED SCENARIO AND ACTIONS: (Continued)

SCENARIO	ACTIONS
Secondary Treatment: (Off-Site Medical, Continued))	In the Hospital, the patient's remaining clothing is removed. Appropriate medical and nursing treatment are initiated. As determined by the Physician, radiological surveys are performed and samples collected. All surveys should be correctly documented, all samples properly labeled.
Contamination levels:	
Wound: 1500 cpm >Bkg.	
Face and Cheek: 100 cpm >Bkg.	Priorities are established for decontamination. Appropriate techniques are utilized for decontamination. Surveys are performed and documented during the decontamination process. The Health Physics Technician makes recommendations and properly maintains control of waste. Area background levels are maintained as low as possible.
Right hand: 100 cpm >Bkg.	
After 1st Decon:	
Wound: 500 cpm >Bkg.	
Face and Cheek: Background.	
Right hand: 100 cpm >Bkg.	

DETAILED SCENARIO AND ACTIONS: (Continued)

SCENARIO	ACTIONS
After 2nd Decon: Wound: 200 cpm >Bkg. After 3rd Decon: Wound: Background.	During the treatment, the wound will continue to exhibit measurable radiation readings in spite of decontamination efforts until the third simulated decontamination attempt. The removal of this contamination essentially completes the treatment and decontamination phase of the scenario.

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: First-Aid/Decon Team

MESSAGE NO.: M - 1 TIME: 0905 (Approx.)

LOCATION: Unit 1 RAB, Drumming Room Entrance

MESSAGE:

Controller Note: Provide the following **information only** to the First-Aid/Decon team members as they earn it by performing the appropriate action.

Vitals:

Respiration:	24
Pulse:	85
B/P:	140/96
Skin:	Pale and wet with perspiration.
Pupils:	Equal.

Survey results of injured Operator:

PC's (general):	5000 cpm >Bkg
Right sleeve:	3000 cpm >Bkg
Gloves:	5000 cpm >Bkg
SRD:	10 mR

After PC's are removed:

Right forearm, wound area:	2500 cpm >Bkg
Face, left cheek:	1000 cpm >Bkg
Right hand:	5000 cpm >Bkg
Left Knee:	2000 cpm >Bkg

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: First-Aid/Decon Team

MESSAGE NO.: M - 2 TIME: 0915 (Approx.)

LOCATION: Unit 1 RAB

MESSAGE:

Controller Note: Provide the following **information only** to the First-Aid/Decon team members as they earn it by performing the appropriate action.

Vital Signs while preparing for transport:

Respiration:	36, shallow, irregular.
Pulse:	110
B/P:	148/98
Skin:	Pale, cool and diaphoretic
Pupils:	Equal, responsive

Patient is still complaining of headache and dizziness, is becoming less lucid, less responsive.

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: First-Aid/Decon Team

MESSAGE NO.: M - 3 TIME: 0945 (Approx.)

LOCATION: Ambulance Loading Point

MESSAGE:

Controller Note: Provide the following **information only** to the First-Aid/Decon team members as they earn it by performing the appropriate action.

Initial decontamination efforts (if performed) yield the following results:

Right forearm, Wound area:	1500 cpm >Bkg
Left Knee:	Background
Face, left cheek:	100 cpm >Bkg
Right hand:	100 cpm >Bkg

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Ambulance Medical Team

MESSAGE NO.: M - 4 TIME: 1000 (Approx.)

LOCATION: Ambulance Loading Point, Enroute to Martin Memorial

MESSAGE:

When the patient is examined and evaluated by Ambulance Medical Team and the entire transport to the Martin Memorial Hospital, provide the following **information only** as it is earned by the ambulance medical team's actions:

Vitals:

Respiration:	43, irregular.
Pulse:	116
B/P:	186/100
Skin:	Pale, clammy, moist
Pupils:	Equal, responsive

Patient is still disoriented, is responsive to painful stimuli but unable to communicate lucidly.

Wound:

There is a shallow laceration outside the right forearm between elbow and wrist approximately 18 cm in length. There is considerable venous bleeding and no evidence of any possible tendon/nerve damage.

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Health Physics Technician

MESSAGE NO.: M - 5 TIME: 1030 (Approx)

LOCATION: Martin Memorial Hospital

MESSAGE:

After the transfer of the patient from the ambulance into the treatment area, clearing of ambulance protective coverings and proper removal of protective clothing by ambulance personnel--survey/status of the ambulance and attendants is:

Ambulance: All Areas Background

Transfer Route: All Areas Background

Ambulance Team Members: All Areas Background

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Medical Team

MESSAGE NO.: M - 6 TIME: 1030 (Approx.)

LOCATION: Martin Memorial Hospital

MESSAGE:

When the patient is examined and evaluated by Medical Team:

Vitals:

Respiration:	38, irregular.
Pulse:	110
B/P:	186/100
Skin:	Pale, clammy, moist
Pupils:	Equal, responsive

Patient is still disoriented, is getting stronger and more responsive.

Wound:

There is a shallow laceration outside the right forearm between elbow and wrist approximately 18 cm in length. There is considerable venous bleeding and no evidence of any possible tendon/nerve damage. The patient is becoming more reactive to inquiry and stimuli and there may be an injury to the head caused by the fall.

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Medical Team

MESSAGE NO.: M - 7 TIME: 1045 (Approx.)

LOCATION: Martin Memorial Hospital

MESSAGE:

Additional patient status during examination:

Vitals:

Pulse:	100
Respiration:	28
B/P:	160/104
Skin:	Pale, clammy, moist

Patient is weak, slightly disoriented, but responsive and can answer inquiries.

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Health Physics Technician

MESSAGE NO.: M - 8 TIME: 1050 (Approx.)

LOCATION: Martin Memorial Hospital

MESSAGE:

Controller Note: Provide the following **information only** to the First-Aid/Decon team members as they earn it by performing the appropriate action.

Additional radiological survey results:

Face and cheek:	100 cpm >Bkg
Right hand:	100 cpm >Bkg
Wound:	1500 cpm >Bkg

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Health Physics Technician

MESSAGE NO.: M - 9 TIME: 1100 (Approx.)

LOCATION: Martin Memorial Hospital

MESSAGE:

Post-Decon radiological survey results after the first decon:

Face and cheek:	Background
Right hand:	100 cpm >Bkg
Wound:	500 cpm >Bkg

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Health Physics Technician

MESSAGE NO.: M - 10 TIME: 1115 (Approx.)

LOCATION: Martin Memorial Hospital

MESSAGE:

Post-Decon radiological survey results after the second decon:

Right hand:	Background
Wound:	200 cpm >Bkg

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Health Physics Technician

MESSAGE NO.: M - 11 TIME: 1130 (Approx.)

LOCATION: Martin Memorial Hospital

MESSAGE:

Post-Decon radiological survey results after third decon:

Wound: Background

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Medical Team

MESSAGE NO.: M - 12 TIME: 1140 (Approx.)

LOCATION: Martin Memorial Hospital

MESSAGE:

The third wound irrigation completes decontamination of the patient. All surveys now indicate Background. You may now evaluate release of the patient to conventional treatment areas.

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Health Physics Technician

MESSAGE NO.: M - 13 TIME: 1150 (Approx)

LOCATION: Martin Memorial Hospital

MESSAGE:

After the transfer of the patient, proper exit and removal of protective clothing by Medical personnel--survey/status is:

Patient:	All Areas Background
Stretcher:	All Areas Background
Medical Team Members:	All Areas Background

MEDICAL EMERGENCY MESSAGE FORM

THIS IS A DRILL

TO: Medical Team

MESSAGE NO.: M - 14 TIME: 1200 (Approx.)

LOCATION: Martin Memorial Hospital

MESSAGE:

The Radiological Medical Emergency Drill is terminated. An in-place critique and discussion will take place.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

4.3 MINI SCENARIOS

This scenario section contains detailed mini-scenarios for the following events:

4.3.1	Fire in the "1B5" 480 Volt Motor Control Center/Loss of the "1B3" 4160 Volt Switchgear
4.3.2	"1A" AFW Pump Maintenance and Recovery
4.3.3	Cross-Feed of the "1B2" 480 Volt Bus
4.3.4	Public Information/Media Mini-Scenario

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

4.3 MINI SCENARIOS

GENERAL REPAIR TEAM CONTROLLER INFORMATION

DO NOT ALLOW PLAYERS TO PHYSICALLY MOVE/CHANGE VALVES, SWITCHES, OR COMPONENT STATUS IN RESPONSE TO SIMULATED SCENARIO EVENTS. The actual movement or alteration of equipment and controls will affect Unit safety and operation and shall not be allowed in response to simulated scenario events. Players should indicate components to be altered and actions to be taken and then given information as to the response to the simulated action.

Do not allow players to over-simulate response actions or skip over time required for response. If component disassembly is to be simulated, the necessary tools must be obtained in order to proceed with the simulation. If replacement of parts is to be simulated, the actual procurement of the replacement part from stores should be accomplished in order to proceed with the simulation.

The following specific task information packages are designed to allow repair team controllers to provide sufficient feedback to players responding to simulated equipment casualties in the Exercise scenario. Provide information from the appropriate section to the repair teams as they EARN it by performing investigation and troubleshooting. Do not allow the Players to "over simulate" their response actions and thereby drastically reduce the time required for actual performance of the simulated task. Be sensitive to the frustration of the players common in situations of excessive simulation and try to avoid the loss of drillsmanship as a consequence of this impediment to their efforts.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

4.3 MINI SCENARIOS

This scenario section contains detailed mini-scenarios for the following events:

4.3.1	Fire in the "1B5" 480 Volt Motor Control Center/Loss of the "1B3" 4160 Volt Switchgear
4.3.2	"1A" AFW Pump Maintenance and Recovery
4.3.3	Cross-Feed of the "1B2" 480 Volt Bus
4.3.4	Public Information/Media Mini-Scenario

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

4.3 MINI SCENARIOS

GENERAL REPAIR TEAM CONTROLLER INFORMATION

DO NOT ALLOW PLAYERS TO PHYSICALLY MOVE/CHANGE VALVES, SWITCHES, OR COMPONENT STATUS IN RESPONSE TO SIMULATED SCENARIO EVENTS. The actual movement or alteration of equipment and controls will affect Unit safety and operation and shall not be allowed in response to simulated scenario events. Players should indicate components to be altered and actions to be taken and then given information as to the response to the simulated action.

Do not allow players to over-simulate response actions or skip over time required for response. If component disassembly is to be simulated, the necessary tools must be obtained in order to proceed with the simulation. If replacement of parts is to be simulated, the actual procurement of the replacement part from stores should be accomplished in order to proceed with the simulation.

The following specific task information packages are designed to allow repair team controllers to provide sufficient feedback to players responding to simulated equipment casualties in the Exercise scenario. Provide information from the appropriate section to the repair teams as they EARN it by performing investigation and troubleshooting. Do not allow the Players to "over simulate" their response actions and thereby drastically reduce the time required for actual performance of the simulated task. Be sensitive to the frustration of the players common in situations of excessive simulation and try to avoid the loss of drillsmanship as a consequence of this impediment to their efforts.

4.3.1 FIRE IN THE "1B5" 480 VOLT MOTOR CONTROL CENTER / LOSS OF THE "1B3" 4160 VOLT SWITCHGEAR

I. Description

A short-circuit condition occurring in the "1B5" 480 Volt Motor Control Center (MCC) in the "B" Switchgear Room of the Unit 1 Reactor Auxiliary Building (RAB) results in the simulated discoloration of MCC cubicles, severe damage to MCC cabling and bussing and a fire burning within the cubicles and overhead cable trays. The initial grounding and arcing condition disrupts and severely damages the MCC and its internals. As a result of the fire, cabling and insulation in the overhead conduits and cable trays are burnt and produce heavy black smoke. The initial fire in the "1B5" MCC damages only the "1B5" and its load wiring. However, as a result of heat and firefighting efforts in close proximity to the "1B3" 4160 Volt Switchgear, a subsequent faulting and loss of the "1B3" switchgear will occur at 0930.

II. Initial Conditions

The fire controller should be standing by in the vicinity of the "1B5" 480 Volt MCC in the Unit 1 "B" Switchgear Room in order to provide as-found conditions to the Fire Team upon their arrival. The scenario will begin with the presentation of fire alarm signals to the Nuclear Plant Supervisor (NPS) in the Unit 1 Control Room indicating the presence of smoke and heat in the Unit 1 "B" Switchgear Room in conjunction with the loss of the "1B5" 480 Volt MCC and supplied loads electrically .

4.3.1 FIRE IN THE "1B5" 480 VOLT MOTOR CONTROL CENTER / LOSS OF THE "1B3" 4160 VOLT SWITCHGEAR

III. Scenario

Time:	Event:
0800	Fire alarms given in the Unit 1 Control Room (given by Controller message, Master Fire Panel Delta point number 1.07.01, Unit 1 Zone 7 A, and Delta point number 1.07.02, Unit 1 Zone 7 B) indicating heat and smoke in the area of the "B" Switchgear Room, Unit 1 RAB. Loss of the "1B5" 480 Volt MCC electrically.

Notes:

Operators should dispatch Senior Nuclear Plant Operator (SNPO) to investigate the alarms. Fire Controller should be staged at the vicinity of the 1B5 Motor Control Center in the "B" Switchgear Room of the Unit 1 RAB to meet responding SNPO/Fire Team. An Electrical Maintenance Controller may be staged in the Operations Support Center (OSC) in order to respond with any Player maintenance teams dispatched. Electrical Maintenance Controller could support the Fire Controller onscene to cover the assessment actions taken after the fire is out.

0805 (Approx.)	SNPO should have responded to the "B" Switchgear Room on the 43' elevation of the Unit 1 RAB and discovered heavy black smoke in the room caused by the "1B5" Motor Control Center.

Notes:

The responding Player will find heavy black smoke. Fire is burning primarily in the cable tray above the motor control center.

The fire in the cable tray is reluctant to firefighting activity until the simulated request for off-site fire assistance at 0830. The last remaining slow-fire sources in the cable insulation will be out with the simulated arrival and support of off-site fire assistance by approximately 0845. As the Fire Team sets up agents and makes entry, the smoke will begin to lift and be primarily ceiling coverage. A search and rescue sweep of the room will find no lost or injured personnel in the fire area.

0845 (Approx.)	Any application of fire fighting agent by this time will have extinguished the fires.

4.3.1 FIRE IN THE "1B5" 480 VOLT MOTOR CONTROL CENTER / LOSS OF THE "1B3" 4160 VOLT SWITCHGEAR

III. Scenario (Continued)

Time:	Event:
0930	Auxiliary Feed Actuation Signal (AFAS) attempts to start the "1B" Auxiliary Feed Water (AFW) Pump. As a result of heat and firefighting efforts in the nearby "1B5" MCC, faulted relays and damaged current transformers (CTs) cause the loss of the "1B3" 4160 Volt Switchgear.

Initial Evaluation of Damages:

Cubicles in the "1B5" 480 Volt MCC are discolored, soot covered and severely burnt.

There are doors and access panels blackened by smoke on the MCC in several locations on both sides.

There is internal fire damage and conductor loss in several cubicles.

The entire overhead cable tray is destroyed by fire. All the cabling and insulation is charred and useless. The damages to the "1B5" MCC are anticipated to prevent recovery of "1B5" loads for the duration of the exercise.

Heat damage and exposure to fire fighting activities in the switchgear and cable trays adjacent to the "1B3" 4160 Volt Switchgear has resulted in damage to the degraded grid/undervoltage relays. When the "1B" AFW pump attempts to start on the AFAS signal, it trips on differential action. The "1B" Emergency Diesel Generator (EDG) autostarted on low voltage on the "1B3" bus, but never closes onto the bus. Failed Current Transformers (CTs) in the diesel cubicle and bus degradation are anticipated to prevent recovery of the "1B3" 4160 Volt bus for the duration of the exercise.

4.3.1 FIRE IN THE "1B5" 480 VOLT MOTOR CONTROL CENTER / LOSS OF THE "1B3" 4160 VOLT SWITCHGEAR

Detailed Evaluation of Damages:

"1B5" 480 Volt MCC:

The fire in the "1B5" MCC has originated in several load modules near the center of the MCC. These controllers are destroyed. Fire has spread along insulation and other conductors damaging them and the internal wiring before spreading to cable trays above the MCC. The cable trays are filled with burnt and carbonized insulation. Conductors are bare of insulation, discolored and twisted.

"1B3" 4160 Volt Switchgear

A multiple failure of the degraded grid/undervoltage relays is producing a complex fault condition. A differential action has occurred on the startup of the "1B" AFW pump as a result of heat damage to the "1B" diesel CTs in the end of the switchgear most closely exposed to the "1B5" fire. The multiple failures of contaminated and heat-damaged circuitry in the switchgear has resulted in loss of stability and inability to reenergize the "1B3" 4160 Volt Switchgear or its individual loads.

4.3.2 "1A" AUXILIARY FEED WATER PUMP MAINTENANCE

I. Description

Ongoing maintenance on the "1A" Auxiliary Feed Water (AFW) Pump is postulated to be occurring at the beginning of the Exercise scenario. The subsequent unrecoverable loss of the "1B3" 4160 Volt Switchgear renders the "1B" AFW Pump powerless. The lockup of the turbine bearings on the "1C" AFW pump disables it. As a result, the recovery of the "1A" AFW pump is not only important to the restoration of Steam Generator (S/G) feed, but is mandated by scenario events and will occur at 1200 by Contingency Message if necessary. The anticipated recovery attempts are considered to be:

1. Reassemble and remount the "1A" AFW Pump Motor.

2. Cross-Over power from the "1A" AFW Pump to the "1B" AFW Pump.

3. "Rob" the "1B" AFW motor and mount it on the "1A" AFW baseplate.

II. Initial Conditions

The "1A" AFW Pump is simulated to be undergoing motor bearing replacement. The motor was tagged-out, disconnected mechanically and electrically and lifted from the pump baseplate. Both endbells have been removed and the coupling-end bearing has been pulled. Due to the maintenance in progress, it may be assumed that all rigging and mounting/demounting tools are in the pumproom. However, any additional tools or materials that the Players wish to utilize in the recovery actions should be ***actually obtained and transported to the area real-time.***

4.3.2 "1A" AUXILIARY FEED WATER PUMP MAINTENANCE

III. Scenario

Time:	Event:
0730	The "1A" AFW Pump has been tagged out and disassembly begun since 1600, February 11, 1992, when the 72 hour LCO clock began.
0930	Major break in the condensate suction header causes loss of Main Condensate Pumps and Main Feed Pumps. Auxiliary Feed Actuation Signal (AFAS) attempts to start the "1B" Auxiliary Feed Water (AFW) Pump. As a result of heat and firefighting efforts in the nearby "1B5" 480 Volt MCC, damaged and faulted relays cause the loss of the "1B3" 4160 Volt Switchgear that supplies power to the "1B" AFW Pump.
0945	Loss of oil in the operating "1C" AFW pump causes the pump turbine bearings to lock up, Total Loss of Feed (TLOF).
1000 (Approx.)	As a result of the TLOF, repair team(s) should be dispatched to attempt and recover the "1A" AFW pump or to investigate restoration of the "1B" AFW Pump.
1145	A Contingency Message will be given (if required) that repair attempts on the "1A" AFW pump are successful and the pump will be available by 1200.
1200	The "1A" AFW pump begins feeding the dry S/G.

4.3.2 "1A" AUXILIARY FEED WATER PUMP MAINTENANCE

IV. Detailed Controller Guidance

The recovery of the "1A" AFW pump is crucial to scenario data fidelity and must occur at 1200. However, the efforts of repair team(s) assigned to the task may warrant the use of simulated delay or the Contingency of accelerated repair. Whether Players choose to reassemble the "1A" motor, cross-connect the "1A" power to the "1B" pump motor or "rob" the "1B" motor, ensure that the simulated activity is performed as realistically as possible and simulation of obtaining parts, tools and work tasks is minimized (with the exception of lifting and slinging equipment which is assumed to be present.) In this regard, any or all of the following items might be utilized for controlling the time interval for recovery:

A baseplate bolt has stripped-out and must be retapped or helicoiled before the motor may be remounted and aligned.

Coupling aligning tools and shims must be actually obtained and brought to the scene.

The coupling lock screw(s) and key on the motor are damaged and must be repaired/replaced before the motor can be recoupled and aligned.

If the Players attempt to energize the "1B" AFW pump from the "1A" pump power feeds, the AFW isolation valves on the "B" side (unpowered as a result of the loss of the "B" side power) may not be opened manually. The motor operator(s) are hydraulically locked by grease in the bellville washer assembly and the motor operator will not declutch. If further action is taken, the valve disk is wedged tight into the seats and cannot be moved.

4.3.3 Cross Feed of the "1B2" 480 Volt Bus

I. Summary:

The lockout and loss of the "1B3" 4160 Volt bus results in the loss of feed to the "1B2" 480 Volt bus and its associated loads. The loss of the "B" side of 480 Volt AC is an important loss due to the many loads associated with it. There is a method of "cross-feeding" the "1B2" 480 Volt bus from the "1A2" 480 Volt bus utilizing the "1AB" bus as a bridge. The recovery of the "1B2" bus and loads is not anticipated during the duration of the scenario and must be prevented in order to maintain scenario data integrity.

The Controller providing information to the team(s) simulating repair actions on this action should provide data in accordance with the following guidelines to allow the Players to perform play action relevant to the scenario.

GUIDANCE

Initial Inspection:

The "1AB" bus supply cabling to the "1B2" 480 Volt Loadcenter is damaged by the recent fire in the nearby "1A5" 480 Volt Motor Control Center. All attempts to jumper and close the bus onto the "1AB" bus will result in an immediate protective action trip. There is heat-damaged and burnt insulation in the "1B2/1AB" connecting cables and breakers.

Anticipated Actions:

1. Attempt to jumper and cross-feed the "1B2" and "1A2" 480 Volt busses through the "1AB" bus.

2. Inspect and test the "1B2/1AB" breakers and supply circuits.

4.3.3 Cross Feed of the "1B2" 480 Volt Bus

Response to Actions:

1. The actual manipulation of the "1A2, 1B2 and 1AB" breakers SHOULD NOT BE PERFORMED; however, the actions required to perform this step should be simulated in a realistic fashion. The conductors and breakers between the "1B2" and the "1AB" 480 Volt busses will trip free. The breakers may not be reclosed without an immediate retrip.

2. Once the proper tools are assembled for simulated opening and testing of the breaker housings and control circuits, the following conditions are found:

There is a hard ground on all three conductors on the "1B2" side of the connections.

Continue to provide information and guidance to the team(s) as required. The "1B2" 480 Volt bus is not anticipated to be recovered during this scenario and *may not be recovered* in order to maintain scenario data fidelity.

4.3.4 Public Information/Media Mini-Scenario

I. Summary:

The fire occurring in the Unit 1 "B" Switchgear room causes media attention as a result of the call for offsite assistance being overheard on scanner. The station has begun making calls to the Corporate Communications Duty Officer requesting information to cover the event.

Media Control "cell" simulating the news desk of the TV station should aggressively seek information from the Corporate Communications Duty Officer by phone.

II. Guidance

Initial Action:
The simulated arrival of the off-site fire assistance group at 0840 should be followed shortly by phone calls requesting information on the event. By 0845, the Media Controller "cell" should be calling the General Office (GO) Corporate Communications Officer requesting information on the situation and activities at the site. At 0905, an actual call for offsite ambulance will occur in response to the emergency medical scenario and this should fuel the media demands for information on the events in progress.

Anticipated Actions:

Media Controller "cell" begins calling the Media information line in the GO requesting immediate details on the fire in Unit 1 from the Corporate Communications Duty Officer.

III. Messages:

The Media Controller "cell" should begin calling the GO Corporate Communications Officer at 0845 and relaying the following messages at the following numbers:

(305) 552-3894
(305) 552-3895

#1:

THIS IS A DRILL, This is the Channel 5 newsdesk. We hear that there has been a fire or explosion or something out there at the St. Lucie Site. There is a fire company responding and we may want to roll a helicopter or mobile crew to get some video of this event. Will you confirm and detail the events in progress? *THIS IS A DRILL*

#2:

THIS IS A DRILL, This is Channel 5 news again. What exactly is the problem? Is there a radiation leak? Why did the problem require outside fire help? We hear that there is a declared emergency of some sort. What public protective measures are being taken? *THIS IS A DRILL*

#3:

THIS IS A DRILL, This is Channel 5. We may want complete access for crews so that we can provide full and live public information on this event. Are you going to allow our mobile crews into your Plant emergency facilities to provide this public information if we dispatch them out there?*THIS IS A DRILL*

#4: (After 0910)

THIS IS A DRILL, This is Channel 5. There's an ambulance ordered for radiation injuries. How many people are injured? Is there a major radiation leak? Are people in danger? . Why haven't you provided this information on a serious problem with exposed and injured workers to the public yet? We are ready to disseminate the information if you will only cooperate. *THIS IS A DRILL*

#5:

THIS IS A DRILL, This is Channel 12. We hear that there is a major radiation accident out at the St. Lucie Plant with dead and injured personnel. We also hear that you are "scooping" this incident to Channel 5. We demand full and equal information. *THIS IS A DRILL*

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

5.0 PLANT PARAMETER DATA

This Scenario Section Contains the Following Information:

Section 5.1	PLANT PARAMETER DATA SUMMARY - A listing of all pertinent plant parameters generated on the PSL simulator for the postulated scenario events.
Section 5.2	PLANT PARAMETER DATA CARDS - The above pertinent plant parameters broken down by time interval for presentation to the Players in the Control Room.
Section 5.3	ANNUNCIATOR MIMIC SHEETS - Mimics of annunciators associated with the plant parameter data timed for concurrent presentation with the Plant Parameter Data Cards.

5.1 PLANT PARAMETER DATA SUMMARY

St. Lucie Nuclear Plant
1992 Evaluated Exercise

	TIME	07:45	08:00	08:05	08:10	08:15	08:30	08:35	08:40	08:45	09:00
NUCLEAR INSTRUMENTATION											
REACTOR POWER (%)											
LINEAR RANGE		100.9	100.9	100.9	100.9	100.9	100.3	98.7	96.5	94.6	87.3
LOG WIDE RANGE		100.2	100.2	100.2	100.2	100.2	99.6	98.0	95.9	94.0	86.8
REACTOR COOLANT SYSTEM (RTGB/QSPDS)											
T AVE (F)											
LOOP A		573.5	573.5	573.5	573.5	573.5	573.3	572.8	572.6	572.4	567.3
LOOPB		573.6	573.6	573.6	573.6	573.6	573.4	572.9	572.8	572.5	567.4
T COLD (F)											
LOOP A1		549.0	549.0	549.0	549.0	549.0	548.8	548.7	549.0	549.3	545.7
LOOP A2		548.7	548.7	548.7	548.7	548.7	548.6	548.4	548.8	549.0	545.4
LOOP B1		549.0	549.0	549.0	549.0	549.0	548.9	548.8	549.1	549.3	545.8
LOOP B2		549.0	549.0	549.0	549.0	549.0	548.9	548.8	549.2	549.4	545.8
T HOT (F)											
LOOP A		598.2	598.2	598.2	598.2	598.2	597.9	597.1	596.4	595.6	589.0
LOOP B		598.3	598.3	598.3	598.3	598.3	597.9	597.1	596.4	595.7	589.1
RCS PRESSURE (PSIA)		2239.1	2235.3	2235.1	2235.0	2234.8	2251.1	2249.0	2250.0	2250.0	2249.9
LOOP FLOW (dP)											
LOOP A1		71.3	71.3	71.3	71.3	71.3	71.3	71.3	71.3	71.3	71.4
LOOP A2		71.3	71.3	71.3	71.3	71.3	71.3	71.3	71.3	71.3	71.4
LOOP B1		71.8	71.8	71.8	71.8	71.8	71.8	71.8	71.7	71.7	71.8
LOOP B2		71.4	71.4	71.4	71.4	71.4	71.4	71.4	71.4	71.4	71.5
PZR LEVEL (%)		63.3	63.3	63.3	63.3	63.3	63.9	63.4	63.9	64.1	59.0
PZR TEMP (F)		652.1	651.8	651.8	651.8	651.8	652.8	652.7	652.7	652.8	652.8
QUENCH TANK LEVEL (%)		63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1
QUENCH TANK TEMP (F)		90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6
QUENCH TANK PRESS (PSIG)		5.6	5.6	5.6	5.6	5.6	5.6	5.6	5.6	5.6	5.6

▒▒▒▒▒▒▒▒=UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

TIME	07:45	08:00	08:05	08:10	08:15	08:30	08:35	08:40	08:45	09:00
CHEMICAL & VOLUME CONTROL SYS										
LETDOWN FLOW (GPM)	38.4	38.1	38.9	37.6	38.8	76.5	70.2	76.7	78.4	64.8
VCT LEVEL (%)	56.7	56.7	56.7	56.7	56.7	57.1	58.2	58.7	59.6	36.2
CHARG. FLOW (GPM)	44.0	44.0	44.0	44.0	44.0	87.9	87.9	87.8	87.9	87.9
CHRG PUMP (ON/OFF)										
A	ON	ON	ON	ON	ON	ON	ON	ON	ON	ON
B	OFF	OFF	OFF	OFF	OFF	ON	ON	ON	ON	ON
C	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF
BORONOMETER (PPM)	668.5	668.5	668.5	668.5	668.5	669.2	670.3	671.8	673.7	680.8
CORE COOLING (QSPDS)										
SUBCOOLED MONITOR (F)										
A	53.7	53.5	53.5	53.5	53.4	54.8	55.5	56.3	57.0	63.6
B	53.7	53.5	53.5	53.5	53.4	54.8	55.5	56.3	57.0	63.6
CORE EXIT THERMOCOUPLE (F)										
HIGHEST CET										
A	600.4	600.8	600.9	600.8	600.9	600.2	599.4	598.6	598.3	591.3
B	600.7	600.8	600.7	600.7	600.9	600.5	599.6	598.8	598.1	591.4
REPRESENTATIVE CET										
A	612.2	612.1	612.5	612.4	612.2	611.7	610.8	610.2	610.2	602.7
B	612.1	612.1	612.6	612.6	612.5	611.9	611.0	610.2	609.8	602.6
RX VESSEL LEVEL (%)										
A	100	100	100	100	100	100	100	100	100	100
B	100	100	100	100	100	100	100	100	100	100
CONTAINMENT										
PRESSURE (PSIG)	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
TEMP (F)	99.8	99.4	99.3	99.2	99.1	98.9	98.8	98.7	98.7	98.5
SUMP LEVEL (FT)										
WIDE RANGE	-1.0	-1.0	-1.0	-1.0	-1.0	-1.0	-1.0	-1.0	-1.0	-1.0
NARROW RANGE	-6.3	-6.4	-6.4	-6.4	-6.5	-6.5	-6.6	-6.6	-6.6	-6.7
CONT. H2 (%)	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

[ ] =UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

TIME	07:45	08:00	08:05	08:10	08:15	08:30	08:35	08:40	08:45	09:00
ENGINEERED SAFETY FEATURES										
HPSI LOOP FLOW (GPM)										
A1	0	0	0	0	0	0	0	0	0	0
A2	0	0	0	0	0	0	0	0	0	0
B1	0	0	0	0	0	0	0	0	0	0
B2	0	0	0	0	0	0	0	0	0	0
LPSI LOOP FLOW (GPM)										
A1	0	0	0	0	0	0	0	0	0	0
A2	0	0	0	0	0	0	0	0	0	0
B1	0	0	0	0	0	0	0	0	0	0
B2	0	0	0	0	0	0	0	0	0	0
CONT. SPRAY PUMP FLOW (GPM)										
A	0	0	0	0	0	0	0	0	0	0
B	0	0	0	0	0	0	0	0	0	0
SIT LEVEL (%)										
A1	86.0	86.0	86.0	86.0	86.0	86.0	86.0	86.0	86.0	86.0
A2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
B1	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
B2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
SIT PRESSURE (PSIG)										
A1	213.5	213.5	213.5	213.5	213.5	213.5	213.5	213.5	213.5	213.5
A2	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6
B1	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8
B2	214.1	214.1	214.1	214.1	214.1	214.1	214.1	214.1	214.1	214.1

[ ]=UNPOWERED/INVALID DATA POINT

	TIME	07:45	08:00	08:05	08:10	08:15	08:30	08:35	08:40	08:45	09:00
STEAM GENERATORS											
SG LEVEL WR (%)											
	A	58.8	58.8	58.8	58.8	58.8	58.4	58.8	59.2	59.1	59.3
	B	58.6	58.6	58.6	58.6	58.6	58.4	58.9	59.2	59.1	59.3
SG LEVEL NR (%)											
	A	63.1	63.1	63.1	63.1	63.1	61.4	63.2	64.5	64.0	63.7
	B	63.4	63.4	63.4	63.4	63.4	62.7	64.4	65.6	65.2	64.9
SG PRESSURE (PSIG)											
	A	858.1	858.1	858.1	858.1	858.1	858.5	858.4	863.9	868.3	848.9
	B	858.2	858.2	858.2	858.2	858.2	858.7	858.5	864.0	868.4	849.0
FEED FLOW (LB/HR)											
	A	5830250	5830460	5829920	5830110	5830620	5807810	5729040	5583050	5442340	5040900
	B	5843530	5843300	5843550	5843680	5844560	5821470	5745250	5596310	5455480	5055520
STEAM FLOW (LB/HR)											
	A	5809590	5809700	5809710	5809700	5809700	5759990	5676400	5538910	5417740	5008040
	B	5823220	5823300	5823340	5823330	5823340	5774020	5690260	5553080	5431590	5021370
SG BLOW DOWN FLOW (GPM)											
	A	10	10	10	10	10	10	10	10	10	10
	B	15	15	15	15	15	15	15	15	15	15
CONDENSER BACK PRESS (in.		2.6	2.6	2.6	2.6	2.6	2.6	2.6	2.5	2.5	2.4
ELECTRICAL PLANT											
EDG LOAD (MW)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
4160V VITAL BUS (VOLTS)											
	1A3	4126.3	4126.2	4125.5	4127.1	4125.5	4126.9	4128.3	4132.3	4136.7	4146.6
	1B3	4133.4	4133.2	4135.2	4135.2	4135.1	4134.1	4136.4	4140.3	4143.8	4154.7
6.9KV SWGR (VOLTS)											
	1A1	6966.4	6966.3	6966.3	6966.4	6966.2	6968.8	6972.8	6979.7	6985.8	7004.4
	1B1	6967.1	6967.1	6967.2	6967.2	6967.2	6969.5	6973.7	6980.6	6986.6	7005.3

=UNPOWERED/INVALID DATA POINT

	TIME	07:45	08:00	08:05	08:10	08:15	08:30	08:35	08:40	08:45	09:00
AUXILIARY SYSTEMS											
AFW FLOW (GPM)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
	C	0	0	0	0	0	0	0	0	0	0
TANK LEVEL											
RWT (FT)		35.0	35.0	35.0	35.0	35.0	35.0	35.0	35.0	35.0	35.0
CST (FT)		26.2	26.2	26.2	26.2	26.2	26.2	26.2	26.2	26.2	26.1
PWT (FT)		19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8
BAMT (%)											
	A	91.9	91.9	91.9	91.9	91.9	90.9	90.4	89.9	89.4	87.9
	B	92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4
COOLING WATER											
CCW HEADER (GPM)											
	A	6551	6542	6536	6531	6573	6559	6564	6549	6549	6545
	B	6978	6980	6979	6952	6976	6977	6986	6982	6990	6973
ICW HEADER (PSIG)											
	A	41	41	41	41	41	41	41	41	42	43
	B	41	41	41	41	41	41	41	41	42	43
CONDENSER HOTWELL LEVEL (FT)		2.8	2.8	2.8	2.8	2.8	2.8	2.8	2.7	2.7	2.7

[redacted]=UNPOWERED/INVALID DATA POINT

TIME	09:15	09:30	09:35	09:40	09:45	09:50	09:55	10:00	10:15	10:20
NUCLEAR INSTRUMENTATION										
REACTOR POWER (%)										
LINEAR RANGE	75.3	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOG WIDE RANGE	74.9	4.05E-03	8.84E-05	2.10E-06	1.13E-07	6.79E-08	6.61E-08	6.58E-08	5.57E-08	5.07E-08
REACTOR COOLANT SYSTEM (RTGB/QSPDS)										
T AVE (F)										
LOOP A	562.1	545.5	536.9	531.1	536.3	529.7	523.4	517.5	445.4	425.6
LOOPB	562.2	545.7	537.1	531.2	536.8	530.1	523.7	517.8	445.8	427.0
T COLD (F)										
LOOP A1	543.1	545.0	536.3	530.0	524.9	519.7	513.7	507.9	428.9	411.0
LOOP A2	542.8	544.7	536.0	529.9	527.3	520.6	514.7	508.8	428.7	409.7
LOOP B1	543.2	545.1	536.4	530.2	524.9	519.7	513.7	507.9	429.6	412.6
LOOP B2	543.2	545.1	536.5	530.2	527.4	520.7	514.7	508.9	429.4	412.8
T HOT (F)										
LOOP A	581.2	546.2	537.7	532.2	546.5	539.3	532.6	526.7	462.0	440.9
LOOP B	581.2	546.3	537.9	532.2	547.4	540.0	533.2	527.2	462.0	441.3
RCS PRESSURE (PSIA)	2250.0	2199.5	2111.7	2113.3	1880.3	1659.5	1467.5	1298.2	502.8	389.3
LOOP FLOW (dP)										
LOOP A1	71.4	70.8	71.0	5.3	0.0	0.0	0.0	0.0	0.0	0.0
LOOP A2	71.4	70.8	71.0	5.4	0.0	0.0	0.0	0.0	0.0	0.0
LOOP B1	71.8	71.2	71.4	5.5	0.0	0.0	0.0	0.0	0.0	0.0
LOOP B2	71.5	70.9	71.1	5.3	0.0	0.0	0.0	0.0	0.0	0.0
PZR LEVEL (%)	55.3	42.2	34.4	29.9	36.6	35.9	35.1	34.4	100.0	100.0
PZR TEMP (F)	652.8	649.3	644.0	642.4	628.3	610.8	594.2	578.1	469.9	447.7
QUENCH TANK LEVEL (%)	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1
QUENCH TANK TEMP (F)	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6
QUENCH TANK PRESS (PSIG)	5.6	5.6	5.6	5.6	5.6	5.6	5.6	5.6	1.4	3.0

=UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

TIME	09:15	09:30	09:35	09:40	09:45	09:50	09:55	10:00	10:15	10:20
CHEMICAL & VOLUME CONTROL SYS										
LETDOWN FLOW (GPM)	75.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8
VCT LEVEL (%)	34.9	40.6	40.6	40.6	40.6	40.6	40.6	40.6	40.6	40.6
CHARG. FLOW (GPM)	87.9	44.0	44.2	43.7	44.8	44.8	44.9	45.0	83.1	79.7
CHRG PUMP (ON/OFF)										
A	ON	ON	ON	ON	ON	ON	ON	ON	ON	ON
B	ON	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF
C	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF	ON	ON
BORONOMETER (PPM)	694.3									
CORE COOLING (QSPDS)										
SUBCOOLED MONITOR (F)										
A	71.7	103.2	105.8	111.4	80.8	70.7	60.8	50.6	5.5	14.2
B	71.7	103.2	105.8	111.4	80.8	70.7	60.8	50.6	5.5	14.2
CORE EXIT THERMOCOUPLE (F)										
HIGHEST CET										
A	582.8	546.5	537.8	532.5	548.3	540.9	534.0	528.0	509.7	508.9
B	582.7	546.4	538.2	532.8	548.4	540.9	534.4	528.6	510.4	511.3
REPRESENTATIVE CET										
A	594.4	558.2	549.3	544.2	559.7	552.2	545.8	539.8	521.6	520.9
B	594.6	558.0	549.8	544.2	559.6	552.3	545.6	540.2	521.8	521.3
RX VESSEL LEVEL (%)										
A	100	100	100	100	100	100	100	100	60	60
B	100	100	100	100	100	100	100	100	60	60
CONTAINMENT										
PRESSURE (PSIG)	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.2	0.8
TEMP (F)	98.2	98.5	100.2	100.4	100.6	100.8	101.0	101.1	103.8	105.6
SUMP LEVEL (FT)										
WIDE RANGE	-1.0	-1.0	-1.0	-1.0	-1.0	-1.0	-1.0	-1.0	-1.0	1.0
NARROW RANGE	-6.8	-6.9	-6.9	-6.9	-6.9	-6.9	-6.9	-6.9	-4.0	0.0
CONT. H2 (%)	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.1

=UNPOWERED/INVALID DATA POINT

TIME	09:15	09:30	09:35	09:40	09:45	09:50	09:55	10:00	10:15	10:20
ENGINEERED SAFETY FEATURES										
HPSI LOOP FLOW (GPM)										
A1	0	0	0	0	0	0	0	0	145	152
A2	0	0	0	0	0	0	0	0	128	134
B1	0	0	0	0	0	0	0	0	145	152
B2	0	0	0	0	0	0	0	0	128	134
LPSI LOOP FLOW (GPM)										
A1	0	0	0	0	0	0	0	0	0	0
A2	0	0	0	0	0	0	0	0	0	0
B1	0	0	0	0	0	0	0	0	0	0
B2	0	0	0	0	0	0	0	0	0	0
CONT. SPRAY PUMP FLOW (GPM)										
A	0	0	0	0	0	0	0	0	0	0
B	0	0	0	0	0	0	0	0	0	0
SIT LEVEL (%)										
A1	86.0	86.0	86.0	86.0	86.0	86.0	86.0	86.0	86.0	86.0
A2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
B1	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
B2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
SIT PRESSURE (PSIG)										
A1	213.5	213.5	213.5	213.5	213.5	213.5	213.5	213.5	213.5	213.5
A2	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6
B1	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8
B2	214.1	214.1	214.1	214.1	214.1	214.1	214.1	214.1	214.1	214.1

=UNPOWERED/INVALID DATA POINT

	TIME	09:15	09:30	09:35	09:40	09:45	09:50	09:55	10:00	10:15	10:20
STEAM GENERATORS											
SG LEVEL WR (%)											
	A	59.5	50.3	45.3	39.5	35.7	31.1	26.7	22.7	7.5	2.3
	B	59.6	50.0	44.8	38.5	35.4	31.0	26.9	22.9	8.0	2.8
SG LEVEL NR (%)											
	A	63.0	33.0	13.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
	B	64.2	32.0	11.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
SG PRESSURE (PSIG)											
	A	845.0	972.5	897.0	866.1	829.4	787.0	746.9	708.2	415.7	278.6
	B	845.1	972.6	897.0	866.1	841.7	793.7	753.6	714.1	414.8	280.0
FEED FLOW (LB/HR)											
	A	4287670	0	0	0	0	0	0	0	0	0
	B	4299090	0	0	0	0	0	0	0	0	0
STEAM FLOW (LB/HR)											
	A	4244660	0	0	0	0	0	0	0	0	0
	B	4256520	0	0	0	0	0	0	0	0	0
SG BLOW DOWN FLOW (GPM)											
	A	10	0	0	0	0	0	0	0	0	0
	B	15	0	0	0	0	0	0	0	0	0
CONDENSER BACK PRESS (in.		2.1	15.0	25.0	29.0	29.0	29.0	29.0	29.0	29.0	29.0
ELECTRICAL PLANT											
EDG LOAD (MW)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
4160V VITAL BUS (VOLTS)											
	1A3	4167.3	4248.6	4249.0	4251.8	4257.1	4257.2	4257.9	4258.2	4252.9	4253.9
	1B3	4174.3	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
6.9KV SWGR (VOLTS)											
	1A1	7037.5	7017.9	7018.6	7022.4	7024.5	7024.5	7024.8	7024.9	7022.3	7022.7
	1B1	7038.3	7032.8	7033.2	7035.6	7035.7	7035.7	7035.7	7035.7	7035.3	7035.3

=UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

		TIME	09:15	09:30	09:35	09:40	09:45	09:50	09:55	10:00	10:15	10:20
AUXILIARY SYSTEMS												
AFW FLOW (GPM)												
	A		0	0	0	0	0	0	0	0	0	0
	B		0	0	0	0	0	0	0	0	0	0
	C		0	0	0	97	0	0	0	0	0	0
TANK LEVEL												
RWT (FT)			35.0	35.0	35.0	35.0	35.0	35.0	35.0	35.0	34.8	34.6
CST (FT)			26.0	25.1	25.1	25.1	25.1	25.1	25.1	25.1	25.1	25.1
PWT (FT)			19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8
BAMT (%)												
	A		86.2	84.7	84.7	84.7	84.7	84.7	84.7	84.7	82.6	80.6
	B		92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4
COOLING WATER												
CCW HEADER (GPM)												
	A		6553	9201	9207	9220	9199	9201	9204	9218	7877	7877
	B		6985	6995	6995	6995	6995	6995	6995	6995	6995	6995
ICW HEADER (PSIG)												
	A		44	49	49	49	49	49	49	48	47	49
	B		44	48	48	48	48	48	48	48	48	48
CONDENSER HOTWELL LEVEL (FT)			2.7	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

=UNPOWERED/INVALID DATA POINT

TIME	10:25	10:30	10:45	11:00	11:05	11:10	11:15	11:30	11:35	11:40
NUCLEAR INSTRUMENTATION										
REACTOR POWER (%)										
LINEAR RANGE	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOG WIDE RANGE	4.69E-08	4.45E-08	3.99E-08	3.85E-08	3.86E-08	3.85E-08	3.82E-08	3.70E-08	3.65E-08	3.61E-08
REACTOR COOLANT SYSTEM (RTGB/QSPDS)										
T AVE (F)										
LOOP A	414.9	413.2	404.9	402.3	406.0	425.5	431.9	446.1	450.2	454.2
LOOPB	415.7	413.7	407.0	405.1	408.9	428.3	434.8	448.9	453.0	457.0
T COLD (F)										
LOOP A1	390.4	388.6	376.6	376.1	383.6	391.1	398.6	421.1	428.6	436.1
LOOP A2	390.6	388.3	376.3	375.8	383.3	390.8	398.3	420.8	428.3	435.8
LOOP B1	391.3	389.5	380.5	382.0	389.5	397.0	404.5	427.0	434.5	442.0
LOOP B2	392.0	388.8	379.8	381.3	388.8	396.3	403.8	426.3	433.8	441.3
T HOT (F)										
LOOP A	439.4	437.9	433.4	428.6	428.6	460.0	465.4	471.2	471.9	472.4
LOOP B	439.8	438.3	433.8	428.6	428.6	460.0	465.4	471.2	471.9	472.4
RCS PRESSURE (PSIA)	388.1	426.4	437.2	376.2	376.2	479.2	400.7	413.9	420.0	374.9
LOOP FLOW (dP)										
LOOP A1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOOP A2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOOP B1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOOP B2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
PZR LEVEL (%)	100.0	100.0	100.0	100.0	100.0	100.0	100.0	100.0	100.0	100.0
PZR TEMP (F)	451.0	452.4	449.8	444.0	447.8	450.1	451.3	454.5	455.2	455.6
QUENCH TANK LEVEL (%)	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1
QUENCH TANK TEMP (F)	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6
QUENCH TANK PRESS (PSIG)	8.9	11.1	13.9	5.3	5.9	11.0	4.8	11.1	12.5	13.4

=UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

TIME	10:25	10:30	10:45	11:00	11:05	11:10	11:15	11:30	11:35	11:40
CHEMICAL & VOLUME CONTROL SYS										
LETDOWN FLOW (GPM)	94.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8
VCT LEVEL (%)	40.6	40.6	40.6	40.6	40.6	40.6	40.6	40.6	40.6	40.6
CHARG. FLOW (GPM)	79.0	79.8	81.1	80.3	80.2	80.1	80.0	79.8	79.8	79.8
CHRG PUMP (ON/OFF)										
A	ON	ON	ON	ON	ON	ON	ON	ON	ON	ON
B	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF
C	ON	ON	ON	ON	ON	ON	ON	ON	ON	ON
BORONOMETER (PPM)										
CORE COOLING (QSPDS)										
SUBCOOLED MONITOR (F)										
A	29.3	49.8	19.6	10.0	10.0	2.7	-20.6	-23.3	-22.6	-34.1
B	29.3	49.8	19.6	10.0	10.0	2.7	-20.6	-23.3	-22.6	-34.1
CORE EXIT THERMOCOUPLE (F)										
HIGHEST CET										
A	507.4	505.9	501.4	496.6	496.6	680.0	700.0	1205.0	1354.0	1370.0
B	509.8	508.3	503.8	498.6	498.6	680.0	700.0	1206.0	1368.0	1370.0
REPRESENTATIVE CET										
A	519.4	517.9	513.4	508.6	508.6	690.0	710.0	1227.0	1380.0	1381.0
B	519.8	518.3	513.8	508.6	508.6	690.0	710.0	1228.0	1381.0	1382.0
RX VESSEL LEVEL (%)										
A	60	50	50	40	20	0	0	0	0	0
B	60	50	50	40	20	0	0	0	0	0
CONTAINMENT										
PRESSURE (PSIG)	1.5	2.4	5.0	5.3	5.3	5.4	5.4	5.6	5.6	5.7
TEMP (F)	112.3	124.7	152.3	150.4	145.5	142.6	142.6	151.5	154.8	157.7
SUMP LEVEL (FT)										
WIDE RANGE	1.2	1.4	2.3	3.0	3.1	3.2	3.5	4.2	4.4	4.7
NARROW RANGE	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CONT. H2 (%)	0.1	0.1	0.1	0.1	0.1	0.1	0.1	1.3	1.3	1.4

=UNPOWERED/INVALID DATA POINT

	TIME	10:25	10:30	10:45	11:00	11:05	11:10	11:15	11:30	11:35	11:40
ENGINEERED SAFETY FEATURES											
HPSI LOOP FLOW (GPM)											
	A1	150	150	151	0	0	0	0	0	0	0
	A2	133	132	133	0	0	0	0	0	0	0
	B1	150	150	151	0	0	0	0	0	0	0
	B2	133	132	133	0	0	0	0	0	0	0
LPSI LOOP FLOW (GPM)											
	A1	0	0	0	0	0	0	0	0	0	0
	A2	0	0	0	0	0	0	0	0	0	0
	B1	0	0	0	0	0	0	0	0	0	0
	B2	0	0	0	0	0	0	0	0	0	0
CONT. SPRAY PUMP FLOW (GPM)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
SIT LEVEL (%)											
	A1	86.0	86.0	86.0	86.0	86.0	86.0	86.0	86.0	86.0	86.0
	A2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
	B1	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
	B2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
SIT PRESSURE (PSIG)											
	A1	213.5	213.5	213.5	213.5	213.5	213.5	213.5	213.5	213.5	213.5
	A2	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6
	B1	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8
	B2	214.1	214.1	214.1	214.1	214.1	214.1	214.1	214.1	214.1	214.1

=UNPOWERED/INVALID DATA POINT

	TIME	10:25	10:30	10:45	11:00	11:05	11:10	11:15	11:30	11:35	11:40
STEAM GENERATORS											
SG LEVEL WR (%)											
	A	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
	B	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
SG LEVEL NR (%)											
	A	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
	B	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
SG PRESSURE (PSIG)											
	A	226.8	87.1	13.5	13.5	13.5	13.5	13.5	13.5	13.5	13.5
	B	227.3	95.9	14.1	14.1	14.1	14.1	14.1	14.1	14.1	14.1
FEED FLOW (LB/HR)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
STEAM FLOW (LB/HR)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
SG BLOW DOWN FLOW (GPM)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
CONDENSER BACK PRESS (In. H		29.0	29.0	29.0	29.0	29.0	29.0	29.0	29.0	29.0	29.0
ELECTRICAL PLANT											
EDG LOAD (MW)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
4160V VITAL BUS (VOLTS)											
	1A3	4252.1	4252.1	4253.9	4257.0	4258.1	4258.1	4257.1	4257.0	4258.1	4257.0
	1B3	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
6.9KV SWGR (VOLTS)											
	1A1	7022.0	7022.0	7022.7	7023.9	7024.3	7024.3	7023.9	7023.9	7024.3	7023.9
	1B1	7035.3	7035.3	7035.3	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4

=UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

		TIME	10:25	10:30	10:45	11:00	11:05	11:10	11:15	11:30	11:35	11:40
AUXILIARY SYSTEMS												
AFW FLOW (GPM)												
	A		0	0	0	0	0	0	0	0	0	0
	B		0	0	0	0	0	0	0	0	0	0
	C		0	0	0	0	0	0	0	0	0	0
TANK LEVEL												
RWT (FT)			34.4	34.2	33.7	33.3	33.3	33.3	33.3	33.3	33.3	33.3
CST (FT)			25.1	25.1	25.1	25.1	25.1	25.1	25.1	25.1	25.1	25.1
PWT (FT)			19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8
BAMT (%)												
	A		78.7	76.8	70.7	64.9	63.0	61.0	58.8	53.0	51.0	49.1
	B		92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4
COOLING WATER												
CCW HEADER (GPM)												
	A		7871	7876	7881	7875	7870	7870	7882	7873	7872	7871
	B		6995	6995	6995	6995	6995	6995	6995	6995	6995	6995
ICW HEADER (PSIG)												
	A		49	48	47	47	47	47	47	47	47	46
	B		48	48	48	48	48	48	48	48	48	48
CONDENSER HOTWELL LEVEL (FT)			0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

=UNPOWERED/INVALID DATA POINT

	TIME	11:45	12:00	12:05	12:10	12:15	12:20	12:25	12:30	12:45	13:00
NUCLEAR INSTRUMENTATION											
REACTOR POWER (%)											
LINEAR RANGE		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOG WIDE RANGE		3.57E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08
REACTOR COOLANT SYSTEM (RTGB/QSPDS)											
T AVE (F)											
LOOP A		458.1	397.3	377.0	370.6	368.3	366.0	363.7	362.7	351.0	342.6
LOOPB		461.0	395.3	375.0	368.6	366.3	364.0	361.7	360.7	349.0	340.6
T COLD (F)											
LOOP A1		443.6	383.0	362.8	356.3	354.1	351.8	349.4	348.5	336.8	328.3
LOOP A2		443.3	402.0	381.8	375.3	373.1	370.8	368.4	367.5	355.8	347.3
LOOP B1		449.5	375.0	354.8	348.3	346.1	343.8	341.4	340.5	328.8	320.3
LOOP B2		448.8	402.0	381.8	375.3	373.1	370.8	368.4	367.5	355.8	347.3
T HOT (F)											
LOOP A		472.8	402.0	381.8	375.3	373.1	370.8	368.4	367.5	355.8	347.3
LOOP B		472.8	402.0	381.8	375.3	373.1	370.8	368.4	367.5	355.8	347.3
RCS PRESSURE (PSIA)		394.6	253.0	200.0	185.0	180.0	175.0	170.0	168.0	145.0	130.0
LOOP FLOW (dP)											
LOOP A1		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOOP A2		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOOP B1		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOOP B2		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
PZR LEVEL (%)		100.0	100.0	100.0	100.0	100.0	100.0	100.0	100.0	100.0	100.0
PZR TEMP (F)		456.7	402.0	381.8	375.3	373.1	370.8	368.4	367.5	355.8	347.3
QUENCH TANK LEVEL (%)		63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1
QUENCH TANK TEMP (F)		90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6
QUENCH TANK PRESS (PSIG)		13.9	5.3	5.9	11.0	4.8	3.0	8.9	11.1	13.9	5.3

=UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

TIME	11:45	12:00	12:05	12:10	12:15	12:20	12:25	12:30	12:45	13:00
CHEMICAL & VOLUME CONTROL SYS										
LETDOWN FLOW (GPM)	94.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8
VCT LEVEL (%)	40.6	40.6	40.6	40.6	40.6	40.6	40.6	40.6	40.6	40.6
CHARG. FLOW (GPM)	79.8	80.2	80.2	80.2	80.2	80.2	80.2	80.2	80.2	80.2
CHRG PUMP (ON/OFF)										
A	ON	ON	ON	ON	ON	ON	ON	ON	ON	ON
B	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF
C	ON	ON	ON	ON	ON	ON	ON	ON	ON	ON
BORONOMETER (PPM)										
CORE COOLING (QSPDS)										
SUBCOOLED MONITOR (F)										
A	-29.5	0.2	-1.7	-0.8	2.5	4.5	-5.2	2.0	2.2	-2.3
B	-29.5	0.2	-1.7	-0.8	2.5	4.5	-5.2	2.0	2.2	-2.3
CORE EXIT THERMOCOUPLE (F)										
HIGHEST CET										
A	1447.0	1473.0	1474.0	1465.0	443.1	440.8	438.4	437.5	425.8	417.3
B	1446.0	1473.0	1473.0	1464.0	443.1	440.8	438.4	437.5	425.8	417.3
REPRESENTATIVE CET										
A	1463.0	1485.0	1484.0	1477.0	453.1	450.8	448.4	447.5	435.8	427.3
B	1464.0	1486.0	1485.0	1476.0	453.1	450.8	448.4	447.5	435.8	427.3
RX VESSEL LEVEL (%)										
A	0	0	0	0	10	20	40	40	70	80
B	0	0	0	0	10	20	40	40	70	80
CONTAINMENT										
PRESSURE (PSIG)	5.7	6.4	6.5	6.5	6.6	6.6	6.7	6.7	6.9	7.0
TEMP (F)	160.2	170.8	170.9	171.1	171.2	171.3	171.5	171.6	172.0	172.3
SUMP LEVEL (FT)										
WIDE RANGE	4.9	5.5	5.7	5.8	6.0	6.1	6.3	6.4	6.9	9.3
NARROW RANGE	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CONT. H2 (%)	1.4	1.8	1.8	1.8	1.8	1.8	1.8	1.8	1.8	1.8

=UNPOWERED/INVALID DATA POINT

TIME	11:45	12:00	12:05	12:10	12:15	12:20	12:25	12:30	12:45	13:00
ENGINEERED SAFETY FEATURES										
HPSI LOOP FLOW (GPM)										
A1	0	0	0	0	0	0	0	0	0	0
A2	0	0	0	0	0	0	0	0	0	0
B1	0	0	0	0	0	0	0	0	0	0
B2	0	0	0	0	0	0	0	0	0	0
LPSI LOOP FLOW (GPM)										
A1	0	0	0	225	300	400	475	500	825	1000
A2	0	0	0	225	300	400	475	500	825	1000
B1	0	0	0	0	0	0	0	0	0	0
B2	0	0	0	0	0	0	0	0	0	0
CONT. SPRAY PUMP FLOW (GPM)										
A	0	0	0	0	0	0	0	0	0	0
B	0	0	0	0	0	0	0	0	0	0
SIT LEVEL (%)										
A1	86.0	86.0	86.0	78.0	74.0	71.0	70.0	64.0	47.0	32.0
A2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
B1	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
B2	86.2	86.2	86.2	80.0	75.0	72.0	70.0	65.0	43.0	30.0
SIT PRESSURE (PSIG)										
A1	213.5	213.5	213.5	199.7	194.7	189.7	184.7	182.7	159.7	144.7
A2	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6
B1	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8
B2	214.1	214.1	214.1	199.7	194.7	189.7	184.7	182.7	159.7	144.7

=UNPOWERED/INVALID DATA POINT

	TIME	11:45	12:00	12:05	12:10	12:15	12:20	12:25	12:30	12:45	13:00
STEAM GENERATORS											
SG LEVEL WR (%)											
	A	0.0	10.0	20.0	30.0	35.0	38.0	40.0	45.0	61.0	63.0
	B	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
SG LEVEL NR (%)											
	A	0.0	0.0	0.0	0.0	0.0	0.0	0.0	12.0	68.0	70.0
	B	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
SG PRESSURE (PSIG)											
	A	13.5	100.0	150.0	185.0	180.0	175.0	170.0	168.0	145.0	130.0
	B	14.1	14.1	14.1	14.1	14.1	14.1	14.1	14.1	14.1	14.1
FEED FLOW (LB/HR)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
STEAM FLOW (LB/HR)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
SG BLOW DOWN FLOW (GPM)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
CONDENSER BACK PRESS (In. H		29.0	29.0	29.0	29.0	29.0	29.0	29.0	29.0	29.0	29.0
ELECTRICAL PLANT											
EDG LOAD (MW)											
	A	0	0	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0	0	0
4160V VITAL BUS (VOLTS)											
	1A3	4257.8	4256.2	4256.2	4256.2	4256.2	4256.2	4256.2	4256.2	4256.2	4256.2
	1B3	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
6.9KV SWGR (VOLTS)											
	1A1	7024.2	7023.6	7023.6	7023.6	7023.6	7023.6	7023.6	7023.6	7023.6	7023.6
	1B1	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4

=UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

		TIME	11:45	12:00	12:05	12:10	12:15	12:20	12:25	12:30	12:45	13:00
AUXILIARY SYSTEMS												
AFW FLOW (GPM)												
	A		0	150	150	300	300	300	300	150	150	150
	B		0	0	0	0	0	0	0	0	0	0
	C		0	0	0	0	0	0	0	0	0	0
TANK LEVEL												
RWT (FT)			33.3	33.3	33.3	33.1	32.9	32.7	32.4	32.0	30.5	28.6
CST (FT)			25.1	25.1	25.1	25.1	25.1	25.1	25.1	25.1	25.1	25.1
PWT (FT)			19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8
BAMT (%)												
	A		47.2	42.4	41.5	40.5	39.6	38.6	37.7	36.7	33.9	31.0
	B		92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4
COOLING WATER												
CCW HEADER (GPM)												
	A		7875	7883	7883	7883	7883	7883	7883	7883	7883	7883
	B		6995	6995	6995	6995	6995	6995	6995	6995	6995	6995
ICW HEADER (PSIG)												
	A		46	46	46	46	46	46	46	46	46	46
	B		48	48	48	48	48	48	48	48	48	48
CONDENSER HOTWELL LEVEL (FT)			0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

=UNPOWERED/INVALID DATA POINT

	TIME	13:15	13:30	13:45	14:00	14:15	14:30	14:45	15:00
NUCLEAR INSTRUMENTATION									
REACTOR POWER (%)									
LINEAR RANGE		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOG WIDE RANGE		3.46E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08	3.46E-08
REACTOR COOLANT SYSTEM (RTGB/QSPDS)									
T AVE (F)									
LOOP A		333.3	323.1	319.4	317.9	313.9	316.2	312.0	307.6
LOOPB		331.3	321.1	317.4	315.9	311.9	316.2	312.0	307.6
T COLD (F)									
LOOP A1		319.1	308.8	305.1	303.6	299.7	316.2	312.0	307.6
LOOP A2		338.1	327.8	324.1	322.6	318.7	316.2	312.0	307.6
LOOP B1		311.1	300.8	297.1	295.6	291.7	316.2	312.0	307.6
LOOP B2		338.1	327.8	324.1	322.6	318.7	316.2	312.0	307.6
T HOT (F)									
LOOP A		338.1	327.8	324.1	322.6	318.7	316.2	312.0	307.6
LOOP B		338.1	327.8	324.1	322.6	318.7	316.2	312.0	307.6
RCS PRESSURE (PSIA)		115.0	100.0	95.0	93.0	88.0	85.0	80.0	75.0
LOOP FLOW (dP)									
LOOP A1		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOOP A2		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOOP B1		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
LOOP B2		0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
PZR LEVEL (%)		100.0	100.0	100.0	100.0	100.0	100.0	100.0	100.0
PZR TEMP (F)		338.1	327.8	324.1	322.6	318.7	316.2	312.0	307.6
QUENCH TANK LEVEL (%)		63.1	63.1	63.1	63.1	63.1	63.1	63.1	63.1
QUENCH TANK TEMP (F)		90.6	90.6	90.6	90.6	90.6	90.6	90.6	90.6
QUENCH TANK PRESS (PSIG)		4.8	8.9	13.4	9.5	17.8	8.9	13.4	9.5

=UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

TIME	13:15	13:30	13:45	14:00	14:15	14:30	14:45	15:00
CHEMICAL & VOLUME CONTROL SYS								
LETDOWN FLOW (GPM)	94.8	94.8	94.8	94.8	94.8	94.8	94.8	94.8
VCT LEVEL (%)	40.6	40.6	40.6	40.6	40.6	40.6	40.6	40.6
CHARG. FLOW (GPM)	80.2	80.2	80.2	80.2	80.2	80.2	80.2	80.2
CHRG PUMP (ON/OFF)								
A	ON	ON	ON	ON	ON	ON	ON	ON
B	OFF	OFF	OFF	OFF	OFF	OFF	OFF	OFF
C	ON	ON	ON	ON	ON	ON	ON	ON
BORONOMETER (PPM)								
CORE COOLING (QSPDS)								
SUBCOOLED MONITOR (F)								
A	2.5	-0.8	0.0	-1.5	-1.7	2.0	2.2	-2.3
B	2.5	-0.8	0.0	-1.5	-1.7	2.0	2.2	-2.3
CORE EXIT THERMOCOUPLE (F)								
HIGHEST CET								
A	408.1	397.8	394.1	392.6	388.7	386.2	382.0	377.6
B	408.1	397.8	394.1	392.6	388.7	386.2	382.0	377.6
REPRESENTATIVE CET								
A	418.1	407.8	404.1	402.6	398.7	396.2	392.0	387.6
B	418.1	407.8	404.1	402.6	398.7	396.2	392.0	387.6
RX VESSEL LEVEL (%)								
A	80	80	80	80	80	80	80	80
B	80	80	80	80	80	80	80	80
CONTAINMENT								
PRESSURE (PSIG)	7.2	7.3	7.5	7.6	7.8	7.9	8.1	8.2
TEMP (F)	172.7	173.1	173.4	173.8	174.2	174.5	174.9	175.2
SUMP LEVEL (FT)								
WIDE RANGE	12.8	16.2	19.7	20.6	21.5	22.4	23.3	24.2
NARROW RANGE	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CONT. H2 (%)	1.8	1.8	1.8	1.8	1.8	1.8	1.8	1.8

=UNPOWERED/INVALID DATA POINT

St. Lucie Nuclear Plant
1992 Evaluated Exercise

TIME	13:15	13:30	13:45	14:00	14:15	14:30	14:45	15:00
ENGINEERED SAFETY FEATURES								
HPSI LOOP FLOW (GPM)								
A1	0	0	0	0	0	0	0	0
A2	0	0	0	0	0	0	0	0
B1	0	0	0	0	0	0	0	0
B2	0	0	0	0	0	0	0	0
LPSI LOOP FLOW (GPM)								
A1	1250	1400	1450	1500	1550	1550	1600	1650
A2	1250	1400	1450	1500	1550	1550	1600	1650
B1	0	0	0	0	0	0	0	0
B2	0	0	0	0	0	0	0	0
CONT. SPRAY PUMP FLOW (GPM)								
A	0	0	0	0	0	0	0	0
B	0	0	0	0	0	0	0	0
SIT LEVEL (%)								
A1	19.0	15.0	13.0	11.1	9.1	7.1	5.1	3.2
A2	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
B1	86.2	86.2	86.2	86.2	86.2	86.2	86.2	86.2
B2	20.0	18.0	16.0	14.0	11.6	8.6	5.6	2.6
SIT PRESSURE (PSIG)								
A1	129.7	85.3	80.3	78.3	73.3	70.3	65.3	60.3
A2	215.6	215.6	215.6	215.6	215.6	215.6	215.6	215.6
B1	213.8	213.8	213.8	213.8	213.8	213.8	213.8	213.8
B2	129.7	85.3	80.3	78.3	73.3	70.3	65.3	60.3

=UNPOWERED/INVALID DATA POINT

	TIME	13:15	13:30	13:45	14:00	14:15	14:30	14:45	15:00
STEAM GENERATORS									
SG LEVEL WR (%)									
	A	63.0	63.0	63.0	63.0	63.0	63.0	63.0	63.0
	B	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
SG LEVEL NR (%)									
	A	70.0	70.0	70.0	70.0	70.0	70.0	70.0	70.0
	B	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
SG PRESSURE (PSIG)									
	A	115.0	100.0	95.0	93.0	88.0	85.0	80.0	75.0
	B	14.1	14.1	14.1	14.1	14.1	14.1	14.1	14.1
FEED FLOW (LB/HR)									
	A	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0
STEAM FLOW (LB/HR)									
	A	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0
SG BLOW DOWN FLOW (GPM)									
	A	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0
CONDENSER BACK PRESS (in. H		29.0	29.0	29.0	29.0	29.0	29.0	29.0	29.0
ELECTRICAL PLANT									
EDG LOAD (MW)									
	A	0	0	0	0	0	0	0	0
	B	0	0	0	0	0	0	0	0
4160V VITAL BUS (VOLTS)									
	1A3	4256.2	4256.2	4256.2	4256.2	4256.2	4256.2	4256.2	4256.2
	1B3	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
6.9KV SWGR (VOLTS)									
	1A1	7023.6	7023.6	7023.6	7023.6	7023.6	7023.6	7023.6	7023.6
	1B1	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4	7035.4

[ ]=UNPOWERED/INVALID DATA POINT

		TIME	13:15	13:30	13:45	14:00	14:15	14:30	14:45	15:00
AUXILIARY SYSTEMS										
AFW FLOW (GPM)										
	A		150	150	150	150	150	150	150	150
	B		0	0	0	0	0	0	0	0
	C		0	0	0	0	0	0	0	0
TANK LEVEL										
RWT (FT)			26.3	23.6	20.7	17.7	14.7	11.5	8.4	5.1
CST (FT)			25.1	25.1	25.1	25.1	25.1	25.1	25.1	25.1
PWT (FT)			19.8	19.8	19.8	19.8	19.8	19.8	19.8	19.8
BAMT (%)										
	A		28.2	25.3	22.5	19.6	16.8	13.9	11.1	8.2
	B		92.4	92.4	92.4	92.4	92.4	92.4	92.4	92.4
COOLING WATER										
CCW HEADER (GPM)										
	A		7883	7883	7883	7883	7883	7883	7883	7883
	B		6995	6995	6995	6995	6995	6995	6995	6995
ICW HEADER (PSIG)										
	A		46	46	46	46	46	46	46	46
	B		48	48	48	48	48	48	48	48
CONDENSER HOTWELL LEVEL (FT)			0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

=UNPOWERED/INVALID DATA POINT

5.2 PLANT PARAMETER DATA CARDS

This Data to be Provided Later

5.3 ANNUNCIATOR MIMIC SHEETS

This Data to be Provided Later

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

6.0 METEOROLOGICAL PARAMETERS

This Scenario Section Contains the Following Information:

Section 6.1	METEOROLOGICAL FORECAST DATA - A simulated weather forecast for the time period affected by the postulated scenario events.
Section 6.2	METEOROLOGICAL DATA SUMMARY - The postulated meteorological conditions for the scenario in summary.
Section 6.3	METEOROLOGICAL DATA CARDS - Mimics of screens presenting meteorological data timed for presentation to the drill participants.

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

6.1 METEOROLOGICAL FORECAST DATA

U.S. WEATHER SERVICE FORECAST DATA

02/12/92

0700 EDT Forecast from the National Weather Service Station at Palm Beach International Airport for the Palm Beaches and vicinity.

A weak high pressure system over the Atlantic is pushing a slow-moving cold front westward towards the Florida Coast. Air ahead of this front is partly cloudy and warm as a result of a stationary high centered offshore east of southern Florida. Cloudy and warm conditions will prevail for the next 8-12 hours until the air mass is pushed into a cold front stretching north and south over central Florida. Contact with this stationary front will produce increased cloudiness and showers by evening.

Present conditions are fair and warm, temperature 78 degrees Fahrenheit with an anticipated high in the low eighties. Moderate scattered clouds with increasing cloudiness towards evening. Winds are mild from the east at 4 to 6 miles per hour continuing predominantly easterly throughout the evening. Probability of precipitation is 30% through Thursday morning.

Marine conditions are: moderate seas, 3-5 feet. Winds from the east at 3-5 knots, visibility 15 miles. Water temperature 68 degrees at Jupiter Inlet.

6.2 METEOROLOGICAL DATA SUMMARY

St. Lucie Nuclear Plant
1992 Exercise

PARAMETER	UNITS	TIME											
		07:30	07:45	08:00	08:15	08:30	08:45	09:00	09:15	09:30	09:45	10:00	10:15
10 METER													
15 Min Avg. Wind Speed	mph	3.7	4.4	4.3	4.4	5.1	4.9	4.5	3.9	5.2	4.4	3.8	5.2
	Knots	3.2	3.8	3.7	3.8	4.4	4.3	3.9	3.4	4.5	3.8	3.3	4.5
15 Min Avg. Wind Direction	Deg	88	89	90	94	95	96	96	95	93	94	96	97
Current Temp	Deg F	78	77.8	77.9	78.1	78.3	78	78.2	78.3	78.4	78.4	78.5	78.4
60 METER													
15 Min Avg. Wind Speed	MPH	4.3	5.2	4.7	5.4	5.2	4.8	5.6	4.3	4.4	5.6	5.3	5
15 Min Avg. Wind Direction	Deg	95	96	94	93	95	91	96	94	97	93	97	95
Current Temp	Deg F	79.2	78.7	79	79.1	79	78.9	78.7	78.8	79.3	79.2	79	79.3
15 Min Avg. Differential Temperature	Deg F	-1.2	-0.9	-1.1	-1	-0.7	-0.9	-0.5	-0.5	-0.9	-0.8	-0.5	-0.9
Stability Class		D	D	D	D	D	D	D	D	D	D	D	D

St. Lucie Nuclear Plant
1992 Exercise

PARAMETER	UNITS	TIME											
		10:30	10:45	11:00	11:15	11:30	11:45	12:00	12:15	12:30	12:45	13:00	13:15
10 METER													
15 Min Avg. Wind Speed	mph	5.2	5.1	4.4	3.9	3.8	4.1	4.3	4.4	4.3	4.1	5.0	4.8
	Knots	4.5	4.4	3.8	3.4	3.3	3.6	3.7	3.8	3.7	3.6	4.3	4.2
15 Min Avg. Wind Direction	Deg	95	96	94	97	98	95	96	97	95	93	96	94
Current Temp	Deg F	77.9	78.2	77.8	78	78.2	78.4	78.6	78.9	78.3	78.5	78.4	78.3
60 METER													
15 Min Avg. Wind Speed	MPH	5.2	4.9	5.4	4.8	5.1	5.4	5.2	5.4	4.9	5.1	5.4	5.3
15 Min Avg. Wind Direction	Deg	94	96	92	94	95	97	95	95	94	93	92	91
Current Temp	Deg F	78.9	79.4	78.8	79.2	79	79.1	79.3	79.8	79.1	79.4	79.2	79
15 Min Avg. Differential Temperature	Deg F	-1	-1.2	-1	-1.2	-0.8	-0.7	-0.7	-0.9	-0.8	-0.9	-0.8	-0.7
Stability Class		D	D	D	D	D	D	D	D	D	D	D	D

St. Lucie Nuclear Plant
1992 Exercise

PARAMETER	UNITS	TIME						
		13:30	13:45	14:00	14:15	14:30	14:45	15:00
10 METER								
15 Min Avg. Wind Speed	mph	4.9	4.7	5.1	5.3	5.4	4.7	5.0
	Knots	4.3	4.1	4.4	4.6	4.7	4.1	4.3
15 Min Avg. Wind Direction	Deg	95	97	98	95	92	92	92
Current Temp	Deg F	78.8	78.7	79	78.5	78.5	78.4	78.4
60 METER								
15 Min Avg. Wind Speed	MPH	4.8	4.7	5	5.3	5.3	5.2	5.2
15 Min Avg. Wind Direction	Deg	93	94	94	96	94	93	93
Current Temp	Deg F	79.6	79.8	79.9	79.6	79.3	79.2	79.2
15 Min Avg. Differential Temperature	Deg F	-0.8	-1.1	-0.9	-1.1	-0.8	-0.8	-0.8
Stability Class		D	D	D	D	D	D	D

6.3 METEOROLOGICAL DATA CARDS

This data to be provided later

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

7.0 RADIOLOGICAL DATA

This Scenario Section Contains the Following Information:

Section 7.1 REACTOR COOLANT SYSTEM RADIOLOGICAL DATA - simulated chemistry and radiochemistry data resulting from the postulated scenario events.
- 7.1.1 Coolant Radiochemistry Summary
- 7.1.2 Coolant Radiochemistry Data Cards
- 7.1.3 Coolant Chemistry Data
- 7.1.4 Containment Atmosphere Summary
- 7.1.5 Containment Atmosphere Sample Cards
- 7.1.6 Containment Sump Activity Summary
- 7.1.7 Containment Sump Sample Cards
- 7.1.8 Steam Generator "1A" Sample Activity Summary

Section 7.2 PROCESS RADIATION MONITOR DATA - Process radiation monitor data for the postulated scenario events.
- 7.2.1 Process Monitor Summary
- 7.2.2 Process Monitor Data Cards

Section 7.3 AREA RADIATION MONITOR DATA - Area radiation monitor data for the postulated scenario events.
- 7.3.1 Area Radiation Monitor Summary
- 7.3.2 Area Radiation Monitor Data Cards

Section 7.4 ONSITE RADIOLOGICAL DATA - Simulated radiation levels in and around onsite buildings as a result of postulated scenario events.
- 7.4.1 Inplant Area Maps
- 7.4.2 Onsite Plume Map

Section 7.5 OFF-SITE RADIOLOGICAL DATA - Simulated radiation levels off-site as a result of postulated scenario events.
- 7.5.1 Off-Site Monitoring Data
- 7.5.2 Off-Site Plume Map

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

7.1 REACTOR COOLANT SYSTEM RADIOLOGICAL DATA

This Scenario Section Contains the Following Information:

7.1.1 COOLANT RADIOCHEMISTRY SUMMARY

St. Lucie Nuclear Plant 1992 Exercise
Reactor Coolant System Activity Data Summary
uCi/ml

ISOTOPE	08:00	11:30	12:00	12:30	13:00	13:30	14:00	14:30	15:00
Kr-85m	4.71E-03	2.71E+04	1.25E+04	5.78E+03	2.89E+03	1.44E+03	7.22E+02	3.61E+02	1.81E+02
Kr-85	2.20E-03	1.62E+03	8.09E+02	4.04E+02	2.02E+02	1.01E+02	5.06E+01	2.53E+01	1.26E+01
Kr-87	8.90E-03	2.39E+04	9.07E+03	3.45E+03	1.73E+03	8.63E+02	4.31E+02	2.16E+02	1.08E+02
Kr-88	9.92E-03	6.17E+04	2.73E+04	1.20E+04	6.02E+03	3.01E+03	1.51E+03	7.53E+02	3.76E+02
Xe-131m	1.96E-02	5.75E+02	2.87E+02	1.43E+02	7.17E+01	3.59E+01	1.79E+01	8.97E+00	4.48E+00
Xe-133	3.86E-02	2.12E+05	1.06E+05	5.27E+04	2.64E+04	1.32E+04	6.59E+03	3.30E+03	1.65E+03
Xe-135	3.87E-02	1.57E+05	7.55E+04	3.63E+04	1.82E+04	9.08E+03	4.54E+03	2.27E+03	1.14E+03
I-131	4.32E-03	1.08E+05	5.38E+04	2.68E+04	1.34E+04	6.71E+03	3.36E+03	1.68E+03	8.39E+02
I-132	2.18E-02	6.75E+04	2.90E+04	1.25E+04	6.23E+03	3.12E+03	1.56E+03	7.79E+02	3.90E+02
I-133	1.94E-02	1.97E+05	9.70E+04	4.77E+04	2.39E+04	1.19E+04	5.96E+03	2.98E+03	1.49E+03
I-134	3.79E-02	4.62E+04	1.56E+04	5.25E+03	2.63E+03	1.31E+03	6.57E+02	3.28E+02	1.64E+02
I-135	2.82E-02	1.50E+05	7.12E+04	3.38E+04	1.69E+04	8.44E+03	4.22E+03	2.11E+03	1.06E+03
Cr-51	2.60E-04	4.47E-03	2.23E-03	1.12E-03	5.58E-04	2.79E-04	1.40E-04	6.98E-05	3.49E-05
Mn-54	9.03E-04	7.40E-04	3.70E-04	1.85E-04	9.26E-05	4.63E-05	2.31E-05	1.16E-05	5.78E-06
Fe-55	2.20E-04	3.79E-03	1.89E-03	9.47E-04	4.74E-04	2.37E-04	1.18E-04	5.92E-05	2.96E-05
Fe-59	1.40E-04	2.41E-03	1.20E-03	6.02E-04	3.01E-04	1.50E-04	7.52E-05	3.76E-05	1.88E-05
Co-58	1.82E-03	3.83E-02	1.92E-02	9.57E-03	4.79E-03	2.39E-03	1.20E-03	5.98E-04	2.99E-04
Co-60	2.80E-04	4.82E-03	2.41E-03	1.21E-03	6.03E-04	3.01E-04	1.51E-04	7.54E-05	3.77E-05
Rb-88	1.20E+00	5.75E+02	8.93E+01	1.39E+01	6.95E+00	3.47E+00	1.74E+00	8.68E-01	4.34E-01
Sr-89	4.80E-05	8.52E+04	4.26E+04	2.13E+04	1.07E+04	5.33E+03	2.66E+03	1.33E+03	6.66E+02
Sr-90	1.40E-06	5.95E+03	2.97E+03	1.49E+03	7.44E+02	3.72E+02	1.86E+02	9.29E+01	4.65E+01
Y-90	1.50E-07	6.11E+03	3.04E+03	1.51E+03	7.55E+02	3.78E+02	1.89E+02	9.44E+01	4.72E+01
Sr-91	7.00E-05	9.07E+04	4.38E+04	2.11E+04	1.06E+04	5.28E+03	2.64E+03	1.32E+03	6.60E+02
Y-91m	3.40E-05	1.15E+04	3.82E+03	1.26E+03	6.31E+02	3.15E+02	1.58E+02	7.89E+01	3.94E+01
Y-91	8.80E-06	1.11E+05	5.55E+04	2.77E+04	1.39E+04	6.93E+03	3.47E+03	1.73E+03	8.66E+02
Zr-95	8.20E-06	1.54E+05	7.72E+04	3.86E+04	1.93E+04	9.64E+03	4.82E+03	2.41E+03	1.21E+03
Nb-95	5.96E-04	1.55E+05	7.77E+04	3.88E+04	1.94E+04	9.71E+03	4.85E+03	2.43E+03	1.21E+03
Mo-99	1.10E-02	1.64E+05	8.18E+04	4.07E+04	2.03E+04	1.02E+04	5.08E+03	2.54E+03	1.27E+03
Tc-99m	5.00E-03	1.15E+05	5.44E+04	2.57E+04	1.28E+04	6.42E+03	3.21E+03	1.60E+03	8.02E+02
Ru-103	6.20E-06	1.51E+05	7.53E+04	3.76E+04	1.88E+04	9.41E+03	4.70E+03	2.35E+03	1.18E+03
Ru-106	1.40E-06	4.26E+04	2.13E+04	1.06E+04	5.32E+03	2.66E+03	1.33E+03	6.65E+02	3.33E+02
Te-129m	1.90E-04	7.89E+03	3.95E+03	1.97E+03	9.87E+02	4.93E+02	2.47E+02	1.23E+02	6.17E+01
Te-129	1.50E-05	8.33E+03	3.05E+03	1.12E+03	5.60E+02	2.80E+02	1.40E+02	7.00E+01	3.50E+01
Te-131m	3.00E-04	1.32E+04	6.54E+03	3.23E+03	1.62E+03	8.09E+02	4.04E+02	2.02E+02	1.01E+02
Te-131	1.00E-04	2.79E+03	6.03E+02	1.30E+02	6.52E+01	3.26E+01	1.63E+01	8.15E+00	4.08E+00
Te-132	3.50E-04	1.29E+05	6.42E+04	3.20E+04	1.60E+04	7.99E+03	4.00E+03	2.00E+03	9.99E+02
Cs-134	8.35E-04	1.74E+04	8.70E+03	4.35E+03	2.17E+03	1.09E+03	5.44E+02	2.72E+02	1.36E+02
Cs-136	1.90E-03	4.85E+03	2.42E+03	1.21E+03	6.04E+02	3.02E+02	1.51E+02	7.55E+01	3.78E+01
Cs-137	7.65E-04	7.98E+03	3.99E+03	1.99E+03	9.97E+02	4.99E+02	2.49E+02	1.25E+02	6.23E+01
Cs-138	5.87E-02	1.19E+04	3.11E+03	8.15E+02	4.08E+02	2.04E+02	1.02E+02	5.09E+01	2.55E+01
Ba-140	3.00E-05	1.60E+05	7.98E+04	3.99E+04	1.99E+04	9.97E+03	4.98E+03	2.49E+03	1.25E+03
La-140	1.90E-05	1.60E+05	7.92E+04	3.93E+04	1.96E+04	9.82E+03	4.91E+03	2.45E+03	1.23E+03
Ce-143	4.90E-06	1.33E+05	6.58E+04	3.26E+04	1.63E+04	8.14E+03	4.07E+03	2.03E+03	1.02E+03
Pr-143	6.80E-06	1.36E+05	6.78E+04	3.38E+04	1.69E+04	8.46E+03	4.23E+03	2.12E+03	1.06E+03
Ce-144	4.50E-06	1.09E+05	5.47E+04	2.74E+04	1.37E+04	6.84E+03	3.42E+03	1.71E+03	8.55E+02
Pr-144	3.10E-06	8.96E+02	1.34E+02	2.02E+01	1.01E+01	5.05E+00	2.52E+00	1.26E+00	6.31E-01
N/G TOT	1.23E-01	4.84E+05	2.31E+05	1.11E+05	5.54E+04	2.77E+04	1.39E+04	6.93E+03	3.47E+03
I2 TOT	1.12E-01	5.69E+05	2.67E+05	1.26E+05	6.30E+04	3.15E+04	1.58E+04	7.88E+03	3.94E+03
PART TOT	1.28E+00	2.00E+06	9.83E+05	4.86E+05	2.43E+05	1.22E+05	6.08E+04	3.04E+04	1.52E+04
TOTAL	1.51E+00	3.05E+06	1.48E+06	7.23E+05	3.62E+05	1.81E+05	9.04E+04	4.52E+04	2.26E+04
I-131 DEQ	1.34E-02	1.77E+05	8.74E+04	4.31E+04	2.16E+04	1.08E+04	5.39E+03	2.70E+03	1.35E+03

7.1.2 COOLANT RADIOCHEMISTRY DATA CARDS

This Data to be Provided Later

7.1.3 COOLANT CHEMISTRY DATA

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
REACTOR COOLANT CHEMISTRY DATA SUMMARY
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		07:30 TO 11:30	12:00	12:30	13:00	13:30	14:00	14:30	15:00
PARAMETER	UNITS								
pH		7.10	8.00	8.20	8.50	8.50	8.50	8.50	8.50
Dissolved O2	ppm	<0.1	1.80	1.30	1.00	1.00	1.00	1.00	1.00
Dissolved H2	cc/kg	25.00	300.00	320.00	260.00	150.00	150.00	150.00	150.00
Conductivity	umhos	10.00	600.00	800.00	900.00	1000.00	1200.00	1200.00	1200.00
Chlorides	ppm	<0.15	150.00	180.00	200.00	200.00	200.00	200.00	200.00
Boron	ppm	750.00	1500.00	2000.00	2000.00	2000.00	2000.00	2000.00	2000.00

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		07:30
PARAMETER	UNITS	
pH		7.10
Dissolved O2	ppm	<0.1
Dissolved H2	cc/kg	25.00
Conductivity	umhos	10.00
Chlorides	ppm	<0.15
Boron	ppm	540.00

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		11:30
PARAMETER	UNITS	
pH		7.10
Dissolved O2	ppm	<0.1
Dissolved H2	cc/kg	25.00
Conductivity	umhos	10.00
Chlorides	ppm	<0.15
Boron	ppm	750.00

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		12:00
PARAMETER	UNITS	
pH		8.00
Dissolved O2	ppm	1.80
Dissolved H2	cc/kg	300.00
Conductivity	umhos	600.00
Chlorides	ppm	150.00
Boron	ppm	1500.00

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		12:30
PARAMETER	UNITS	
pH		8.20
Dissolved O2	ppm	1.30
Dissolved H2	cc/kg	320.00
Conductivity	umhos	800.00
Chlorides	ppm	180.00
Boron	ppm	2000.00

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		13:00
PARAMETER	UNITS	
pH		8.50
Dissolved O2	ppm	. 1.00
Dissolved H2	cc/kg	260.00
Conductivity	umhos	900.00
Chlorides	ppm	200.00
Boron	ppm	2000.00

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		13:30
PARAMETER	UNITS	
pH		8.50
Dissolved O2	ppm	1.00
Dissolved H2	cc/kg	150.00
Conductivity	umhos	1000.00
Chlorides	ppm	200.00
Boron	ppm	2000.00

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		14:00
PARAMETER	UNITS	
pH		8.50
Dissolved O2	ppm	1.00
Dissolved H2	cc/kg	150.00
Conductivity	umhos	1200.00
Chlorides	ppm	200.00
Boron	ppm	2000.00

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		14:30
PARAMETER	UNITS	
pH		8.50
Dissolved O2	ppm	1.00
Dissolved H2	cc/kg	150.00
Conductivity	umhos	1200.00
Chlorides	ppm	200.00
Boron	ppm	2000.00

ST. LUCIE NUCLEAR PLANT 1992 EXERCISE
CHEMISTRY RESULTS ON REACTOR COOLANT SAMPLES

TIME OF SAMPLE:		15:00
PARAMETER	UNITS	
pH		8.50
Dissolved O2	ppm	1.00
Dissolved H2	cc/kg	150.00
Conductivity	umhos	1200.00
Chlorides	ppm	200.00
Boron	ppm	2000.00

St. Lucie Nuclear Plant 1992 Exercise
Containment Atmosphere Data Summary
uCi/cc

ISOTOPE	INITIAL	10:15	10:30	10:45	11:00	11:15	11:30	11:45	12:00	12:15
Kr-85m	3.50E-07	5.77E-07	4.13E-06	7.90E-06	1.17E-05	3.93E+00	7.85E+00	5.66E+01	7.08E+01	8.14E+01
Kr-85	4.00E-08	1.46E-07	1.80E-06	3.57E-06	5.33E-06	2.35E-01	4.70E-01	3.52E+00	4.40E+00	5.06E+00
Kr-87	1.90E-07	6.18E-07	7.33E-06	1.45E-05	2.16E-05	3.46E+00	6.93E+00	4.53E+01	5.66E+01	6.51E+01
Kr-88	6.30E-07	1.11E-06	8.58E-06	1.65E-05	2.45E-05	8.96E+00	1.79E+01	1.26E+02	1.58E+02	1.82E+02
Xe-131m	8.90E-08	1.03E-06	1.58E-05	3.15E-05	4.72E-05	8.35E-02	1.67E-01	1.25E+00	1.56E+00	1.80E+00
Xe-133	2.20E-05	2.39E-05	5.30E-05	8.39E-05	1.15E-04	3.08E+01	6.16E+01	4.61E+02	5.76E+02	6.63E+02
Xe-135	1.00E-06	2.86E-06	3.20E-05	6.31E-05	9.41E-05	2.28E+01	4.55E+01	3.35E+02	4.19E+02	4.81E+02
I-131	7.50E-06	7.71E-06	1.10E-05	1.44E-05	1.79E-05	1.56E+01	3.13E+01	2.34E+02	2.93E+02	3.37E+02
I-132	1.60E-06	2.65E-06	1.91E-05	3.66E-05	5.40E-05	9.80E+00	1.96E+01	1.36E+02	1.70E+02	1.96E+02
I-133	6.40E-06	7.33E-06	2.20E-05	3.75E-05	5.31E-05	2.86E+01	5.73E+01	4.26E+02	5.32E+02	6.12E+02
I-134	7.88E-07	2.61E-06	3.12E-05	6.16E-05	9.20E-05	6.71E+00	1.34E+01	8.26E+01	1.03E+02	1.19E+02
I-135	1.70E-06	3.06E-06	2.43E-05	4.69E-05	6.95E-05	2.18E+01	4.36E+01	3.18E+02	3.98E+02	4.58E+02
Cr-51	5.20E-09	1.77E-08	2.14E-07	4.22E-07	6.31E-07	9.64E-07	1.30E-06	9.73E-06	1.22E-05	1.40E-05
Mn-54	1.30E-09	4.47E-08	7.25E-07	1.45E-06	2.17E-06	1.19E-06	2.15E-07	1.61E-06	2.02E-06	2.32E-06
Fe-55	4.40E-09	1.50E-08	1.81E-07	3.57E-07	5.34E-07	8.17E-07	1.10E-06	8.25E-06	1.03E-05	1.19E-05
Fe-59	3.20E-09	9.94E-09	1.15E-07	2.28E-07	3.40E-07	5.20E-07	6.99E-07	5.24E-06	6.55E-06	7.54E-06
Co-58	4.50E-08	1.33E-07	1.50E-06	2.96E-06	4.42E-06	7.77E-06	1.11E-05	8.34E-05	1.04E-04	1.20E-04
Co-60	5.70E-09	1.92E-08	2.30E-07	4.55E-07	6.79E-07	1.04E-06	1.40E-06	1.05E-05	1.31E-05	1.51E-05
Rb-88	1.20E-08	5.76E-05	9.59E-04	1.92E-03	2.88E-03	8.48E-02	1.67E-01	6.98E-01	8.72E-01	1.00E+00
Sr-89	1.30E-09	3.61E-09	3.98E-08	7.83E-08	1.17E-07	1.24E+01	2.47E+01	1.86E+02	2.32E+02	2.67E+02
Sr-90	0.00E+00	6.74E-11	1.12E-09	2.25E-09	3.37E-09	8.63E-01	1.73E+00	1.30E+01	1.62E+01	1.86E+01
Y-90	0.00E+00	7.22E-12	1.20E-10	2.41E-10	3.61E-10	8.86E-01	1.77E+00	1.33E+01	1.66E+01	1.91E+01
Sr-91	7.00E-10	4.07E-09	5.68E-08	1.13E-07	1.69E-07	1.32E+01	2.63E+01	1.94E+02	2.43E+02	2.79E+02
Y-91m	4.60E-10	2.10E-09	2.77E-08	5.50E-08	8.22E-08	1.68E+00	3.35E+00	2.04E+01	2.55E+01	2.94E+01
Y-91	1.90E-10	6.13E-10	7.25E-09	1.43E-08	2.14E-08	1.61E+01	3.22E+01	2.42E+02	3.02E+02	3.47E+02
Zr-95	1.90E-10	5.85E-10	6.77E-09	1.33E-08	1.99E-08	2.24E+01	4.48E+01	3.36E+02	4.20E+02	4.83E+02
Nb-95	1.90E-11	2.87E-08	4.78E-07	9.56E-07	1.43E-06	2.26E+01	4.51E+01	3.38E+02	4.23E+02	4.86E+02
Mo-99	2.40E-07	7.69E-07	9.06E-06	1.79E-05	2.67E-05	2.39E+01	4.77E+01	3.57E+02	4.46E+02	5.13E+02
Tc-99m	3.80E-07	6.21E-07	4.39E-06	8.40E-06	1.24E-05	1.67E+01	3.34E+01	2.44E+02	3.04E+02	3.50E+02
Ru-103	1.30E-10	4.28E-10	5.10E-09	1.01E-08	1.50E-08	2.19E+01	4.37E+01	3.28E+02	4.10E+02	4.71E+02
Ru-106	3.20E-11	9.94E-11	1.15E-09	2.28E-09	3.40E-09	6.18E+00	1.24E+01	9.27E+01	1.16E+02	1.33E+02
Te-129m	3.90E-09	1.30E-08	1.56E-07	3.09E-07	4.61E-07	1.15E+00	2.29E+00	1.72E+01	2.15E+01	2.47E+01
Te-129	9.00E-09	9.72E-09	2.10E-08	3.31E-08	4.51E-08	1.21E+00	2.42E+00	1.55E+01	1.94E+01	2.23E+01
Te-131m	4.60E-09	1.90E-08	2.45E-07	4.86E-07	7.26E-07	1.92E+00	3.84E+00	2.87E+01	3.58E+01	4.12E+01
Te-131	7.70E-09	1.25E-08	8.79E-08	1.68E-07	2.48E-07	4.05E-01	8.10E-01	4.00E+00	4.99E+00	5.74E+00
Te-132	6.10E-08	7.78E-08	3.42E-07	6.22E-07	9.03E-07	1.87E+01	3.75E+01	2.80E+02	3.50E+02	4.03E+02
Cs-134	7.90E-08	1.19E-07	7.49E-07	1.42E-06	2.09E-06	2.52E+00	5.05E+00	3.79E+01	4.73E+01	5.44E+01
Cs-136	3.40E-08	1.25E-07	1.56E-06	3.08E-06	4.60E-06	7.03E-01	1.41E+00	1.05E+01	1.32E+01	1.52E+01
Cs-137	5.30E-08	8.98E-08	6.66E-07	1.28E-06	1.89E-06	1.16E+00	2.32E+00	1.74E+01	2.17E+01	2.50E+01
Cs-138	0.00E+00	2.82E-06	4.71E-05	9.41E-05	1.41E-04	1.72E+00	3.44E+00	1.87E+01	2.34E+01	2.69E+01
Ba-140	6.00E-10	2.04E-09	2.47E-08	4.87E-08	7.28E-08	2.32E+01	4.64E+01	3.48E+02	4.35E+02	5.00E+02
La-140	6.90E-10	1.60E-09	1.59E-08	3.12E-08	4.64E-08	2.32E+01	4.64E+01	3.46E+02	4.33E+02	4.98E+02
Ce-143	0.00E+00	2.36E-10	3.93E-09	7.86E-09	1.18E-08	1.93E+01	3.86E+01	2.88E+02	3.60E+02	4.14E+02
Pr-143	1.30E-10	4.57E-10	5.58E-09	1.10E-08	1.65E-08	1.97E+01	3.94E+01	2.95E+02	3.69E+02	4.24E+02
Ce-144	1.30E-10	3.47E-10	3.74E-09	7.35E-09	1.10E-08	1.59E+01	3.18E+01	2.38E+02	2.98E+02	3.43E+02
Pr-144	3.10E-10	4.59E-10	2.80E-09	5.28E-09	7.76E-09	1.30E-01	2.60E-01	1.07E+00	1.34E+00	1.54E+00
N/G TOT	2.43E-05	3.02E-05	1.23E-04	2.21E-04	3.19E-04	7.02E+01	1.40E+02	1.03E+03	1.29E+03	1.48E+03
I2 TOT	1.80E-05	2.34E-05	1.07E-04	1.97E-04	2.86E-04	8.26E+01	1.65E+02	1.20E+03	1.50E+03	1.72E+03
PART TOT	9.54E-07	6.25E-05	1.03E-03	2.05E-03	3.08E-03	2.90E+02	5.79E+02	4.31E+03	5.39E+03	6.20E+03
TOTAL	4.32E-05	1.16E-04	1.26E-03	2.47E-03	3.68E-03	4.42E+02	8.85E+02	6.54E+03	8.17E+03	9.40E+03
I-131 DEQ	9.45E-06	1.01E-05	2.02E-05	3.09E-05	4.16E-05	2.57E+01	5.13E+01	3.82E+02	4.78E+02	5.50E+02

St. Lucie Nuclear Plant 1992 Exercise
Containment Atmosphere Data Summary
uCi/cc

ISOTOPE	12:30	12:45	13:00	13:15	13:30	13:45	14:00	14:15	14:30	14:45
Kr-85m	8.14E+01	8.14E+01	8.14E+01	8.14E+01	8.14E+01	8.14E+01	8.14E+01	8.14E+01	8.14E+01	8.14E+01
Kr-85	5.06E+00	5.06E+00	5.06E+00	5.06E+00	5.06E+00	5.06E+00	5.06E+00	5.06E+00	5.06E+00	5.06E+00
Kr-87	6.51E+01	6.51E+01	6.51E+01	6.51E+01	6.51E+01	6.51E+01	6.51E+01	6.51E+01	6.51E+01	6.51E+01
Kr-88	1.82E+02	1.82E+02	1.82E+02	1.82E+02	1.82E+02	1.82E+02	1.82E+02	1.82E+02	1.81E+02	1.81E+02
Xe-131m	1.80E+00	1.80E+00	1.80E+00	1.80E+00	1.80E+00	1.80E+00	1.80E+00	1.80E+00	1.80E+00	1.80E+00
Xe-133	6.63E+02	6.63E+02	6.63E+02	6.63E+02	6.63E+02	6.63E+02	6.63E+02	6.63E+02	6.63E+02	6.63E+02
Xe-135	4.81E+02	4.81E+02	4.81E+02	4.81E+02	4.81E+02	4.81E+02	4.81E+02	4.81E+02	4.81E+02	4.81E+02
I-131	3.37E+02	3.37E+02	3.37E+02	3.37E+02	3.37E+02	3.37E+02	3.37E+02	3.37E+02	3.37E+02	3.37E+02
I-132	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02	1.96E+02
I-133	6.12E+02	6.12E+02	6.12E+02	6.12E+02	6.12E+02	6.12E+02	6.12E+02	6.12E+02	6.12E+02	6.12E+02
I-134	1.19E+02	1.19E+02	1.19E+02	1.19E+02	1.19E+02	1.19E+02	1.19E+02	1.19E+02	1.19E+02	1.19E+02
I-135	4.58E+02	4.58E+02	4.58E+02	4.58E+02	4.58E+02	4.58E+02	4.58E+02	4.58E+02	4.58E+02	4.58E+02
Cr-51	1.40E-05	1.40E-05	1.40E-05	1.40E-05	1.40E-05	1.40E-05	1.40E-05	1.40E-05	1.40E-05	1.40E-05
Mn-54	2.32E-06	2.32E-06	2.32E-06	2.32E-06	2.32E-06	2.32E-06	2.32E-06	2.32E-06	2.32E-06	2.32E-06
Fe-55	1.19E-05	1.19E-05	1.19E-05	1.19E-05	1.19E-05	1.19E-05	1.19E-05	1.19E-05	1.19E-05	1.19E-05
Fe-59	7.54E-06	7.54E-06	7.54E-06	7.54E-06	7.54E-06	7.54E-06	7.54E-06	7.54E-06	7.54E-06	7.54E-06
Co-58	1.20E-04	1.20E-04	1.20E-04	1.20E-04	1.20E-04	1.20E-04	1.20E-04	1.20E-04	1.20E-04	1.20E-04
Co-60	1.51E-05	1.51E-05	1.51E-05	1.51E-05	1.51E-05	1.51E-05	1.51E-05	1.51E-05	1.51E-05	1.51E-05
Rb-88	1.00E+00	1.00E+00	1.00E+00	1.00E+00	1.00E+00	1.00E+00	1.00E+00	1.00E+00	1.00E+00	9.99E-01
Sr-89	2.67E+02	2.67E+02	2.67E+02	2.67E+02	2.67E+02	2.67E+02	2.67E+02	2.67E+02	2.67E+02	2.67E+02
Sr-90	1.86E+01	1.86E+01	1.86E+01	1.86E+01	1.86E+01	1.86E+01	1.86E+01	1.86E+01	1.86E+01	1.86E+01
Y-90	1.91E+01	1.91E+01	1.91E+01	1.91E+01	1.91E+01	1.91E+01	1.91E+01	1.91E+01	1.91E+01	1.91E+01
Sr-91	2.79E+02	2.79E+02	2.79E+02	2.79E+02	2.79E+02	2.79E+02	2.79E+02	2.79E+02	2.79E+02	2.79E+02
Y-91m	2.94E+01	2.94E+01	2.94E+01	2.94E+01	2.94E+01	2.94E+01	2.93E+01	2.93E+01	2.93E+01	2.93E+01
Y-91	3.47E+02	3.47E+02	3.47E+02	3.47E+02	3.47E+02	3.47E+02	3.47E+02	3.47E+02	3.47E+02	3.47E+02
Zr-95	4.83E+02	4.83E+02	4.83E+02	4.83E+02	4.83E+02	4.83E+02	4.83E+02	4.83E+02	4.83E+02	4.83E+02
Nb-95	4.86E+02	4.86E+02	4.86E+02	4.86E+02	4.86E+02	4.86E+02	4.86E+02	4.86E+02	4.86E+02	4.86E+02
Mo-99	5.13E+02	5.13E+02	5.13E+02	5.13E+02	5.13E+02	5.13E+02	5.13E+02	5.13E+02	5.13E+02	5.13E+02
Tc-99m	3.50E+02	3.50E+02	3.50E+02	3.50E+02	3.50E+02	3.50E+02	3.50E+02	3.50E+02	3.50E+02	3.50E+02
Ru-103	4.71E+02	4.71E+02	4.71E+02	4.71E+02	4.71E+02	4.71E+02	4.71E+02	4.71E+02	4.71E+02	4.71E+02
Ru-106	1.33E+02	1.33E+02	1.33E+02	1.33E+02	1.33E+02	1.33E+02	1.33E+02	1.33E+02	1.33E+02	1.33E+02
Te-129m	2.47E+01	2.47E+01	2.47E+01	2.47E+01	2.47E+01	2.47E+01	2.47E+01	2.47E+01	2.47E+01	2.47E+01
Te-129	2.23E+01	2.23E+01	2.23E+01	2.23E+01	2.23E+01	2.23E+01	2.23E+01	2.23E+01	2.23E+01	2.23E+01
Te-131m	4.12E+01	4.12E+01	4.12E+01	4.12E+01	4.12E+01	4.12E+01	4.12E+01	4.12E+01	4.12E+01	4.12E+01
Te-131	5.74E+00	5.74E+00	5.74E+00	5.74E+00	5.74E+00	5.73E+00	5.73E+00	5.73E+00	5.73E+00	5.73E+00
Te-132	4.03E+02	4.03E+02	4.03E+02	4.03E+02	4.03E+02	4.03E+02	4.03E+02	4.03E+02	4.03E+02	4.03E+02
Cs-134	5.44E+01	5.44E+01	5.44E+01	5.44E+01	5.44E+01	5.44E+01	5.44E+01	5.44E+01	5.44E+01	5.44E+01
Cs-136	1.52E+01	1.52E+01	1.52E+01	1.52E+01	1.52E+01	1.52E+01	1.52E+01	1.52E+01	1.52E+01	1.52E+01
Cs-137	2.50E+01	2.50E+01	2.50E+01	2.50E+01	2.50E+01	2.50E+01	2.50E+01	2.50E+01	2.50E+01	2.50E+01
Cs-138	2.69E+01	2.69E+01	2.69E+01	2.69E+01	2.69E+01	2.68E+01	2.68E+01	2.68E+01	2.68E+01	2.68E+01
Ba-140	5.00E+02	5.00E+02	5.00E+02	5.00E+02	5.00E+02	5.00E+02	5.00E+02	5.00E+02	5.00E+02	5.00E+02
La-140	4.98E+02	4.98E+02	4.98E+02	4.98E+02	4.98E+02	4.98E+02	4.98E+02	4.98E+02	4.98E+02	4.98E+02
Ce-143	4.14E+02	4.14E+02	4.14E+02	4.14E+02	4.14E+02	4.14E+02	4.14E+02	4.14E+02	4.14E+02	4.14E+02
Pr-143	4.24E+02	4.24E+02	4.24E+02	4.24E+02	4.24E+02	4.24E+02	4.24E+02	4.24E+02	4.24E+02	4.24E+02
Ce-144	3.43E+02	3.43E+02	3.43E+02	3.43E+02	3.43E+02	3.43E+02	3.43E+02	3.43E+02	3.43E+02	3.43E+02
Pr-144	1.54E+00	1.54E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00	1.53E+00
N/G TOT.	1.48E+03	1.48E+03	1.48E+03	1.48E+03	1.48E+03	1.48E+03	1.48E+03	1.48E+03	1.48E+03	1.48E+03
I2 TOT.	1.72E+03	1.72E+03	1.72E+03	1.72E+03	1.72E+03	1.72E+03	1.72E+03	1.72E+03	1.72E+03	1.72E+03
PART. TOT.	6.20E+03	6.20E+03	6.20E+03	6.20E+03	6.20E+03	6.20E+03	6.20E+03	6.20E+03	6.20E+03	6.20E+03
TOTAL	9.40E+03	9.40E+03	9.40E+03	9.40E+03	9.40E+03	9.40E+03	9.40E+03	9.40E+03	9.40E+03	9.40E+03
I-131 DEQ	5.50E+02	5.50E+02	5.50E+02	5.50E+02	5.50E+02	5.50E+02	5.50E+02	5.50E+02	5.50E+02	5.50E+02

St. Lucie Nuclear Plant 1992 Exercise
Containment Atmosphere Data Summary
uCi/cc

ISOTOPE	15:00
Kr-85m	8.14E+01
Kr-85	5.06E+00
Kr-87	6.51E+01
Kr-88	1.81E+02
Xe-131m	1.80E+00
Xe-133	6.63E+02
Xe-135	4.81E+02
I-131	3.37E+02
I-132	1.96E+02
I-133	6.12E+02
I-134	1.19E+02
I-135	4.58E+02
Cr-51	1.40E-05
Mn-54	2.32E-06
Fe-55	1.19E-05
Fe-59	7.54E-06
Co-58	1.20E-04
Co-60	1.51E-05
Rb-88	9.99E-01
Sr-89	2.67E+02
Sr-90	1.86E+01
Y-90	1.91E+01
Sr-91	2.79E+02
Y-91m	2.93E+01
Y-91	3.47E+02
Zr-95	4.83E+02
Nb-95	4.86E+02
Mo-99	5.13E+02
Tc-99m	3.50E+02
Ru-103	4.71E+02
Ru-106	1.33E+02
Te-129m	2.47E+01
Te-129	2.23E+01
Te-131m	4.12E+01
Te-131	5.73E+00
Te-132	4.03E+02
Cs-134	5.44E+01
Cs-136	1.52E+01
Cs-137	2.50E+01
Cs-138	2.68E+01
Ba-140	5.00E+02
La-140	4.98E+02
Ce-143	4.14E+02
Pr-143	4.24E+02
Ce-144	3.43E+02
Pr-144	1.53E+00
N/G TOT.	1.48E+03
I2 TOT.	1.72E+03
PART. TOT.	6.20E+03
TOTAL	9.40E+03
I-131 DEQ	5.50E+02

7.1.5 CONTAINMENT ATMOSPHERE SAMPLE CARDS

This Data to be Provided Later

7.1.6 CONTAINMENT SUMP ACTIVITY SUMMARY

St. Lucie Nuclear Plant 1992 Evaluated Exercise
Containment Sump Data Summary
uCi/ml

ISOTOPE	INITIAL	10:15	10:30	11:00	11:30	12:00	12:30	13:00	13:30	14:00
I-131	7.50E-05	3.15E-04	6.54E-04	7.66E-04	7.13E-04	5.38E+02	2.68E+01	2.68E+01	2.67E+00	2.67E+00
I-132	1.60E-05	1.25E-03	3.04E-03	3.71E-03	3.52E-03	2.90E+02	1.25E+01	1.07E+01	9.21E-01	7.92E-01
I-133	6.40E-04	1.70E-03	3.15E-03	3.57E-03	3.27E-03	9.70E+02	4.77E+01	4.69E+01	4.61E+00	4.54E+00
I-134	7.88E-06	2.15E-03	5.28E-03	6.43E-03	6.11E-03	1.56E+02	5.25E+00	3.54E+00	2.39E-01	1.61E-01
I-135	1.70E-05	1.61E-03	3.93E-03	4.79E-03	4.55E-03	7.12E+02	3.38E+01	3.20E+01	3.04E+00	2.89E+00
Cr-51	5.20E-07	1.52E-05	3.65E-05	4.43E-05	4.20E-05	2.23E-05	1.12E-06	1.12E-06	1.11E-07	1.11E-07
Mn-54	1.70E-07	5.13E-05	1.26E-04	1.53E-04	1.46E-04	3.70E-06	1.85E-07	1.85E-07	1.85E-08	1.85E-08
Fe-55	4.40E-07	1.29E-05	3.09E-05	3.75E-05	3.56E-05	1.89E-05	9.47E-07	9.47E-07	9.47E-08	9.47E-08
Fe-59	3.20E-07	8.23E-06	1.97E-05	2.39E-05	2.26E-05	1.20E-05	6.02E-07	6.02E-07	6.01E-08	6.01E-08
Co-58	4.50E-06	1.07E-04	2.56E-04	3.11E-04	2.94E-04	1.92E-04	9.57E-06	9.57E-06	9.57E-07	9.57E-07
Co-60	5.70E-07	1.64E-05	3.93E-05	4.78E-05	4.53E-05	2.41E-05	1.21E-06	1.21E-06	1.21E-07	1.21E-07
Rb-88	1.20E-06	6.77E-02	1.66E-01	2.03E-01	1.93E-01	8.93E-01	1.39E-02	4.32E-03	1.34E-04	4.18E-05
Sr-89	1.30E-07	2.84E-06	6.77E-06	8.20E-06	7.77E-06	4.26E+02	2.13E+01	2.13E+01	2.13E+00	2.13E+00
Sr-90	0.00E+00	7.92E-08	1.95E-07	2.38E-07	2.26E-07	2.97E+01	1.49E+00	1.49E+00	1.49E-01	1.49E-01
Y-90	0.00E+00	8.49E-09	2.09E-08	2.54E-08	2.42E-08	3.04E+01	1.51E+00	1.50E+00	1.49E-01	1.49E-01
Sr-91	7.00E-08	4.03E-06	9.79E-06	1.19E-05	1.13E-05	4.38E+02	2.11E+01	2.04E+01	1.97E+00	1.90E+00
Y-91m	4.60E-08	1.97E-06	4.76E-06	5.79E-06	5.49E-06	3.82E+01	1.26E+00	8.35E-01	5.52E-02	3.65E-02
Y-91	1.90E-08	5.16E-07	1.24E-06	1.50E-06	1.42E-06	5.55E+02	2.77E+01	2.77E+01	2.77E+00	2.77E+00
Zr-95	1.90E-08	4.82E-07	1.15E-06	1.40E-06	1.33E-06	7.72E+02	3.86E+01	3.86E+01	3.86E+00	3.85E+00
Nb-95	1.90E-08	3.38E-05	8.29E-05	1.01E-04	9.61E-05	7.77E+02	3.88E+01	3.88E+01	3.88E+00	3.88E+00
Mo-99	2.40E-05	6.45E-04	1.55E-03	1.88E-03	1.78E-03	8.18E+02	4.07E+01	4.05E+01	4.03E+00	4.00E+00
Tc-99m	3.80E-05	3.19E-04	7.22E-04	8.65E-04	8.15E-04	5.44E+02	2.57E+01	2.42E+01	2.29E+00	2.16E+00
Ru-103	1.30E-08	3.63E-07	8.72E-07	1.06E-06	1.00E-06	7.53E+02	3.76E+01	3.76E+01	3.76E+00	3.76E+00
Ru-106	3.20E-09	8.23E-08	1.97E-07	2.39E-07	2.26E-07	2.13E+02	1.06E+01	1.06E+01	1.06E+00	1.06E+00
Te-129m	3.90E-07	1.11E-05	2.67E-05	3.24E-05	3.07E-05	3.95E+01	1.97E+00	1.97E+00	1.97E-01	1.97E-01
Te-129	9.00E-07	1.70E-06	2.73E-06	2.94E-06	2.62E-06	3.05E+01	1.12E+00	8.21E-01	6.02E-02	4.41E-02
Te-131m	4.60E-07	1.74E-05	4.21E-05	5.11E-05	4.85E-05	6.54E+01	3.23E+00	3.20E+00	3.16E-01	3.12E-01
Te-131	7.70E-07	6.39E-06	1.45E-05	1.73E-05	1.63E-05	6.03E+00	1.30E-01	5.64E-02	2.44E-03	1.05E-03
Te-132	6.10E-06	2.56E-05	5.30E-05	6.21E-05	5.78E-05	6.42E+02	3.20E+01	3.18E+01	3.17E+00	3.16E+00
Cs-134	7.90E-06	5.47E-05	1.22E-04	1.45E-04	1.36E-04	8.70E+01	4.35E+00	4.35E+00	4.35E-01	4.35E-01
Cs-136	3.40E-06	1.11E-04	2.67E-04	3.24E-04	3.07E-04	2.42E+01	1.21E+00	1.21E+00	1.21E-01	1.20E-01
Cs-137	5.30E-06	4.83E-05	1.10E-04	1.32E-04	1.25E-04	3.99E+01	1.99E+00	1.99E+00	1.99E-01	1.99E-01
Cs-138	0.00E+00	3.32E-03	8.16E-03	9.95E-03	9.46E-03	3.11E+01	8.15E-01	4.27E-01	2.24E-02	1.17E-02
Ba-140	6.00E-08	1.75E-06	4.22E-06	5.12E-06	4.85E-06	7.98E+02	3.99E+01	3.98E+01	3.98E+00	3.97E+00
La-140	6.90E-08	1.14E-06	2.69E-06	3.25E-06	3.08E-06	7.92E+02	3.93E+01	3.89E+01	3.86E+00	3.83E+00
Ce-143	0.00E+00	2.77E-07	6.81E-07	8.31E-07	7.90E-07	6.58E+02	3.26E+01	3.22E+01	3.19E+00	3.15E+00
Pr-143	1.30E-08	3.97E-07	9.55E-07	1.16E-06	1.10E-06	6.78E+02	3.38E+01	3.38E+01	3.38E+00	3.37E+00
Ce-144	1.30E-08	2.67E-07	6.35E-07	7.69E-07	7.28E-07	5.47E+02	2.74E+01	2.74E+01	2.74E+00	2.74E+00
Pr-144	3.10E-08	2.05E-07	4.53E-07	5.39E-07	5.06E-07	1.34E+00	2.02E-02	6.06E-03	1.82E-04	5.46E-05
I2 TOT	7.56E-04	7.03E-03	1.61E-02	1.93E-02	1.82E-02	2.31E+03	1.11E+02	1.07E+02	1.03E+01	9.97E+00
PART TOT	9.54E-05	7.25E-02	1.78E-01	2.17E-01	2.06E-01	2.67E+03	1.26E+02	1.20E+02	1.15E+01	1.10E+01
TOTAL	8.51E-04	7.96E-02	1.94E-01	2.36E-01	2.24E-01	9.83E+03	4.86E+02	4.82E+02	4.78E+01	4.74E+01
I-131 DEQ	2.51E-04	9.93E-04	2.04E-03	2.38E-03	2.21E-03	1.48E+04	7.23E+02	7.08E+02	6.95E+01	6.84E+01

St. Lucie Nuclear Plant 1992 Evaluated Exercise
Containment Sump Data Summary
uCi/ml

ISOTOPE	14:30	15:00
I-131	2.67E+00	2.66E+00
I-132	6.81E-01	5.85E-01
I-133	4.46E+00	4.39E+00
I-134	1.09E-01	7.34E-02
I-135	2.74E+00	2.60E+00
Cr-51	1.11E-07	1.11E-07
Mn-54	1.85E-08	1.85E-08
Fe-55	9.47E-08	9.47E-08
Fe-59	6.01E-08	6.01E-08
Co-58	9.57E-07	9.56E-07
Co-60	1.21E-07	1.21E-07
Rb-88	1.30E-05	4.04E-06
Sr-89	2.13E+00	2.13E+00
Sr-90	1.49E-01	1.49E-01
Y-90	1.48E-01	1.47E-01
Sr-91	1.83E+00	1.77E+00
Y-91m	2.41E-02	1.60E-02
Y-91	2.77E+00	2.77E+00
Zr-95	3.85E+00	3.85E+00
Nb-95	3.88E+00	3.87E+00
Mo-99	3.98E+00	3.96E+00
Tc-99m	2.04E+00	1.93E+00
Ru-103	3.76E+00	3.76E+00
Ru-106	1.06E+00	1.06E+00
Te-129m	1.97E-01	1.97E-01
Te-129	3.24E-02	2.37E-02
Te-131m	3.09E-01	3.05E-01
Te-131	4.56E-04	1.97E-04
Te-132	3.14E+00	3.13E+00
Cs-134	4.35E-01	4.35E-01
Cs-136	1.20E-01	1.20E-01
Cs-137	1.99E-01	1.99E-01
Cs-138	6.16E-03	3.23E-03
Ba-140	3.97E+00	3.96E+00
La-140	3.79E+00	3.76E+00
Ce-143	3.12E+00	3.09E+00
Pr-143	3.37E+00	3.37E+00
Ce-144	2.74E+00	2.74E+00
Pr-144	1.64E-05	4.93E-06
I2 TOT.	9.66E+00	9.39E+00
PART. TOT.	1.07E+01	1.03E+01
TOTAL	4.71E+01	4.67E+01
I-131 DEQ	6.74E+01	6.64E+01

7.1.7 CONTAINMENT SUMP SAMPLE CARDS

This Data to be Provided Later

7.1.8 STEAM GENERATOR "1A" SAMPLE ACTIVITY SUMMARY

St. Lucie Nuclear Plant 1992 Exercise
Steam Generator "A" Activity Data Summary
uCi/ml

ISOTOPE	08:00	12:00	12:15	12:30	12:45	13:00	13:30	14:00	14:30	15:00
Kr-85m	<MDA	1.25E+02	1.25E+00	1.25E-02	1.25E-04	1.25E-06	1.25E-07	1.25E-08	1.25E-09	1.25E-10
Kr-85	<MDA	8.09E+00	8.09E-02	8.09E-04	8.09E-06	8.09E-08	8.09E-09	8.09E-10	8.09E-11	8.09E-12
Kr-87	<MDA	9.07E+01	9.07E-01	9.07E-03	9.07E-05	9.07E-07	9.07E-08	9.07E-09	9.07E-10	9.07E-11
Kr-88	<MDA	2.73E+02	2.73E+00	2.73E-02	2.73E-04	2.73E-06	2.73E-07	2.73E-08	2.73E-09	2.73E-10
Xe-131m	<MDA	2.87E+00	2.87E-02	2.87E-04	2.87E-06	2.87E-08	2.87E-09	2.87E-10	2.87E-11	2.87E-12
Xe-133	<MDA	1.06E+03	1.06E+01	1.06E-01	1.06E-03	1.06E-05	1.06E-06	1.06E-07	1.06E-08	1.06E-09
Xe-135	<MDA	7.55E+02	7.55E+00	7.55E-02	7.55E-04	7.55E-06	7.55E-07	7.55E-08	7.55E-09	7.55E-10
I-131	<MDA	5.38E+02	5.38E+00	5.38E-02	5.38E-04	5.38E-06	5.38E-07	5.38E-08	5.38E-09	5.38E-10
I-132	<MDA	2.90E+02	2.90E+00	2.90E-02	2.90E-04	2.90E-06	2.90E-07	2.90E-08	2.90E-09	2.90E-10
I-133	<MDA	9.70E+02	9.70E+00	9.70E-02	9.70E-04	9.70E-06	9.70E-07	9.70E-08	9.70E-09	9.70E-10
I-134	<MDA	1.56E+02	1.56E+00	1.56E-02	1.56E-04	1.56E-06	1.56E-07	1.56E-08	1.56E-09	1.56E-10
I-135	<MDA	7.12E+02	7.12E+00	7.12E-02	7.12E-04	7.12E-06	7.12E-07	7.12E-08	7.12E-09	7.12E-10
Cr-51	<MDA	2.23E-05	2.23E-07	2.23E-09	2.23E-11	2.23E-13	2.23E-14	2.23E-15	2.23E-16	2.23E-17
Mn-54	<MDA	3.70E-06	3.70E-08	3.70E-10	3.70E-12	3.70E-14	3.70E-15	3.70E-16	3.70E-17	3.70E-18
Fe-55	<MDA	1.89E-05	1.89E-07	1.89E-09	1.89E-11	1.89E-13	1.89E-14	1.89E-15	1.89E-16	1.89E-17
Fe-59	<MDA	1.20E-05	1.20E-07	1.20E-09	1.20E-11	1.20E-13	1.20E-14	1.20E-15	1.20E-16	1.20E-17
Co-58	<MDA	1.92E-04	1.92E-06	1.92E-08	1.92E-10	1.92E-12	1.92E-13	1.92E-14	1.92E-15	1.92E-16
Co-60	<MDA	2.41E-05	2.41E-07	2.41E-09	2.41E-11	2.41E-13	2.41E-14	2.41E-15	2.41E-16	2.41E-17
Rb-88	<MDA	8.93E-01	8.93E-03	8.93E-05	8.93E-07	8.93E-09	8.93E-10	8.93E-11	8.93E-12	8.93E-13
Sr-89	<MDA	4.26E+02	4.26E+00	4.26E-02	4.26E-04	4.26E-06	4.26E-07	4.26E-08	4.26E-09	4.26E-10
Sr-90	<MDA	2.97E+01	2.97E-01	2.97E-03	2.97E-05	2.97E-07	2.97E-08	2.97E-09	2.97E-10	2.97E-11
Y-90	<MDA	3.04E+01	3.04E-01	3.04E-03	3.04E-05	3.04E-07	3.04E-08	3.04E-09	3.04E-10	3.04E-11
Sr-91	<MDA	4.38E+02	4.38E+00	4.38E-02	4.38E-04	4.38E-06	4.38E-07	4.38E-08	4.38E-09	4.38E-10
Y-91m	<MDA	3.82E+01	3.82E-01	3.82E-03	3.82E-05	3.82E-07	3.82E-08	3.82E-09	3.82E-10	3.82E-11
Y-91	<MDA	5.55E+02	5.55E+00	5.55E-02	5.55E-04	5.55E-06	5.55E-07	5.55E-08	5.55E-09	5.55E-10
Zr-95	<MDA	7.72E+02	7.72E+00	7.72E-02	7.72E-04	7.72E-06	7.72E-07	7.72E-08	7.72E-09	7.72E-10
Nb-95	<MDA	7.77E+02	7.77E+00	7.77E-02	7.77E-04	7.77E-06	7.77E-07	7.77E-08	7.77E-09	7.77E-10
Mo-99	<MDA	8.18E+02	8.18E+00	8.18E-02	8.18E-04	8.18E-06	8.18E-07	8.18E-08	8.18E-09	8.18E-10
Tc-99m	<MDA	5.44E+02	5.44E+00	5.44E-02	5.44E-04	5.44E-06	5.44E-07	5.44E-08	5.44E-09	5.44E-10
Ru-103	<MDA	7.53E+02	7.53E+00	7.53E-02	7.53E-04	7.53E-06	7.53E-07	7.53E-08	7.53E-09	7.53E-10
Ru-106	<MDA	2.13E+02	2.13E+00	2.13E-02	2.13E-04	2.13E-06	2.13E-07	2.13E-08	2.13E-09	2.13E-10
Te-129m	<MDA	3.95E+01	3.95E-01	3.95E-03	3.95E-05	3.95E-07	3.95E-08	3.95E-09	3.95E-10	3.95E-11
Te-129	<MDA	3.05E+01	3.05E-01	3.05E-03	3.05E-05	3.05E-07	3.05E-08	3.05E-09	3.05E-10	3.05E-11
Te-131m	<MDA	6.54E+01	6.54E-01	6.54E-03	6.54E-05	6.54E-07	6.54E-08	6.54E-09	6.54E-10	6.54E-11
Te-131	<MDA	6.03E+00	6.03E-02	6.03E-04	6.03E-06	6.03E-08	6.03E-09	6.03E-10	6.03E-11	6.03E-12
Te-132	<MDA	6.42E+02	6.42E+00	6.42E-02	6.42E-04	6.42E-06	6.42E-07	6.42E-08	6.42E-09	6.42E-10
Cs-134	<MDA	8.70E+01	8.70E-01	8.70E-03	8.70E-05	8.70E-07	8.70E-08	8.70E-09	8.70E-10	8.70E-11
Cs-136	<MDA	2.42E+01	2.42E-01	2.42E-03	2.42E-05	2.42E-07	2.42E-08	2.42E-09	2.42E-10	2.42E-11
Cs-137	<MDA	3.99E+01	3.99E-01	3.99E-03	3.99E-05	3.99E-07	3.99E-08	3.99E-09	3.99E-10	3.99E-11
Cs-138	<MDA	3.11E+01	3.11E-01	3.11E-03	3.11E-05	3.11E-07	3.11E-08	3.11E-09	3.11E-10	3.11E-11
Ba-140	<MDA	7.98E+02	7.98E+00	7.98E-02	7.98E-04	7.98E-06	7.98E-07	7.98E-08	7.98E-09	7.98E-10
La-140	<MDA	7.92E+02	7.92E+00	7.92E-02	7.92E-04	7.92E-06	7.92E-07	7.92E-08	7.92E-09	7.92E-10
Ce-143	<MDA	6.58E+02	6.58E+00	6.58E-02	6.58E-04	6.58E-06	6.58E-07	6.58E-08	6.58E-09	6.58E-10
Pr-143	<MDA	6.78E+02	6.78E+00	6.78E-02	6.78E-04	6.78E-06	6.78E-07	6.78E-08	6.78E-09	6.78E-10
Ce-144	<MDA	5.47E+02	5.47E+00	5.47E-02	5.47E-04	5.47E-06	5.47E-07	5.47E-08	5.47E-09	5.47E-10
Pr-144	<MDA	1.34E+00	1.34E-02	1.34E-04	1.34E-06	1.34E-08	1.34E-09	1.34E-10	1.34E-11	1.34E-12
N/G TOT	<MDA	2.31E+03	2.31E+01	2.31E-01	2.31E-03	2.31E-05	2.31E-06	2.31E-07	2.31E-08	2.31E-09
I2 TOT	<MDA	2.67E+03	2.67E+01	2.67E-01	2.67E-03	2.67E-05	2.67E-06	2.67E-07	2.67E-08	2.67E-09
PART TOT	<MDA	9.83E+03	9.83E+01	9.83E-01	9.83E-03	9.83E-05	9.83E-06	9.83E-07	9.83E-08	9.83E-09
TOTAL	<MDA	1.48E+04	1.48E+02	1.48E+00	1.48E-02	1.48E-04	1.48E-05	1.48E-06	1.48E-07	1.48E-08
I-131 DEQ	1.34E-02	8.74E+02	8.74E+00	8.74E-02	8.74E-04	8.74E-06	8.74E-07	8.74E-08	8.74E-09	8.74E-10

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

7.2 PROCESS RADIATION MONITOR DATA

This Scenario Section Contains the Following Information:

7.2.1 PROCESS MONITOR SUMMARY

THIS IS A DRILL

ST. LUCIE EVALUATED EXERCISE FEBRUARY 12, 1992
PROCESS MONITORING SYSTEM

MONITOR DESCRIPTION	UNITS	07:30	08:00	08:15	08:30	08:45	09:00	09:15	09:30	09:45
PLANT VENT SYSTEM										
GASEOUS ACTIVITY	uCi/cc	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06
IODINE ACTIVITY	cpm	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01
PARTICULATE ACTIVITY	cpm	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01
CONTAINMENT ATMOSPHERE										
GASEOUS ACTIVITY	cpm	2.53E+01	2.53E+01	2.53E+01	2.53E+01	2.53E+01	2.53E+01	2.53E+01	OOS	OOS
IODINE ACTIVITY	cpm	3.47E+01	3.47E+01	3.47E+01	3.47E+01	3.47E+01	3.47E+01	3.47E+01	OOS	OOS
PARTICULATE ACTIVITY	cpm	7.87E+01	7.87E+01	7.87E+01	7.87E+01	7.87E+01	7.87E+01	7.87E+01	7.87E+01	7.87E+01
ECCS EXHAUST - A	uCi/sec	1.33E-07	1.33E-07	1.33E-07	1.33E-07	1.33E-07	1.33E-07	1.33E-07	1.33E-07	1.33E-07
ECCS EXHAUST - B	uCi/sec	1.33E-07	1.33E-07	1.33E-07	1.33E-07	1.33E-07	1.33E-07	1.33E-07	OOS	OOS
FUEL HANDLING BLDG EXHAUST										
GASEOUS ACTIVITY	uCi/cc	2.53E-06	2.53E-06	2.53E-06	2.53E-06	2.53E-06	2.53E-06	2.53E-06	OOS	OOS
IODINE ACTIVITY	cpm	3.47E+01	3.47E+01	3.47E+01	3.47E+01	3.47E+01	3.47E+01	3.47E+01	OOS	OOS
PARTICULATE ACTIVITY	cpm	7.87E+01	7.87E+01	7.87E+01	7.87E+01	7.87E+01	7.87E+01	7.87E+01	OOS	OOS
MAIN STEAM LINE - 1A	mR/hr	9.05E-01	9.05E-01	9.05E-01	9.05E-01	9.05E-01	9.05E-01	9.05E-01	9.05E-01	9.05E-01
MAIN STEAM LINE - 1B	mR/hr	1.59E+00	1.59E+00	1.59E+00	1.59E+00	1.59E+00	1.59E+00	1.59E+00	1.59E+00	1.59E+00
CONDENSER AIR EJECTOR	cpm	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02
COMPONENT COOLING WATE	cpm	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02
COMPONENT COOLING WATE	cpm	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	OOS	OOS
STEAM GEN A BLOWDOWN	cpm	9.00E+02	9.00E+02	9.00E+02	9.00E+02	9.00E+02	9.00E+02	9.00E+02	9.00E+02	9.00E+02
STEAM GEN B BLOWDOWN	cpm	1.00E+03	1.00E+03	1.00E+03	1.00E+03	1.00E+03	1.00E+03	1.00E+03	OOS	OOS
GASEOUS RADWASTE	cpm	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04
LIQUID RADWASTE	cpm	5.90E+01	5.90E+01	5.90E+01	5.90E+01	5.90E+01	5.90E+01	5.90E+01	OOS	OOS

OOS = Out of Service

THIS IS A DRILL

ST. LUCIE EVALUATED EXERCISE FEBRUARY 12, 1992
PROCESS MONITORING SYSTEM

MONITOR DESCRIPTION	UNITS	10:00	10:15	10:30	10:45	11:00	11:15	11:30	11:45	12:00
PLANT VENT SYSTEM										
GASEOUS ACTIVITY	uCi/cc	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06
IODINE ACTIVITY	cpm	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01
PARTICULATE ACTIVITY	cpm	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01
CONTAINMENT ATMOSPHERE										
GASEOUS ACTIVITY	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
IODINE ACTIVITY	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
PARTICULATE ACTIVITY	cpm	7.87E+01	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06
ECCS EXHAUST - A	uCi/sec	1.33E-07	2.00E-07	3.00E-07	3.00E-07	3.00E-07	3.00E-07	3.00E-07	3.00E-07	3.00E-07
ECCS EXHAUST - B	uCi/sec	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
FUEL HANDLING BLDG EXHAUST										
GASEOUS ACTIVITY	uCi/cc	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
IODINE ACTIVITY	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
PARTICULATE ACTIVITY	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
MAIN STEAM LINE - 1A	mR/hr	9.05E-01	9.05E-01	9.05E-01	9.05E-01	9.05E-01	9.05E-01	9.05E-01	9.05E-01	2.40E+03
MAIN STEAM LINE - 1B	mR/hr	1.59E+00	1.59E+00	1.59E+00	1.59E+00	1.59E+00	1.59E+00	1.59E+00	1.59E+00	3.00E+00
CONDENSER AIR EJECTOR	cpm	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02
COMPONENT COOLING WATE	cpm	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02
COMPONENT COOLING WATE	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
STEAM GEN A BLOWDOWN	cpm	9.00E+02	9.00E+02	9.00E+02	9.00E+02	9.00E+02	9.00E+02	9.00E+02	9.00E+02	>1.00E+06
STEAM GEN B BLOWDOWN	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
GASEOUS RADWASTE	cpm	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04
LIQUID RADWASTE	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS

OOS = Out of Service

THIS IS A DRILL

ST. LUCIE EVALUATED EXERCISE FEBRUARY 12, 1992
PROCESS MONITORING SYSTEM

MONITOR DESCRIPTION	UNITS	12:15	12:30	12:45	13:00	13:15	13:30	13:45	14:00	14:15
PLANT VENT SYSTEM										
GASEOUS ACTIVITY	uCi/cc	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06	2.03E-06
IODINE ACTIVITY	cpm	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01	1.44E+01
PARTICULATE ACTIVITY	cpm	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01	1.79E+01
CONTAINMENT ATMOSPHERE										
GASEOUS ACTIVITY	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
IODINE ACTIVITY	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
PARTICULATE ACTIVITY	cpm	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06
ECCS EXHAUST - A	uCi/sec	4.00E-06	5.00E-07	6.00E-07	6.00E-07	6.00E-07	6.00E-07	6.00E-07	6.00E-07	6.00E-07
ECCS EXHAUST - B	uCi/sec	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
FUEL HANDLING BLDG EXHAUST										
GASEOUS ACTIVITY	uCi/cc	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
IODINE ACTIVITY	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
PARTICULATE ACTIVITY	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
MAIN STEAM LINE - 1A	mR/hr	1.10E+03	2.50E+02	6.00E+01	9.20E+00	2.20E+00	5.30E-01	5.10E-01	5.00E-01	5.00E-01
MAIN STEAM LINE - 1B	mR/hr	3.00E+00	3.00E+00	3.00E+00	3.00E+00	2.50E+00	1.00E+00	1.00E+00	1.00E+00	1.00E+00
CONDENSER AIR EJECTOR	cpm	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02	1.00E+02
COMPONENT COOLING WATE	cpm	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02	2.50E+02
COMPONENT COOLING WATE	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
STEAM GEN A BLOWDOWN	cpm	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06	>1.00E+06
STEAM GEN B BLOWDOWN	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
GASEOUS RADWASTE	cpm	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04	1.00E+04
LIQUID RADWASTE	cpm	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS

OOS = Out of Service

THIS IS A DRILL

ST. LUCIE EVALUATED EXERCISE FEBRUARY 12, 1992
PROCESS MONITORING SYSTEM

MONITOR DESCRIPTION	UNITS	14:30	14:45	15:00
PLANT VENT SYSTEM				
GASEOUS ACTIVITY	uCi/cc	2.03E-06	2.03E-06	2.03E-06
IODINE ACTIVITY	cpm	1.44E+01	1.44E+01	1.44E+01
PARTICULATE ACTIVITY	cpm	1.79E+01	1.79E+01	1.79E+01
CONTAINMENT ATMOSPHERE				
GASEOUS ACTIVITY	cpm	OOS	OOS	OOS
IODINE ACTIVITY	cpm	OOS	OOS	OOS
PARTICULATE ACTIVITY	cpm	>1.00E+06	>1.00E+06	>1.00E+06
ECCS EXHAUST - A	uCi/sec	6.00E-07	6.00E-07	6.00E-07
ECCS EXHAUST - B	uCi/sec	OOS	OOS	OOS
FUEL HANDLING BLDG EXHAUST				
GASEOUS ACTIVITY	uCi/cc	OOS	OOS	OOS
IODINE ACTIVITY	cpm	OOS	OOS	OOS
PARTICULATE ACTIVITY	cpm	OOS	OOS	OOS
MAIN STEAM LINE - 1A	mR/hr	5.00E-01	5.00E-01	5.00E-01
MAIN STEAM LINE - 1B	mR/hr	1.00E+00	1.00E+00	1.00E+00
CONDENSER AIR EJECTOR	cpm	1.00E+02	1.00E+02	1.00E+02
COMPONENT COOLING WATE	cpm	2.50E+02	2.50E+02	2.50E+02
COMPONENT COOLING WATE	cpm	OOS	OOS	OOS
STEAM GEN A BLOWDOWN	cpm	>1.00E+06	>1.00E+06	>1.00E+06
STEAM GEN B BLOWDOWN	cpm	OOS	OOS	OOS
GASEOUS RADWASTE	cpm	1.00E+04	1.00E+04	1.00E+04
LIQUID RADWASTE	cpm	OOS	OOS	OOS

OOS = Out of Service

7.2.2 PROCESS MONITOR DATA CARDS

This Data to be Provided Later

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

7.3 AREA RADIATION MONITOR DATA

This Scenario Section Contains the Following Information:

Section 7.3.1	AREA RADIATION MONITOR SUMMARY
Section 7.3.2	AREA RADIATION MONITOR DATA CARDS

ST. LUCIE NUCLEAR PLANT EVALUATED EXERCISE
AREA RADIATION MONITOR DATA SUMMARY

MONITOR #	MONITOR DESCRIPTION	UNITS	TIME: 07:30	07:45	08:00	08:15	08:30	08:45	09:00	09:15
RE-1	CONTROL ROOM AREA	mR/hr	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01
RE-3	CIS...A 0 DEGREES	mR/hr	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00	2.34E+00
RE-4	CIS...B 270 DEGREES	mR/hr	6.98E+00	6.98E+00	6.98E+00	6.98E+00	6.98E+00	6.98E+00	6.98E+00	6.98E+00
RE-5	CIS...C 180 DEGREES	mR/hr	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00	4.14E+00
RE-6	CIS...D 90 DEGREES	mR/hr	1.07E+00	1.07E+00	1.07E+00	1.07E+00	1.07E+00	1.07E+00	1.07E+00	1.07E+00
RE-7	SPENT FUEL POOL	mR/hr	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00
RE-8	SPENT FUEL POOL	mR/hr	1.68E-01	1.68E-01	1.68E-01	1.68E-01	1.68E-01	1.68E-01	1.68E-01	1.68E-01
RE-9	SPENT FUEL POOL	mR/hr	1.58E+00	1.58E+00	1.58E+00	1.58E+00	1.58E+00	1.58E+00	1.58E+00	1.58E+00
RE-10	BORIC ACID PRECON. FILT.	mR/hr	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01
RE-11	WASTE FILTER MON.	mR/hr	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00
RE-12	LAUNDRY FILTER	mR/hr	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00	2.07E+00
RE-13	WASTE GAS COMPRESSOR	mR/hr	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00
RE-14	CHARGING PUMP AREA	mR/hr	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00
RE-15	HOLD UP DRAIN PUMP AREA	mR/hr	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01
RE-16	SAMPLE ROOM AREA	mR/hr	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00
RE-17	ION EXCH. CORRIDOR	mR/hr	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01
RE-18	ION EXCH. VALVE GALLERY	mR/hr	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01
RE-19	DRUMMING STATION AREA	mR/hr	1.29E+00	1.29E+00	1.29E+00	1.29E+00	1.29E+00	1.29E+00	1.29E+00	1.29E+00
RE-20	PURIFICATION FILTER AREA	mR/hr	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00
RE-21	SPENT RESIN CORRIDOR	mR/hr	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01
RE-22	REACTOR DRAIN PUMPS AREA	mR/hr	1.89E+00	1.89E+00	1.89E+00	1.89E+00	1.89E+00	1.89E+00	1.89E+00	1.89E+00
RE-24	HVAC EQUIPMENT ROOM AREA	mR/hr	1.84E-01	1.84E-01	1.84E-01	1.84E-01	1.84E-01	1.84E-01	1.84E-01	1.84E-01
RE-25	CHEMICAL DRAIN PUMP AREA	mR/hr	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01
RE-26	VOL CONTROL TK CORRIDOR	mR/hr	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01
RE-27	LETDOWN HEAT EXCH.	mR/hr	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00
RE-36	FUEL POOL	mR/hr	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00	3.03E+00
RE-37	NEW FUEL STORAGE	mR/hr	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01
RE-38	AERATED WASTE STORAGE	mR/hr	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01
RE-39	BORIC ACID CONCENTRATOR	mR/hr	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01
RE-52	POST LOCA A	mR/hr	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01
RE-53	POST LOCA B	mR/hr	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01
RE-58	CHIRM A	R/hr	1.04E+00	1.04E+00	1.04E+00	1.04E+00	1.04E+00	1.04E+00	1.04E+00	1.04E+00
RE-59	CHIRM B	R/hr	1.29E+00	1.29E+00	1.29E+00	1.29E+00	1.29E+00	1.29E+00	1.29E+00	1.29E+00

OOS=OUT OF SERVICE

ST. LUCIE NUCLEAR PLANT EVALUATED EXERCISE
AREA RADIATION MONITOR DATA SUMMARY

MONITOR #	MONITOR DESCRIPTION	UNITS	TIME: 09:30	09:45	10:00	10:15	10:30	10:45	11:00	11:15
RE-1	CONTROL ROOM AREA	mR/hr	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	3.00E-01
RE-3	CIS...A 0 DEGREES	mR/hr	2.34E+00	2.34E+00	2.34E+00	2.30E+01	4.18E+01	3.00E+02	1.80E+03	>1.00E+05
RE-4	CIS...B 270 DEGREES	mR/hr	6.98E+00	6.98E+00	6.98E+00	2.30E+01	4.18E+01	3.00E+02	1.80E+03	>1.00E+05
RE-5	CIS...C 180 DEGREES	mR/hr	4.14E+00	4.14E+00	4.14E+00	2.30E+01	4.18E+01	3.00E+02	1.80E+03	>1.00E+05
RE-6	CIS...D 90 DEGREES	mR/hr	1.07E+00	1.07E+00	1.07E+00	2.30E+01	4.18E+01	3.00E+02	1.80E+03	>1.00E+05
RE-7	SPENT FUEL POOL	mR/hr	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	1.24E+00	>1.00E+04
RE-8	SPENT FUEL POOL	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-9	SPENT FUEL POOL	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-10	BORIC ACID PRECON. FILT.	mR/hr	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01
RE-11	WASTE FILTER MON.	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-12	LAUNDRY FILTER	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-13	WASTE GAS COMPRESSOR	mR/hr	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00
RE-14	CHARGING PUMP AREA	mR/hr	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00
RE-15	HOLD UP DRAIN PUMP AREA	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-16	SAMPLE ROOM AREA	mR/hr	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00
RE-17	ION EXCH. CORRIDOR	mR/hr	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01
RE-18	ION EXCH. VALVE GALLERY	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-19	DRUMMING STATION AREA	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-20	PURIFICATION FILTER AREA	mR/hr	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00	2.33E+00	4.00E+00
RE-21	SPENT RESIN CORRIDOR	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-22	REACTOR DRAIN PUMPS AREA	mR/hr	1.89E+00	1.89E+00	1.89E+00	1.89E+00	1.89E+00	1.89E+00	1.89E+00	3.00E+01
RE-24	HVAC EQUIPMENT ROOM AREA	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-25	CHEMICAL DRAIN PUMP AREA	mR/hr	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01
RE-26	VOL CONTROL TK CORRIDOR	mR/hr	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01
RE-27	LETDOWN HEAT EXCH.	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-36	FUEL POOL	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-37	NEW FUEL STORAGE	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-38	AERATED WASTE STORAGE	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-39	BORIC ACID CONCENTRATOR	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-52	POST LOCA A	mR/hr	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01	1.55E-01	2.00E+03
RE-53	POST LOCA B	mR/hr	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.69E-01	2.00E+03
RE-58	CHIRM A	R/hr	6.04E-01	6.04E-01	6.04E-01	6.04E-01	6.04E-01	6.04E-01	1.30E+00	2.60E+04
RE-59	CHIRM B	R/hr	6.00E-01	6.00E-01	6.00E-01	6.00E-01	6.00E-01	6.00E-01	1.30E+00	2.60E+04

OOS=OUT OF SERVICE

ST. LUCIE NUCLEAR PLANT EVALUATED EXERCISE
AREA RADIATION MONITOR DATA SUMMARY

MONITOR #	MONITOR DESCRIPTION	UNITS	TIME: 11:30	11:45	12:00	12:15	12:30	12:45	13:00	13:15
RE-1	CONTROL ROOM AREA	mR/hr	8.00E-01	2.00E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01
RE-3	CIS...A 0 DEGREES	mR/hr	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05
RE-4	CIS...B 270 DEGREES	mR/hr	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05
RE-5	CIS...C 180 DEGREES	mR/hr	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05
RE-6	CIS...D 90 DEGREES	mR/hr	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05
RE-7	SPENT FUEL POOL	mR/hr	>1.00E+04	>1.00E+04	>1.00E+04	>1.00E+04	>1.00E+04	>1.00E+04	>1.00E+04	>1.00E+04
RE-8	SPENT FUEL POOL	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-9	SPENT FUEL POOL	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-10	BORIC ACID PRECON. FILT.	mR/hr	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01
RE-11	WASTE FILTER MON.	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-12	LAUNDRY FILTER	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-13	WASTE GAS COMPRESSOR	mR/hr	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00
RE-14	CHARGING PUMP AREA	mR/hr	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00
RE-15	HOLD UP DRAIN PUMP AREA	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-16	SAMPLE ROOM AREA	mR/hr	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00
RE-17	ION EXCH. CORRIDOR	mR/hr	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01
RE-18	ION EXCH. VALVE GALLERY	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-19	DRUMMING STATION AREA	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-20	PURIFICATION FILTER AREA	mR/hr	8.00E+00	8.00E+00	1.23E+01	1.13E+01	1.13E+01	1.13E+01	1.13E+01	1.13E+01
RE-21	SPENT RESIN CORRIDOR	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-22	REACTOR DRAIN PUMPS AREA	mR/hr	7.50E+01	7.50E+01	1.00E+02	9.00E+01	9.00E+01	9.00E+01	9.00E+01	9.00E+01
RE-24	HVAC EQUIPMENT ROOM AREA	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-25	CHEMICAL DRAIN PUMP AREA	mR/hr	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01
RE-26	VOL CONTROL TK CORRIDOR	mR/hr	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01
RE-27	LETDOWN HEAT EXCH.	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-36	FUEL POOL	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-37	NEW FUEL STORAGE	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-38	AERATED WASTE STORAGE	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-39	BORIC ACID CONCENTRATOR	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-52	POST LOCA A	mR/hr	5.00E+03	5.00E+03	7.00E+03	6.50E+03	6.00E+03	6.00E+03	6.00E+03	6.00E+03
RE-53	POST LOCA B	mR/hr	5.00E+03	5.00E+03	7.00E+03	6.50E+03	6.00E+03	6.00E+03	6.00E+03	6.00E+03
RE-58	CHHRM A	R/hr	5.00E+04	5.00E+04	6.75E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04
RE-59	CHHRM B	R/hr	5.00E+04	5.00E+04	6.75E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04

OOS=OUT OF SERVICE

ST. LUCIE NUCLEAR PLANT EVALUATED EXERCISE
AREA RADIATION MONITOR DATA SUMMARY

MONITOR #	MONITOR DESCRIPTION	UNITS	TIME: 13:30	13:45	14:00	14:15	14:30	14:45	15:00
RE-1	CONTROL ROOM AREA	mR/hr	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01	1.90E-01
RE-3	CIS...A 0 DEGREES	mR/hr	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05
RE-4	CIS...B 270 DEGREES	mR/hr	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05
RE-5	CIS...C 180 DEGREES	mR/hr	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05
RE-6	CIS...D 90 DEGREES	mR/hr	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05	>1.00E+05
RE-7	SPENT FUEL POOL	mR/hr	>1.00E+04	>1.00E+04	>1.00E+04	>1.00E+04	>1.00E+04	>1.00E+04	>1.00E+04
RE-8	SPENT FUEL POOL	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-9	SPENT FUEL POOL	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-10	BORIC ACID PRECON. FILT.	mR/hr	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01	5.69E-01
RE-11	WASTE FILTER MON.	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-12	LAUNDRY FILTER	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-13	WASTE GAS COMPRESSOR	mR/hr	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00	1.50E+00
RE-14	CHARGING PUMP AREA	mR/hr	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00	1.82E+00
RE-15	HOLD UP DRAIN PUMP AREA	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-16	SAMPLE ROOM AREA	mR/hr	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00	6.45E+00
RE-17	ION EXCH. CORRIDOR	mR/hr	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01	4.94E-01
RE-18	ION EXCH. VALVE GALLERY	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-19	DRUMMING STATION AREA	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-20	PURIFICATION FILTER AREA	mR/hr	1.13E+01	1.13E+01	1.13E+01	1.13E+01	1.13E+01	1.13E+01	1.13E+01
RE-21	SPENT RESIN CORRIDOR	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-22	REACTOR DRAIN PUMPS AREA	mR/hr	9.00E+01	9.00E+01	9.00E+01	9.00E+01	9.00E+01	9.00E+01	9.00E+01
RE-24	HVAC EQUIPMENT ROOM AREA	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-25	CHEMICAL DRAIN PUMP AREA	mR/hr	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01	7.29E-01
RE-26	VOL CONTROL TK CORRIDOR	mR/hr	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01	1.60E-01
RE-27	LETDOWN HEAT EXCH.	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-36	FUEL POOL	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-37	NEW FUEL STORAGE	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-38	AERATED WASTE STORAGE	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-39	BORIC ACID CONCENTRATOR	mR/hr	OOS	OOS	OOS	OOS	OOS	OOS	OOS
RE-52	POST LOCA A	mR/hr	6.00E+03	6.00E+03	6.00E+03	6.00E+03	6.00E+03	6.00E+03	6.00E+03
RE-53	POST LOCA B	mR/hr	6.00E+03	6.00E+03	6.00E+03	6.00E+03	6.00E+03	6.00E+03	6.00E+03
RE-58	CHIRM A	R/hr	6.50E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04
RE-59	CHIRM B	R/hr	6.50E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04	6.50E+04

OOS=OUT OF SERVICE

7.3.1 AREA RADIATION MONITOR SUMMARY

7.3.2 AREA RADIATION MONITOR DATA CARDS

This Data to be Provided Later

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

7.4 ONSITE RADIOLOGICAL DATA

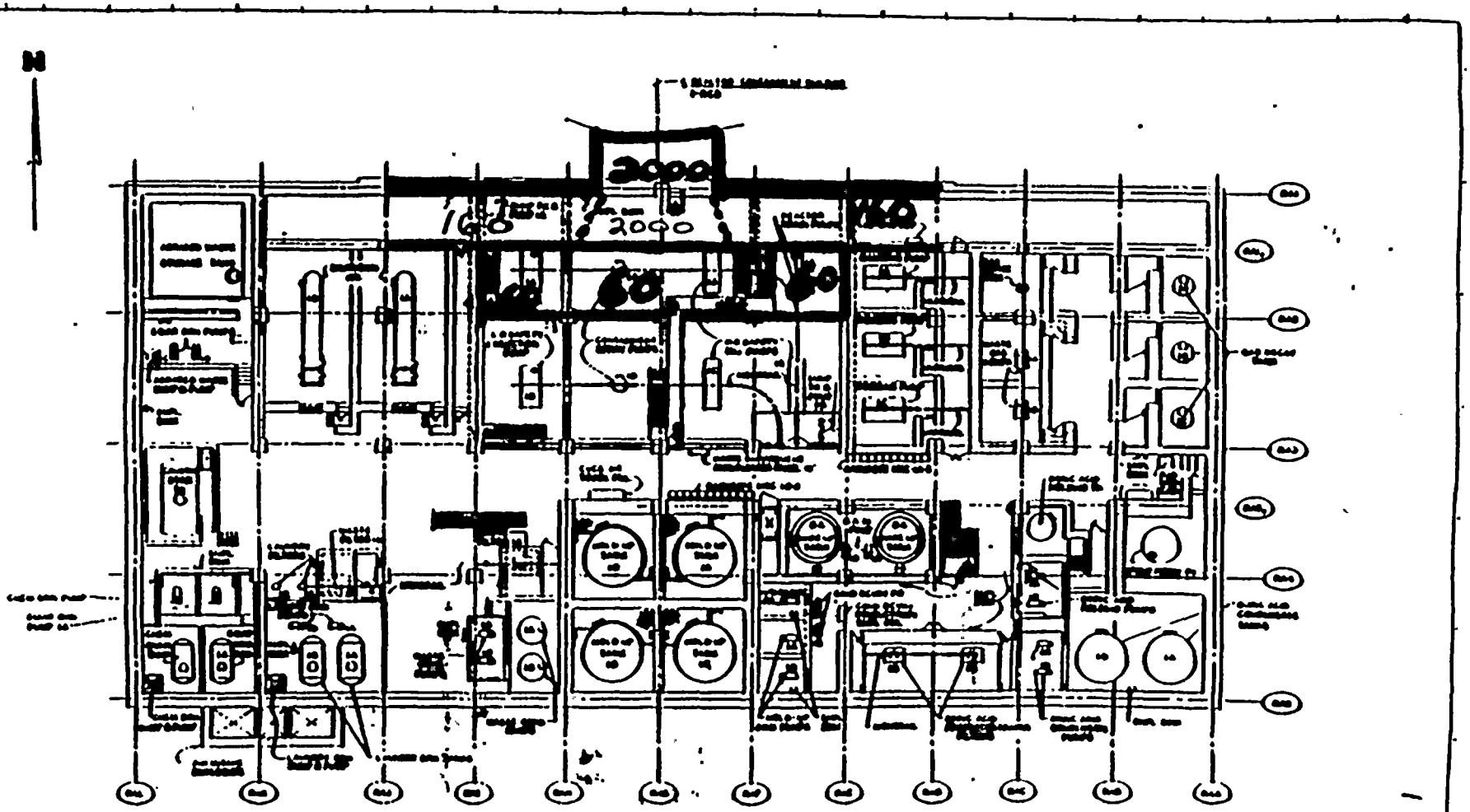
This Scenario Section Contains the Following Information:

Section 7.4.1	INPLANT AREA MAPS
Section 7.4.2	ONSITE PLUME MAP

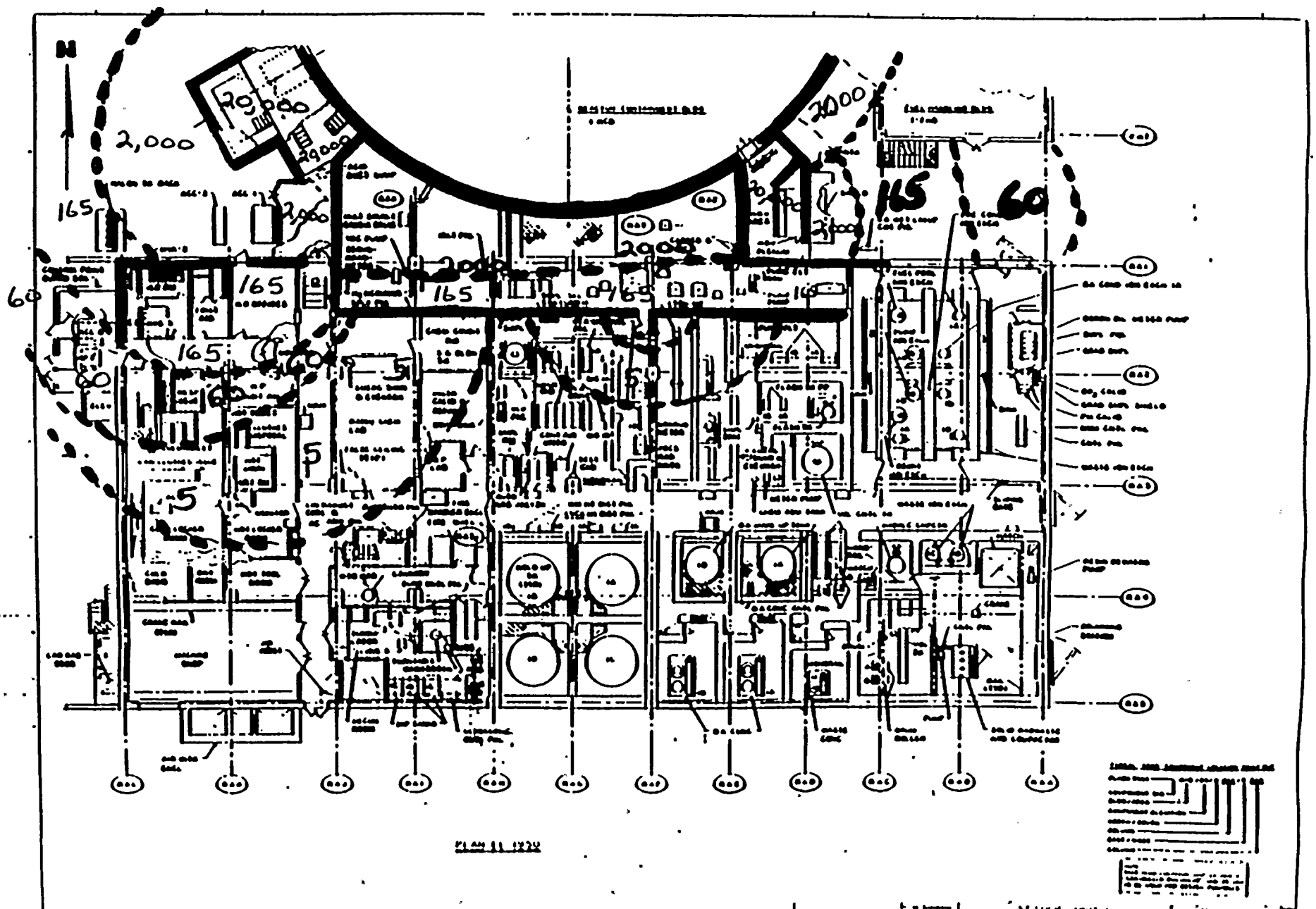
7.4.1 INPLANT AREA MAPS

TIME: 1115 - 1129

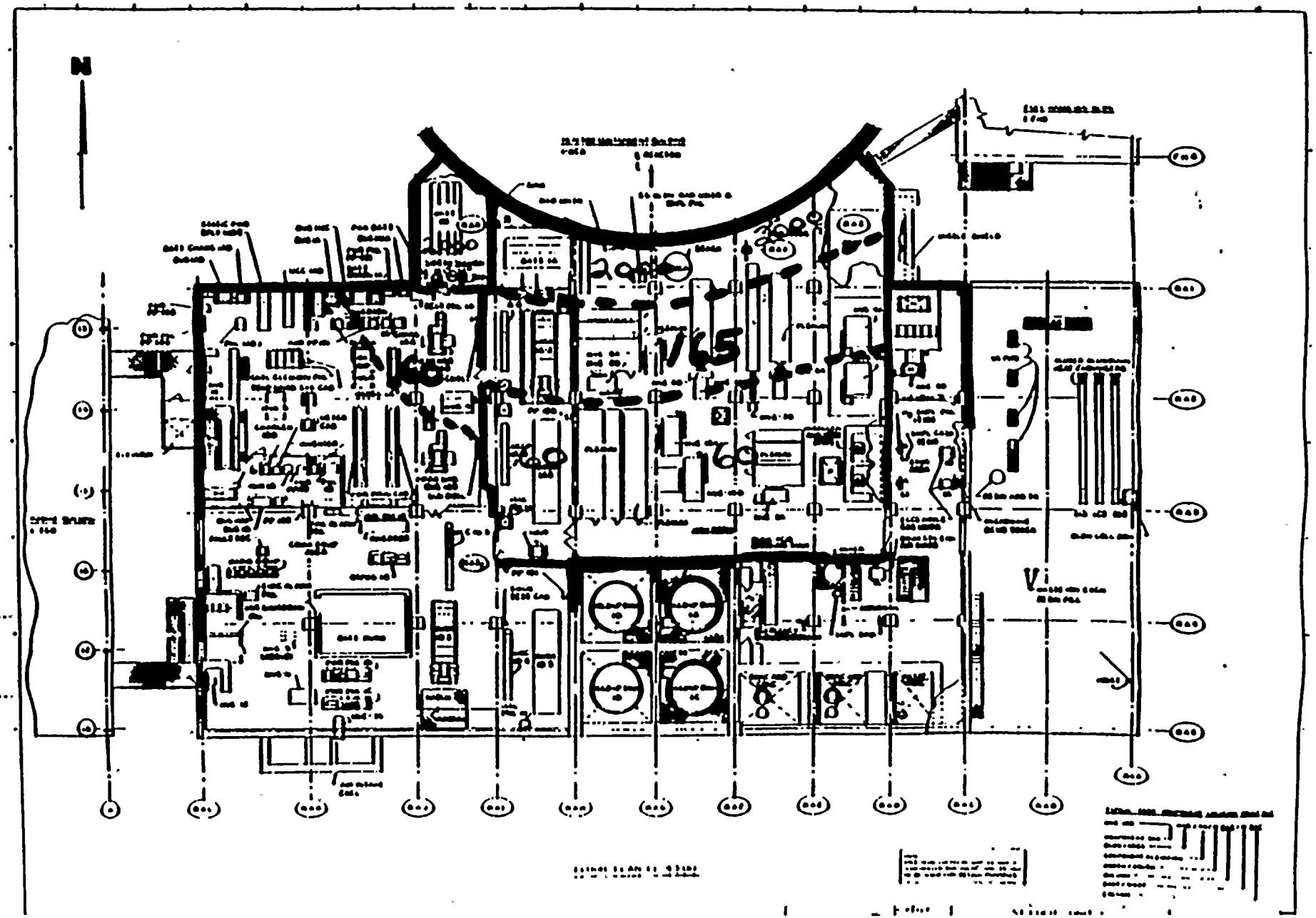
ALL SHEETS
- DOSE RATES IN MR/HR ?? (OPEN = CLOSED WINDOW)
- CONTAMINATION (CPM) = ACTUAL ?
- AIR ACTIVITY = ACTUAL



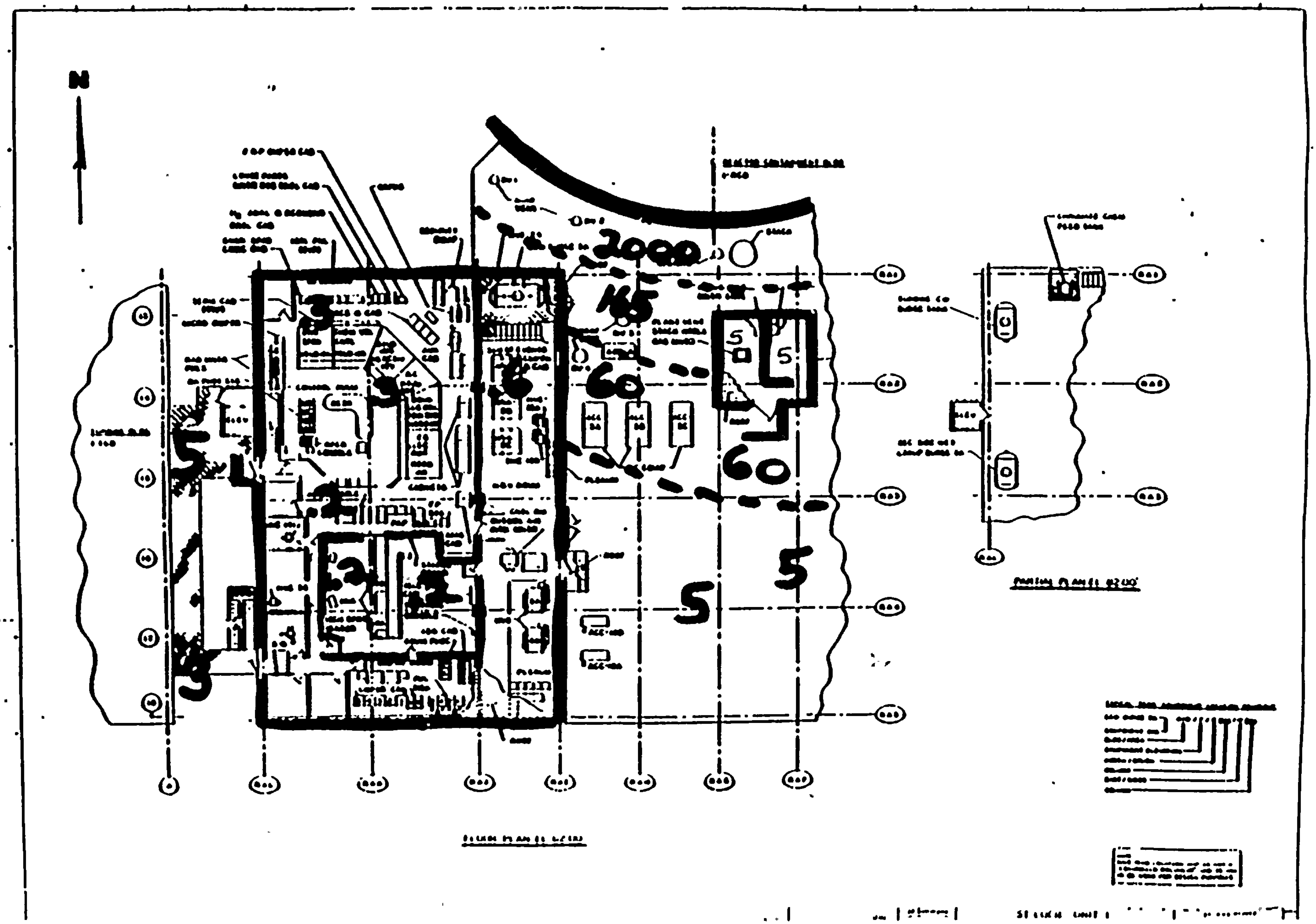
Time : 1115-1129



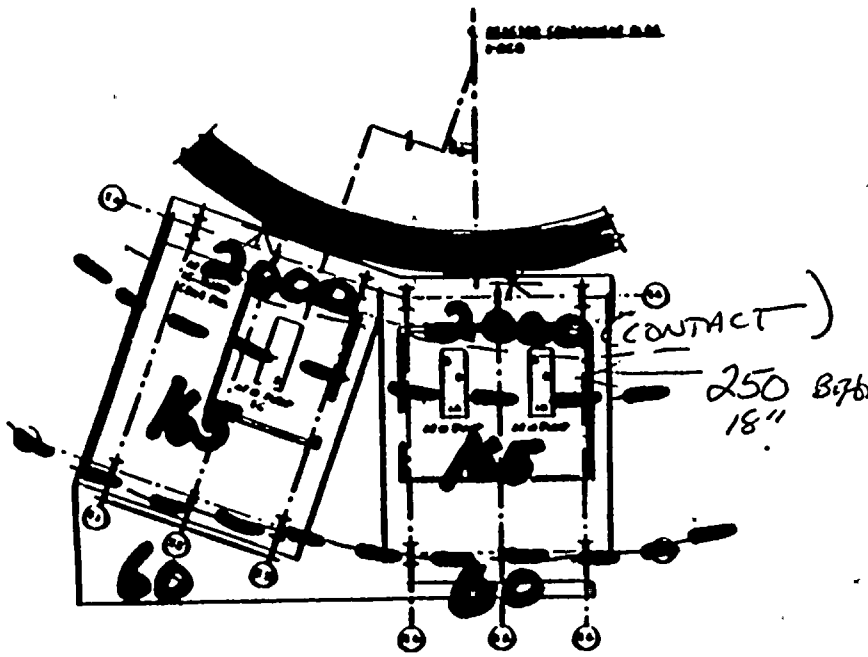
TIME 1115 - 1129

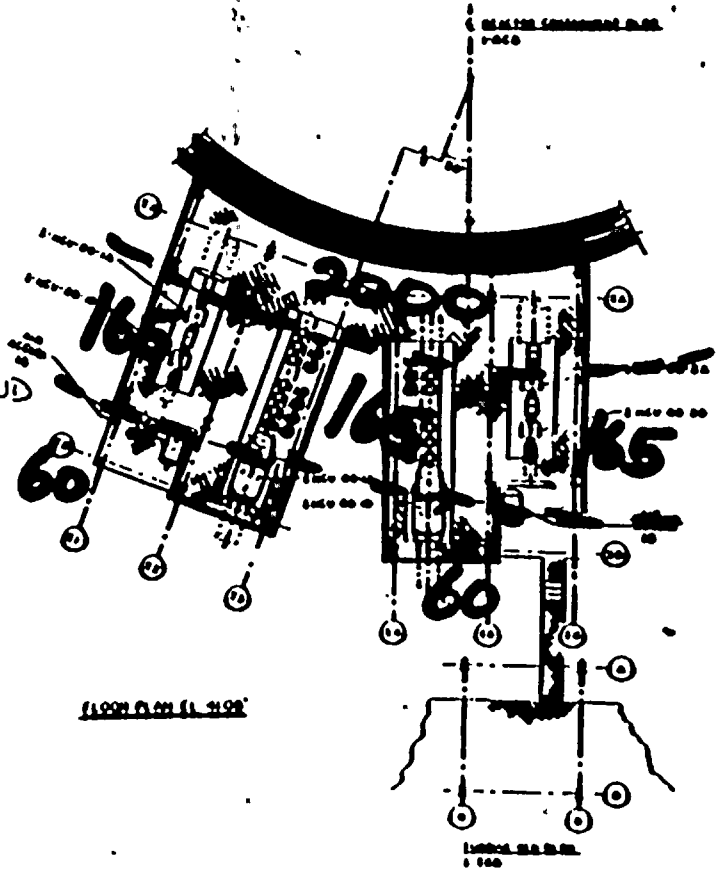


TIME: 1115-1129

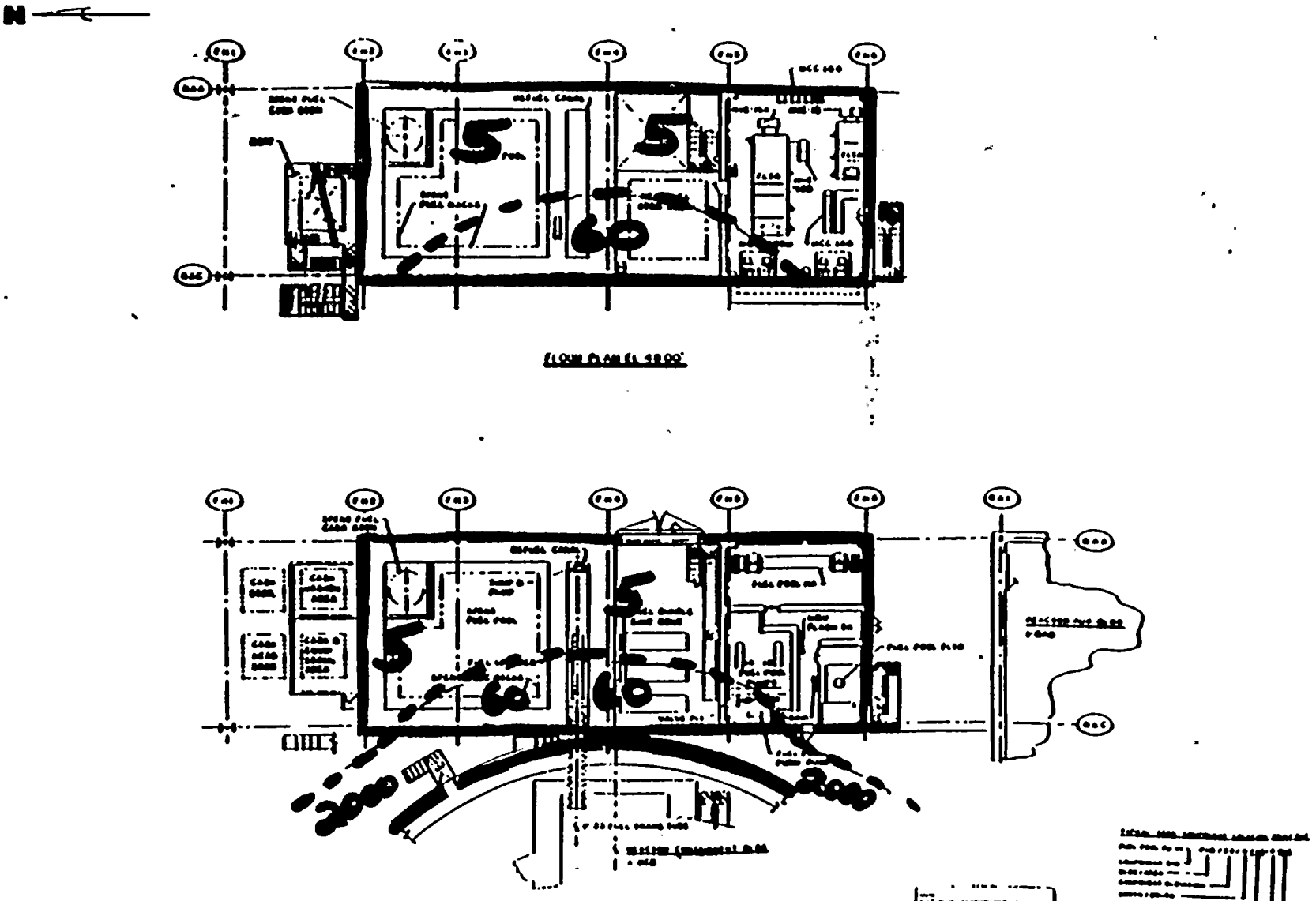


TIME: 1115-1129



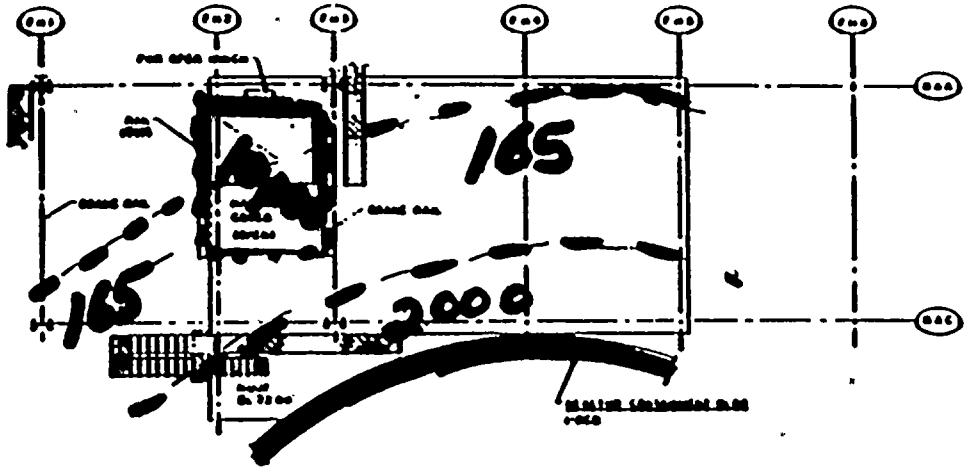


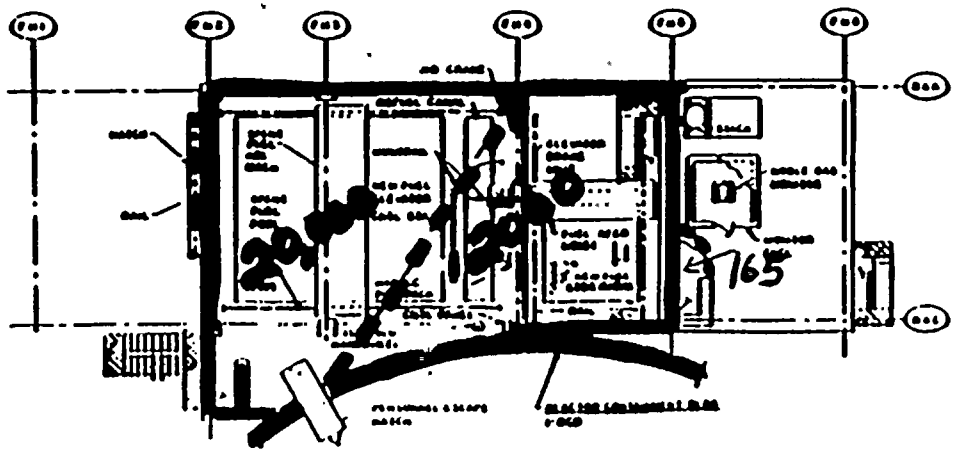
TIME: 1115 - 1129



TIME: 1115 - 1129

N

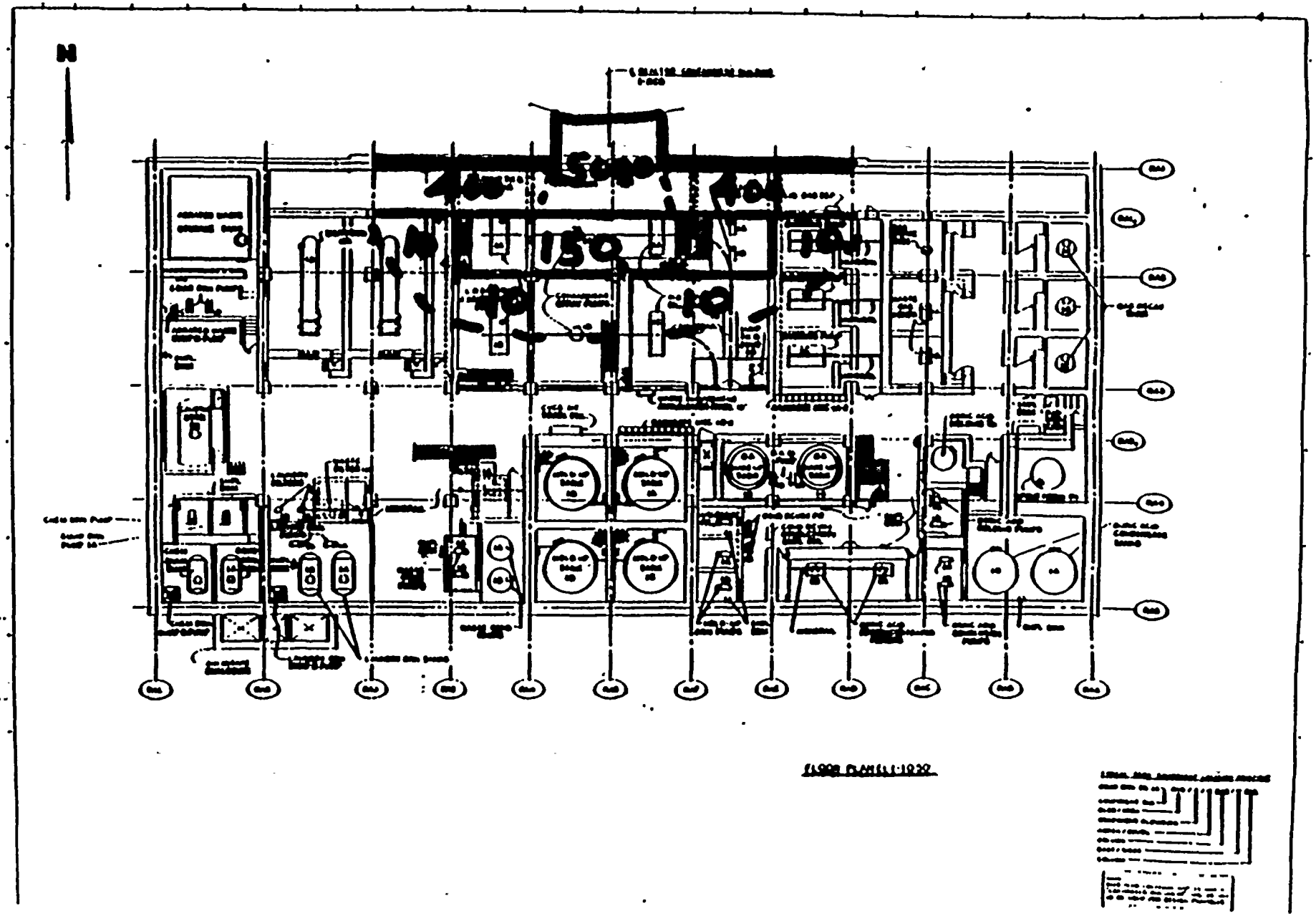




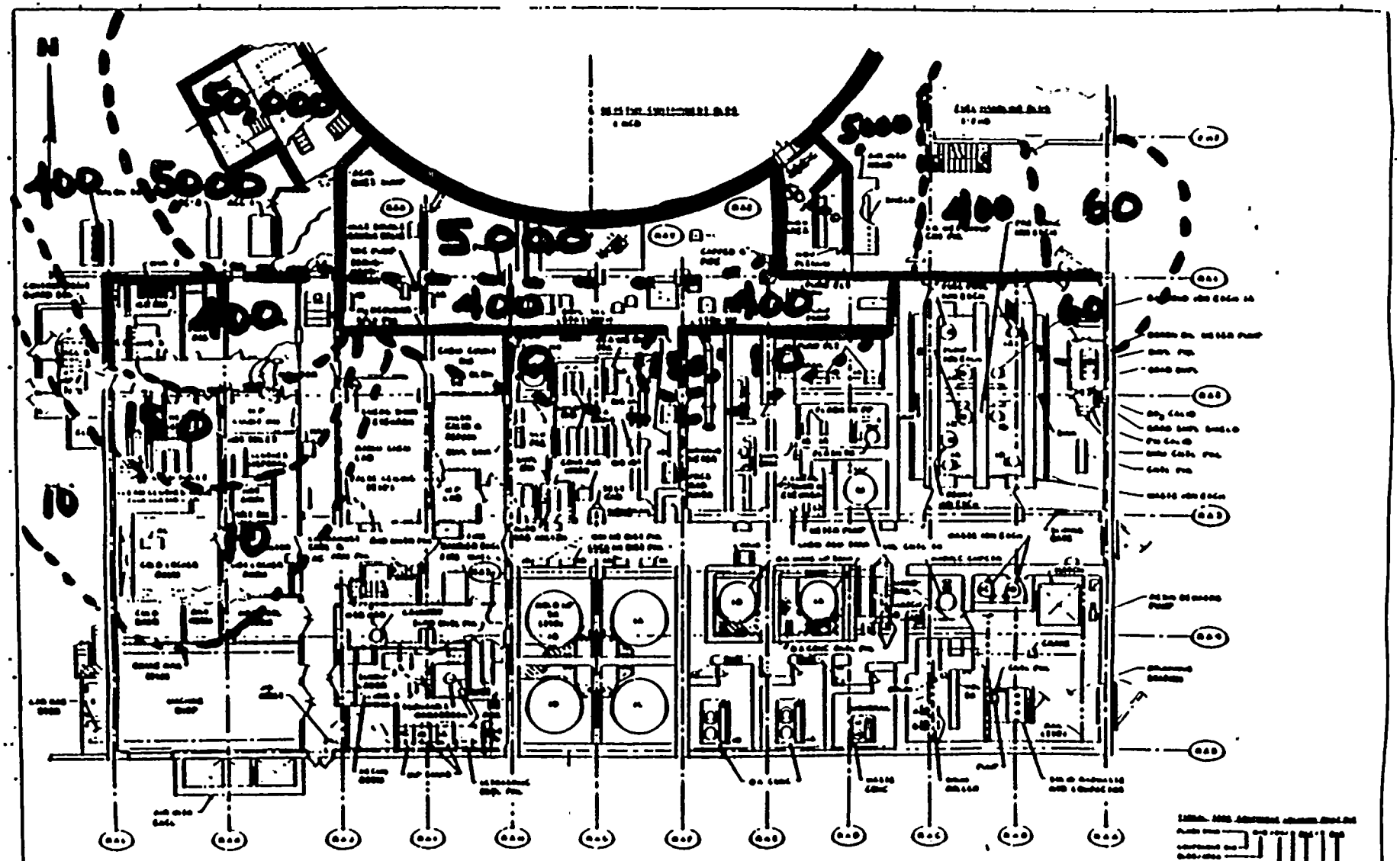
TIME : 1130 - 1159

ALL SHEETS

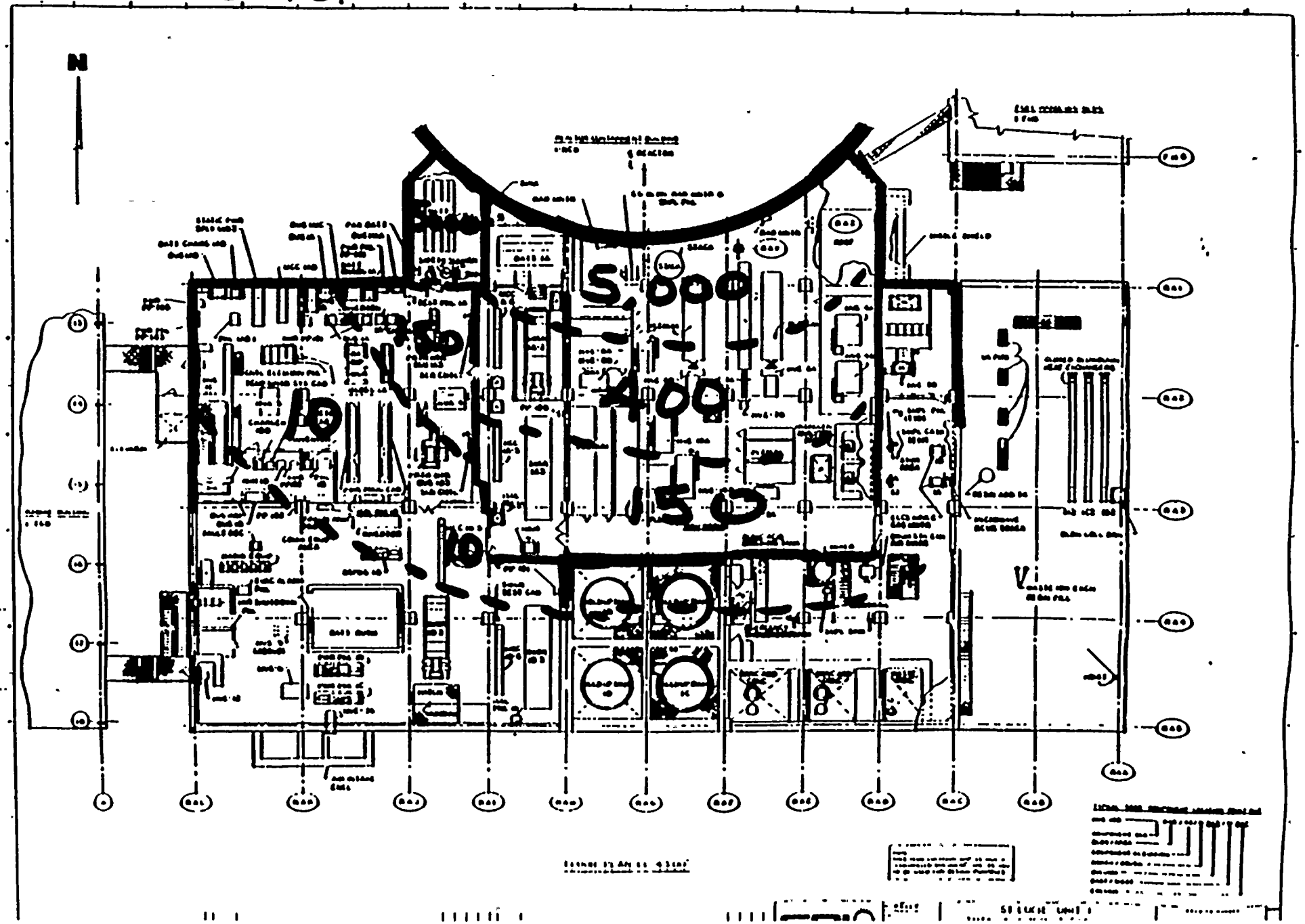
- DOSE RATES IN MR/HR BY (OPEN = CLOSED W/ MIRROR)
- CONTAMINATION (CPM) = ACTUAL
- AIR ACTIVITY = ACTUAL



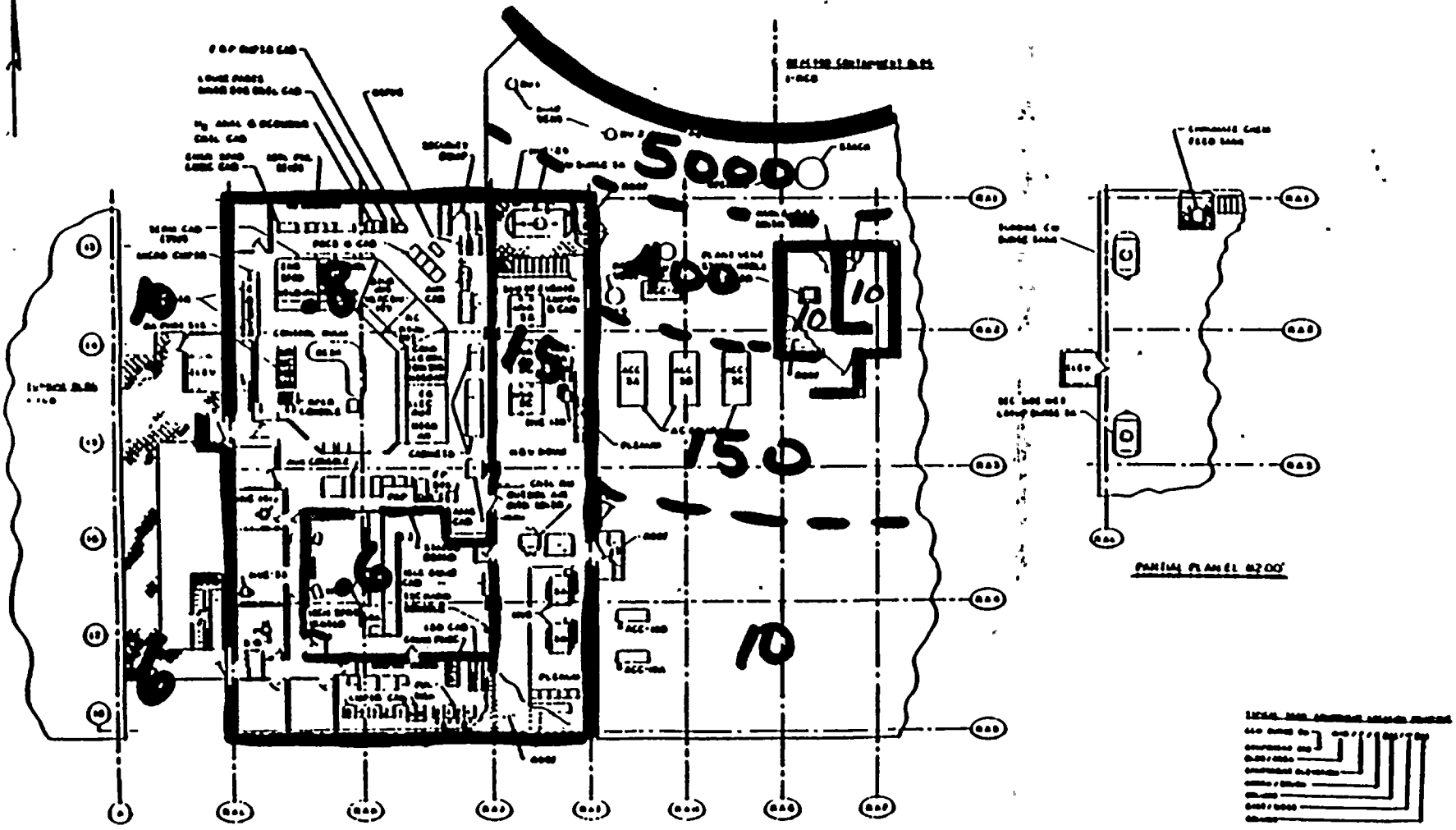
TIME : 1130 – 1159



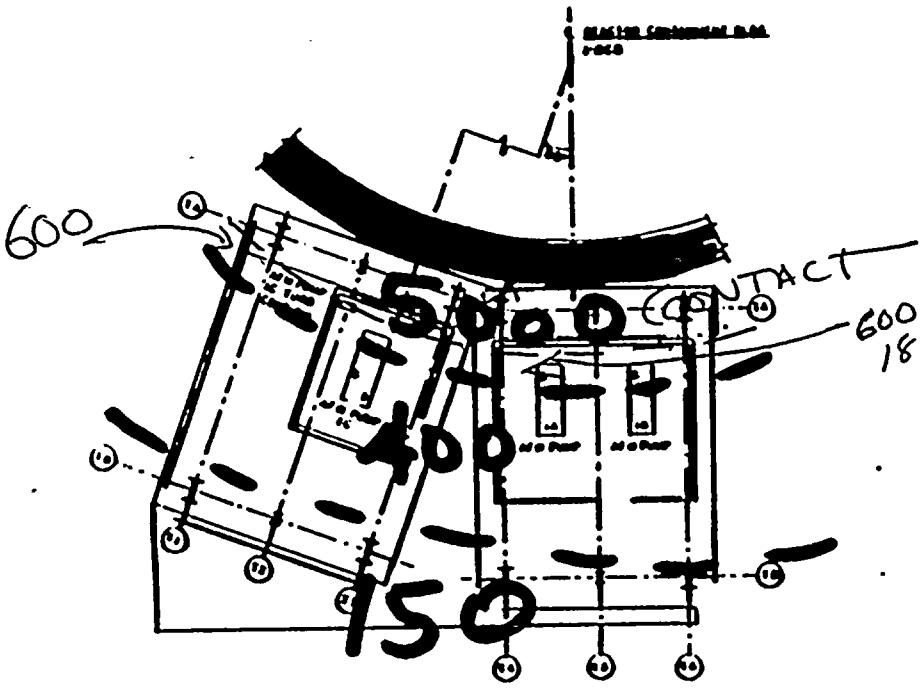
TIME: 1130-1159

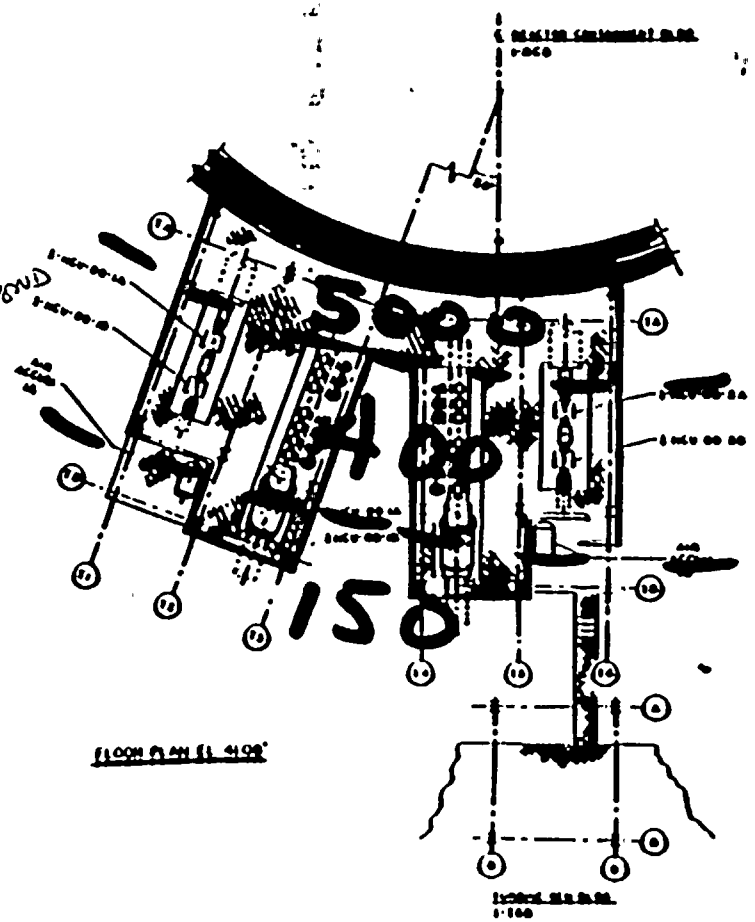


TIME: 1130-1159

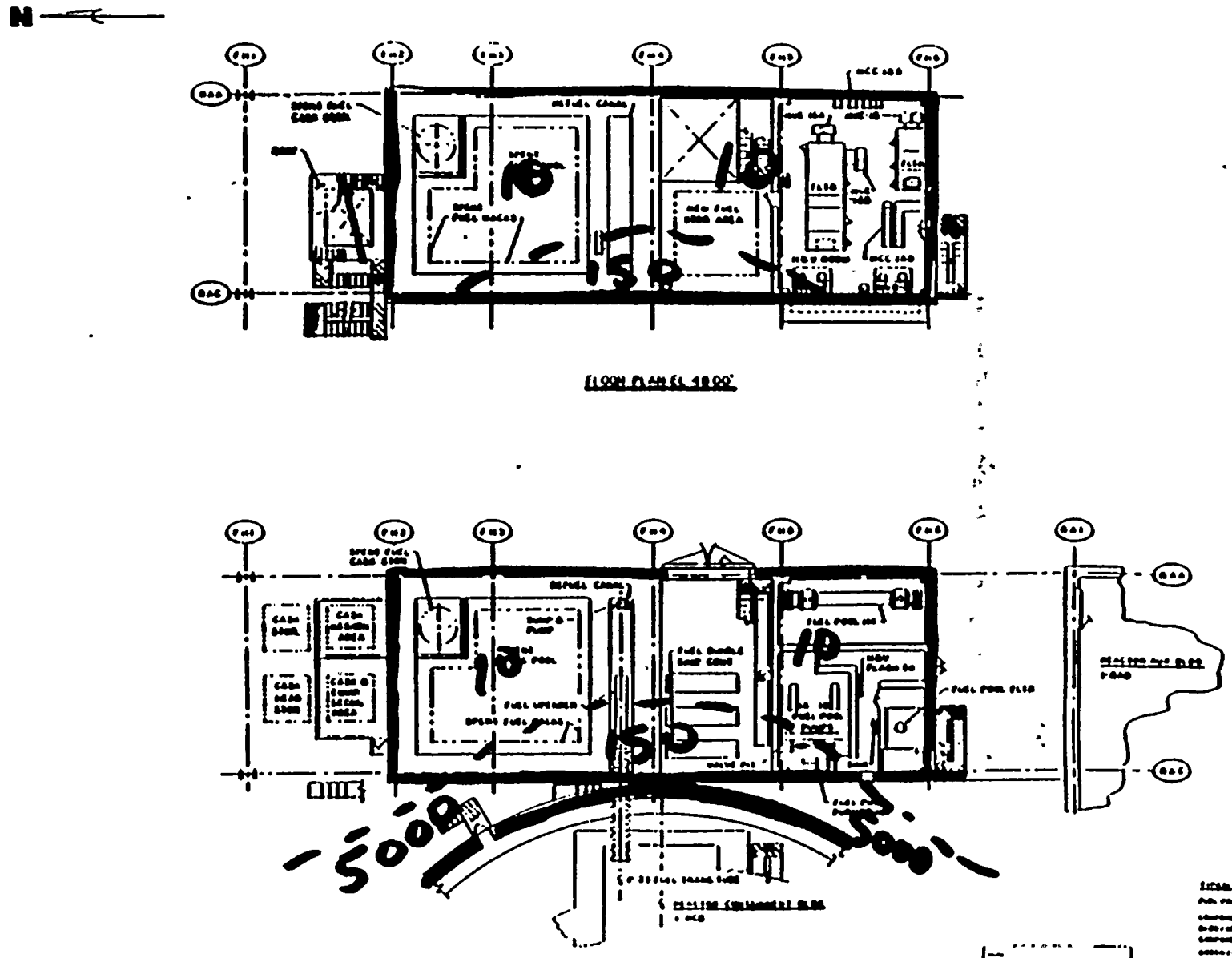


TIME: 1130-1159



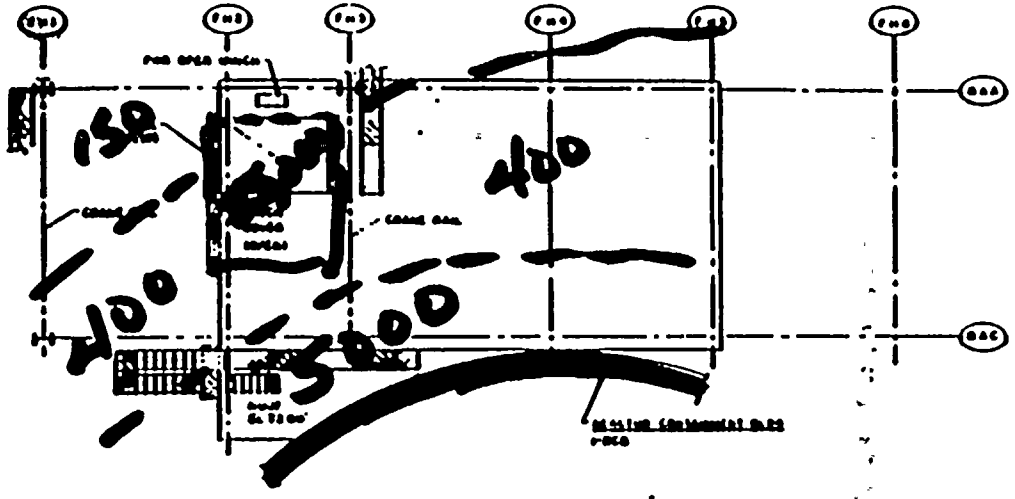


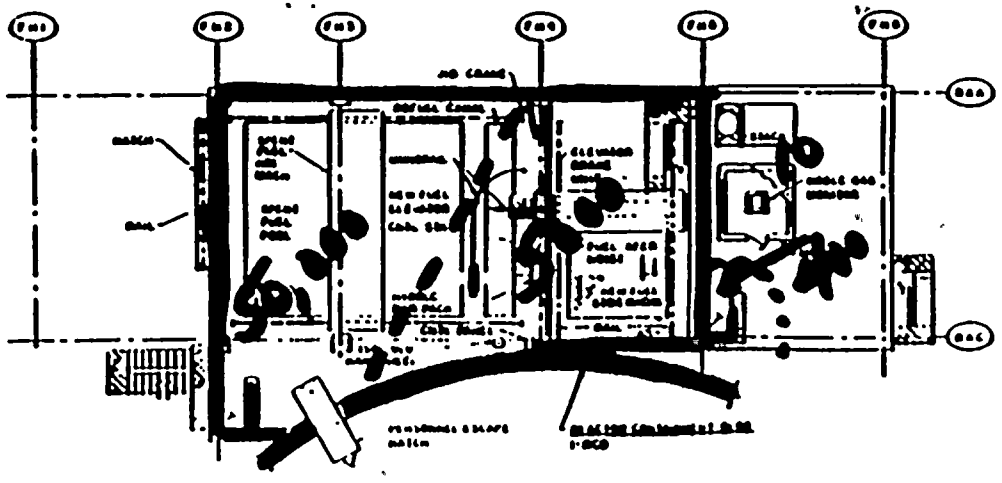
TIME: 1130 - 1159

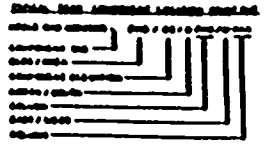


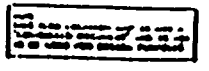
TIME: 1130 - 1159

N









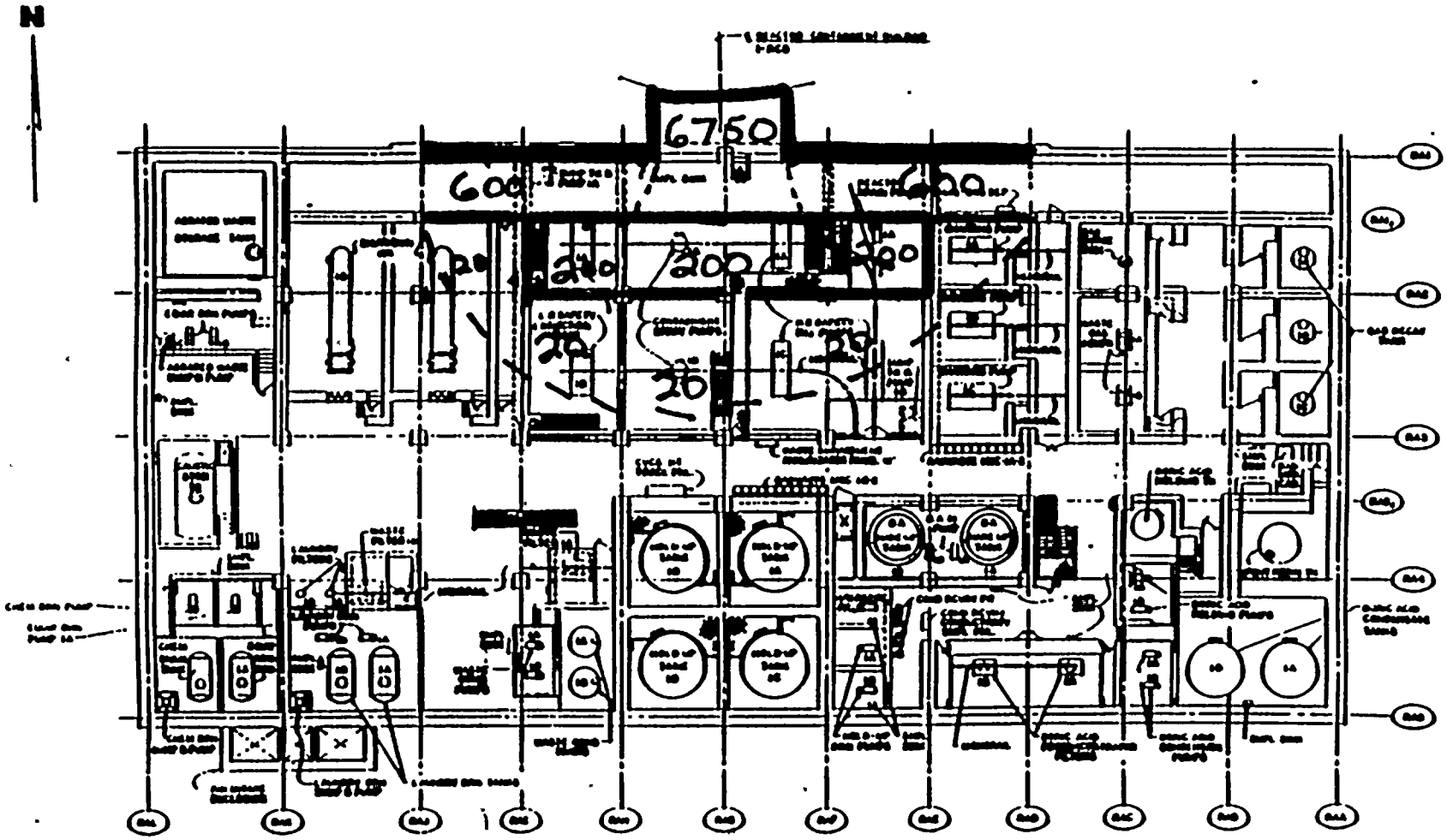
CPI

ST LUCIE - UNIT 1
INITIAL EQUIPMENT DATA BASE

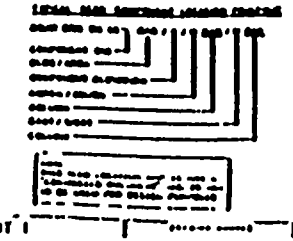
1 PAGE 2

TI 1200 – 1214

TIME: 1200 – 1214

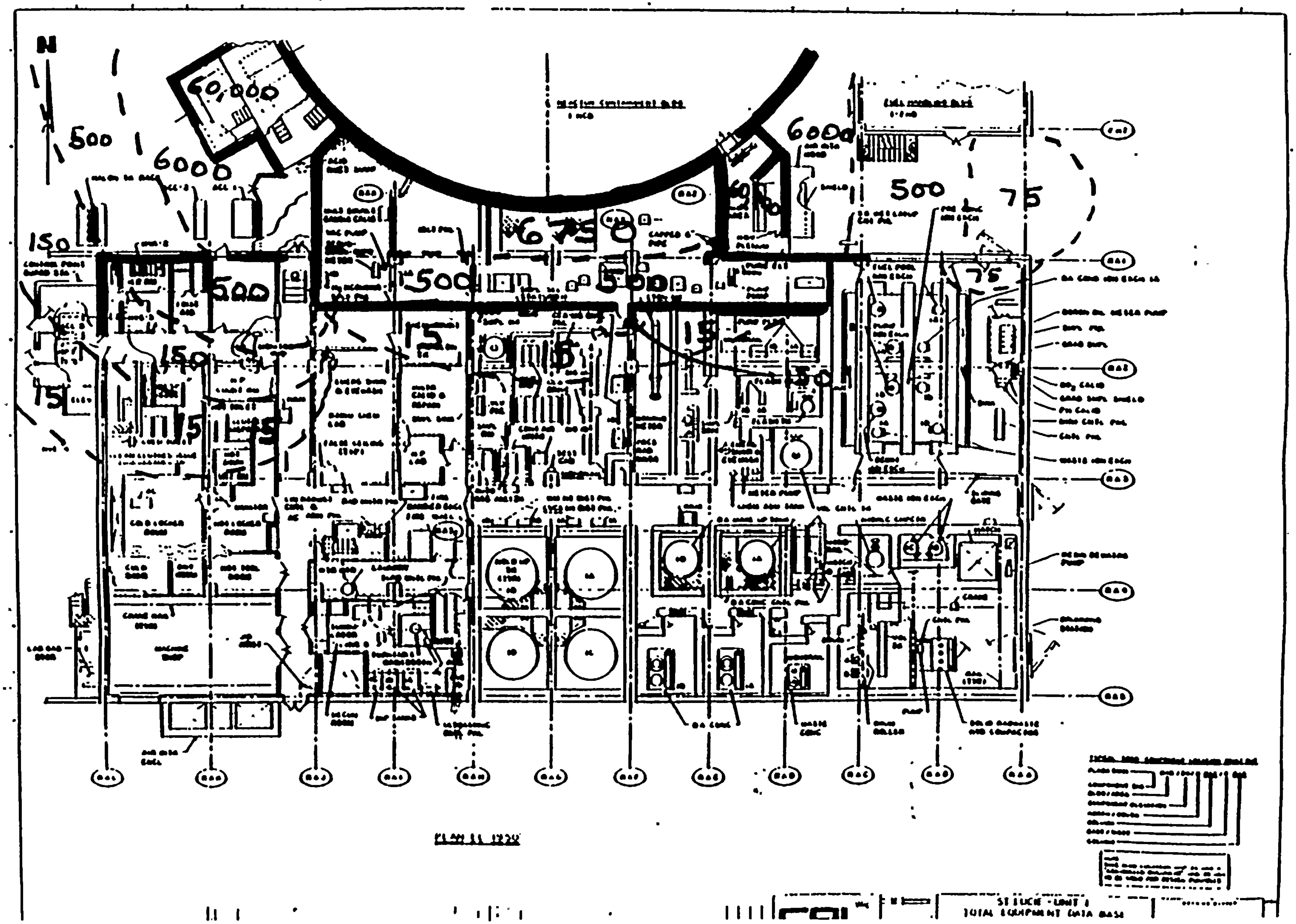


FLOOR PLANS EL 1050'

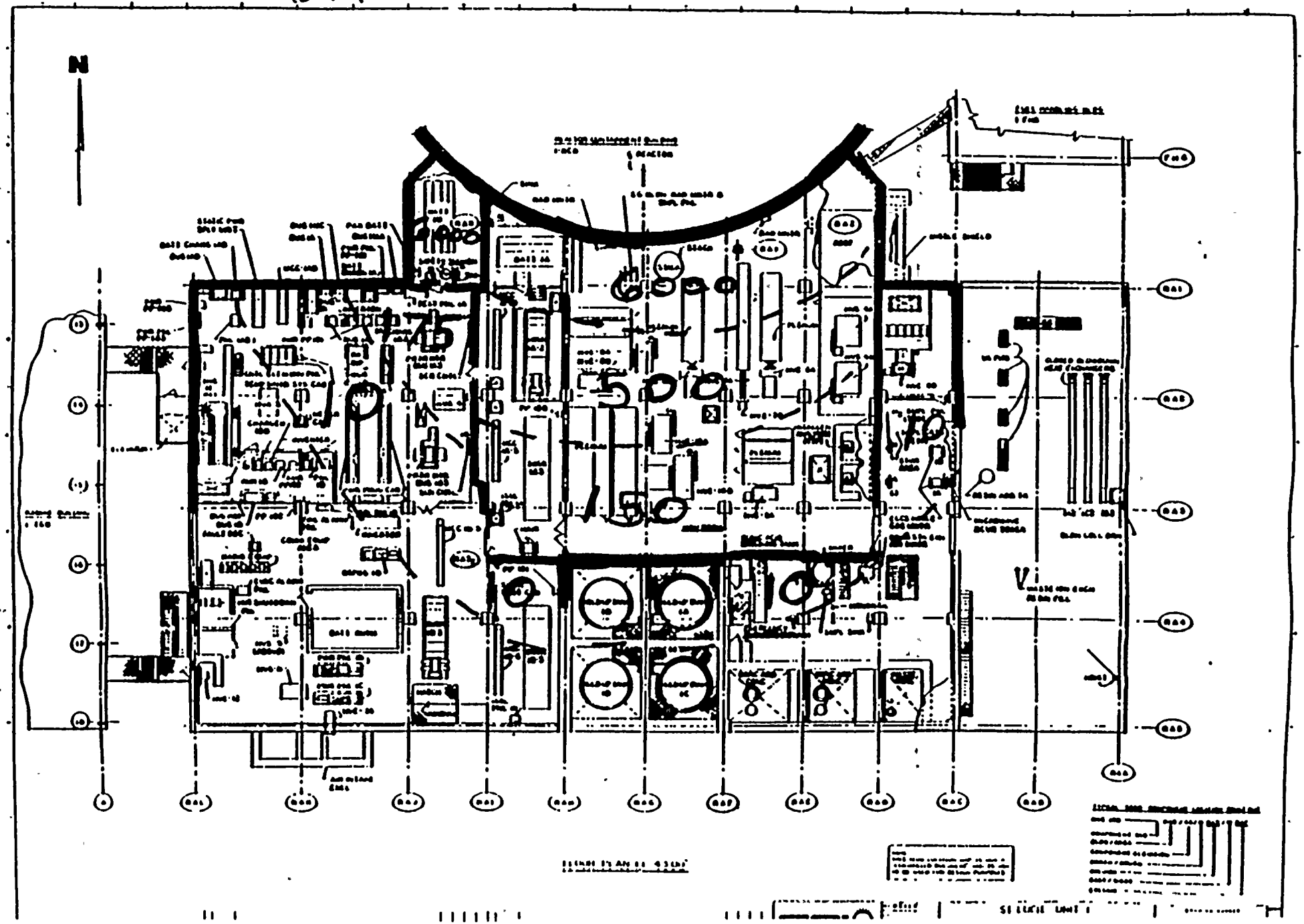


ST LUCIE UNIT 1

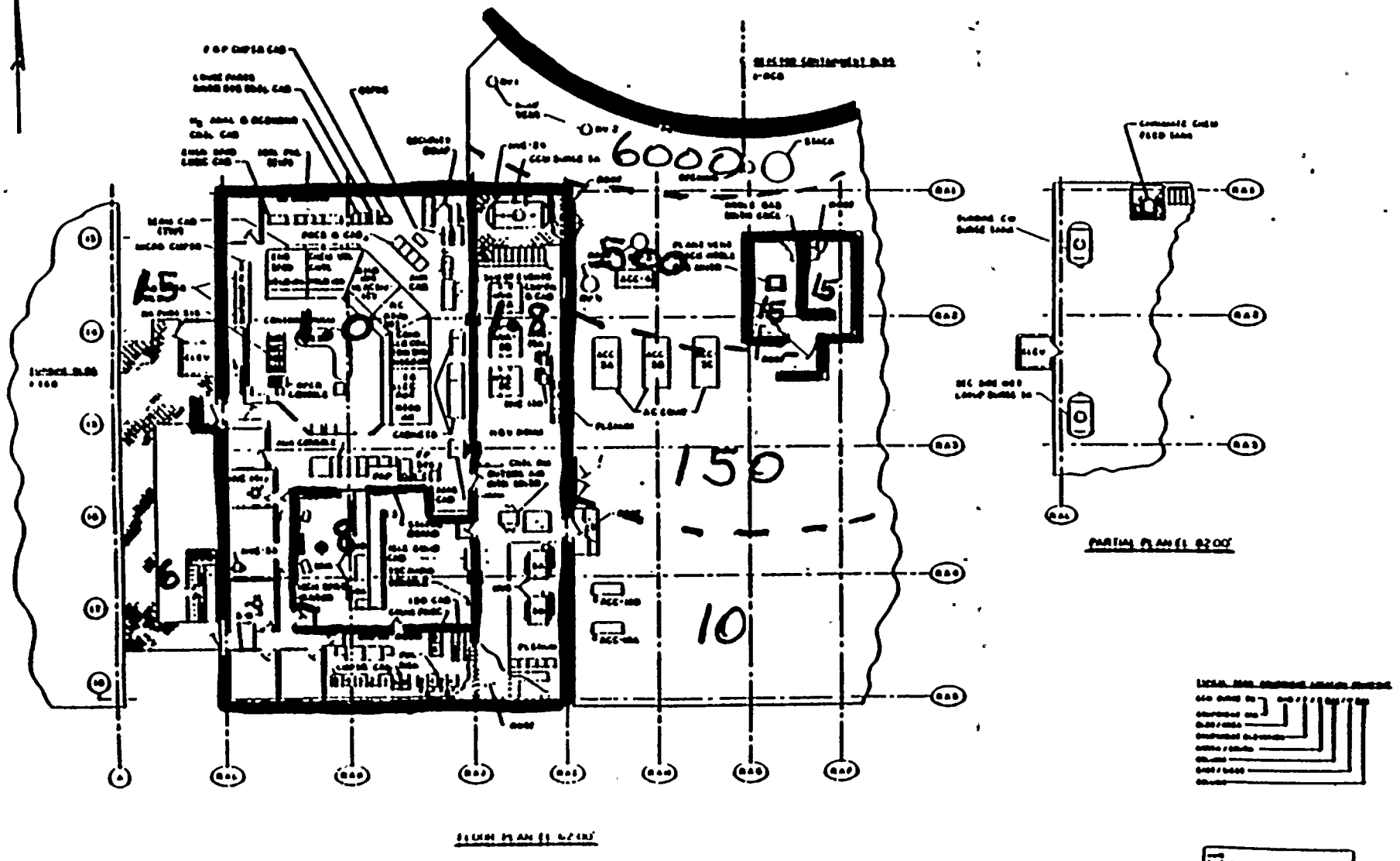
TIME: 1200-1214



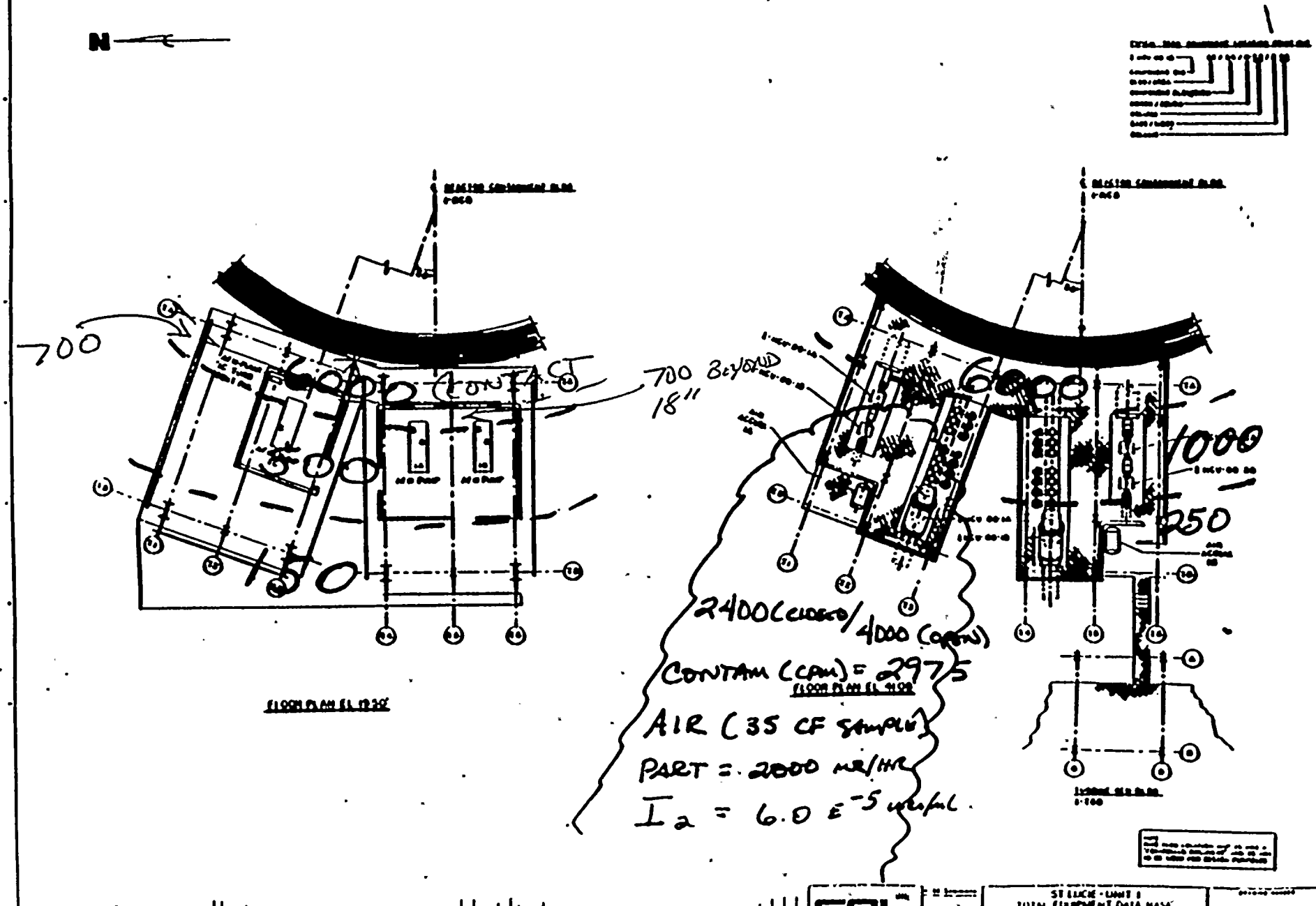
TIME: 1200 - 1214



TIME: 1200-1214

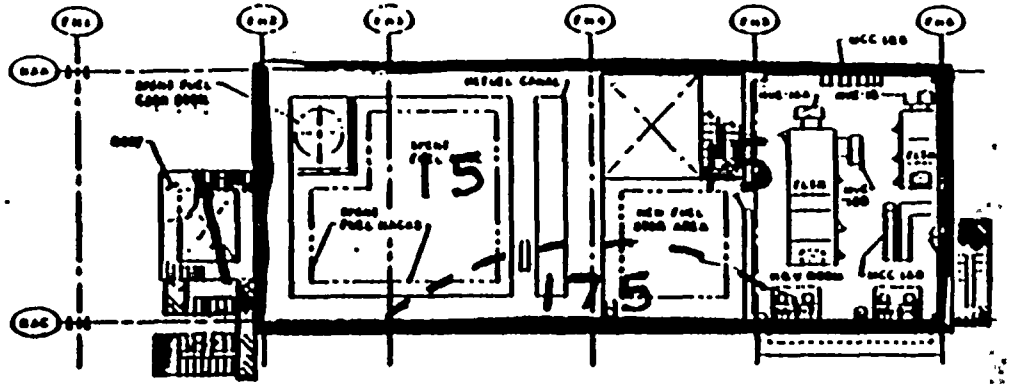


TIME: 1200-1214

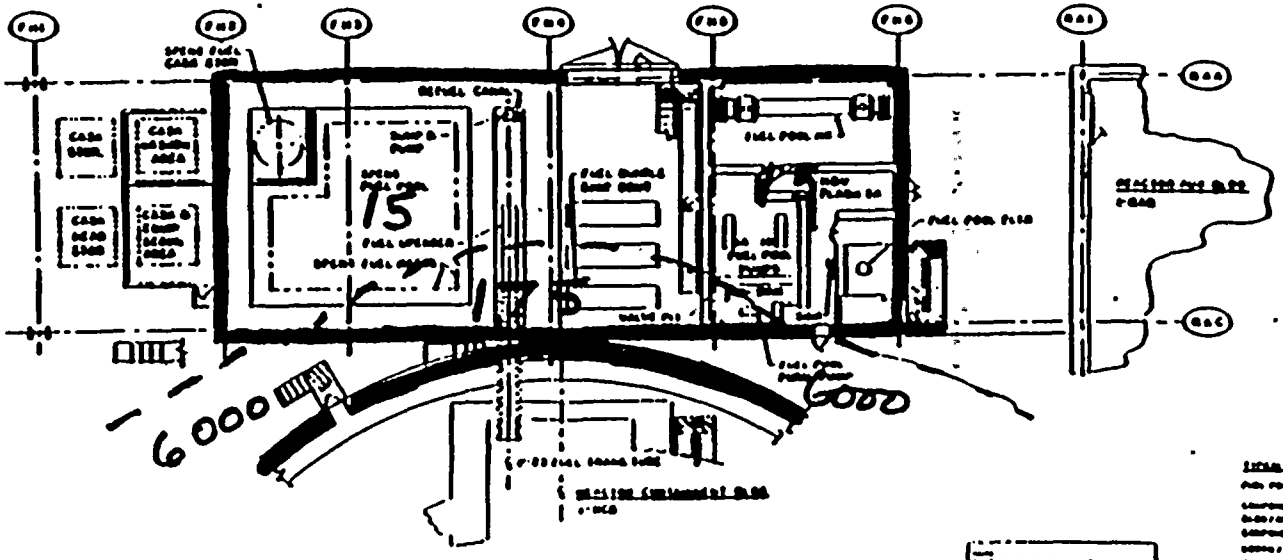


TIME: 1200-1214

N



FLOOR PLAN EL 1900'



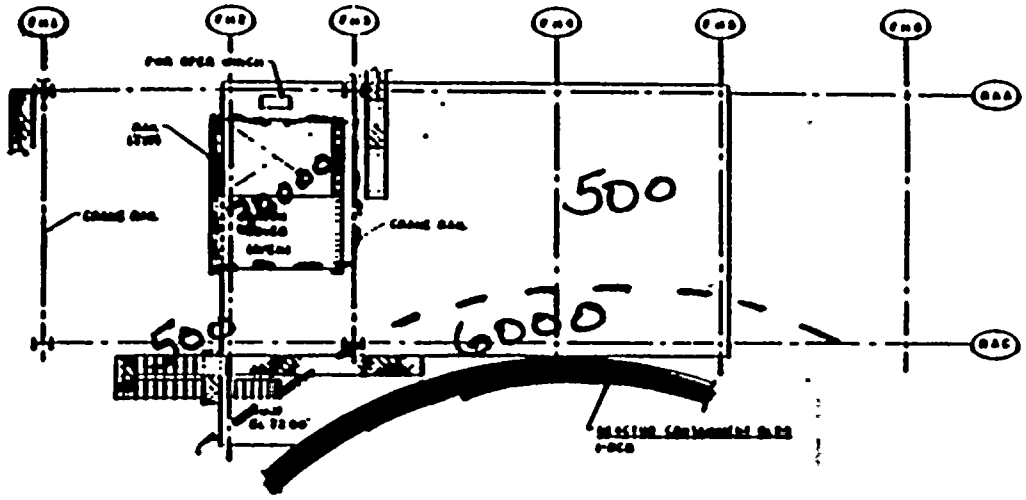
FLOOR PLAN EL 1950'

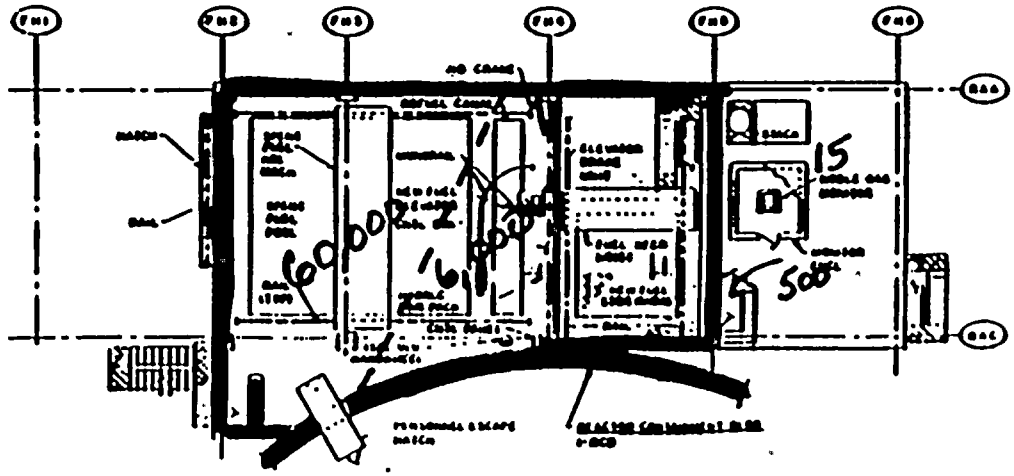
FPL

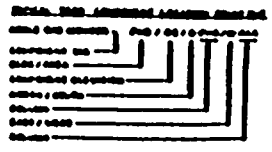
ST LUCIE UNIT 1
TOTAL EQUIPMENT DATA BASE
LOCATION MAP

TIME: 1200-1214

N



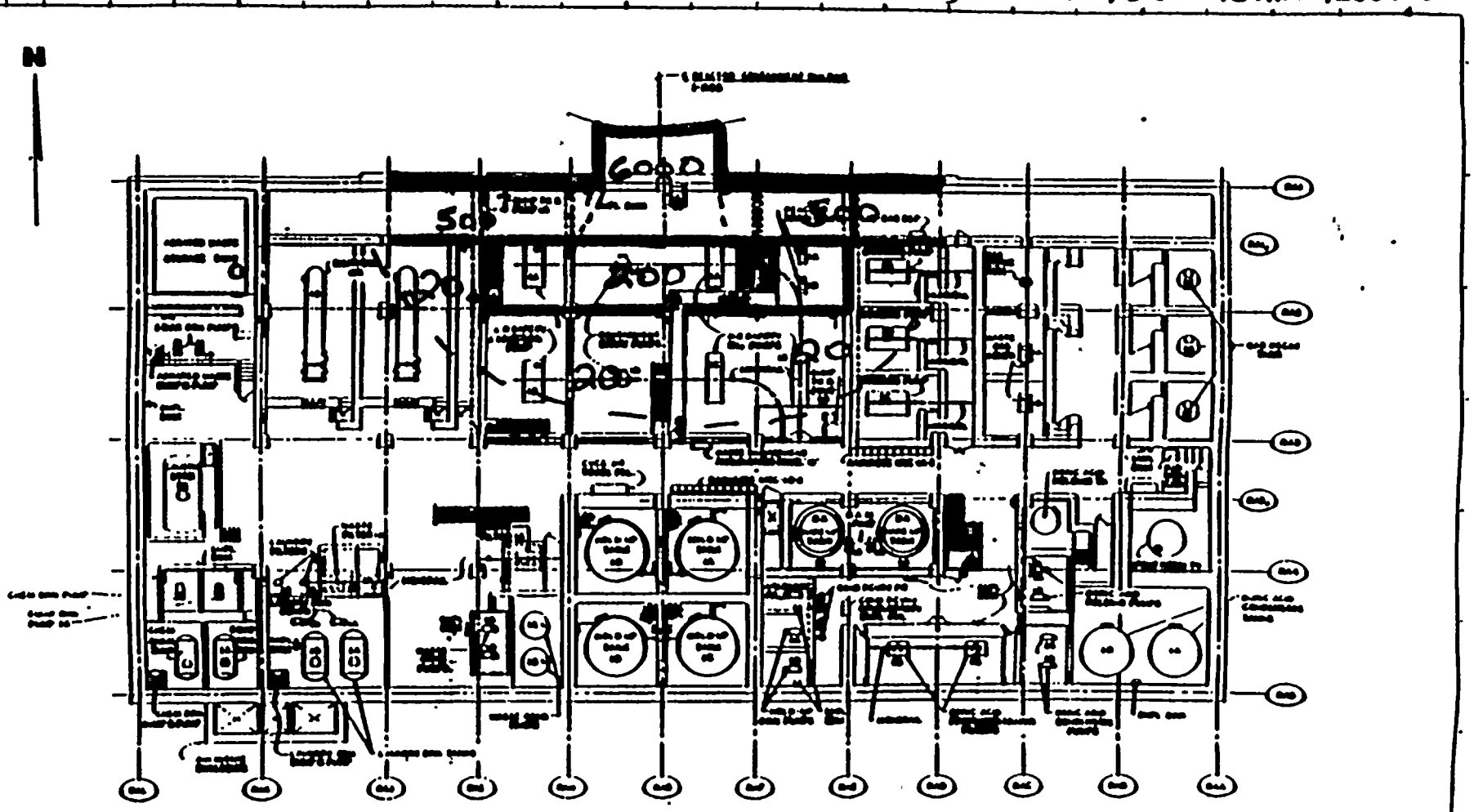




ST LUCIE - UNIT 1

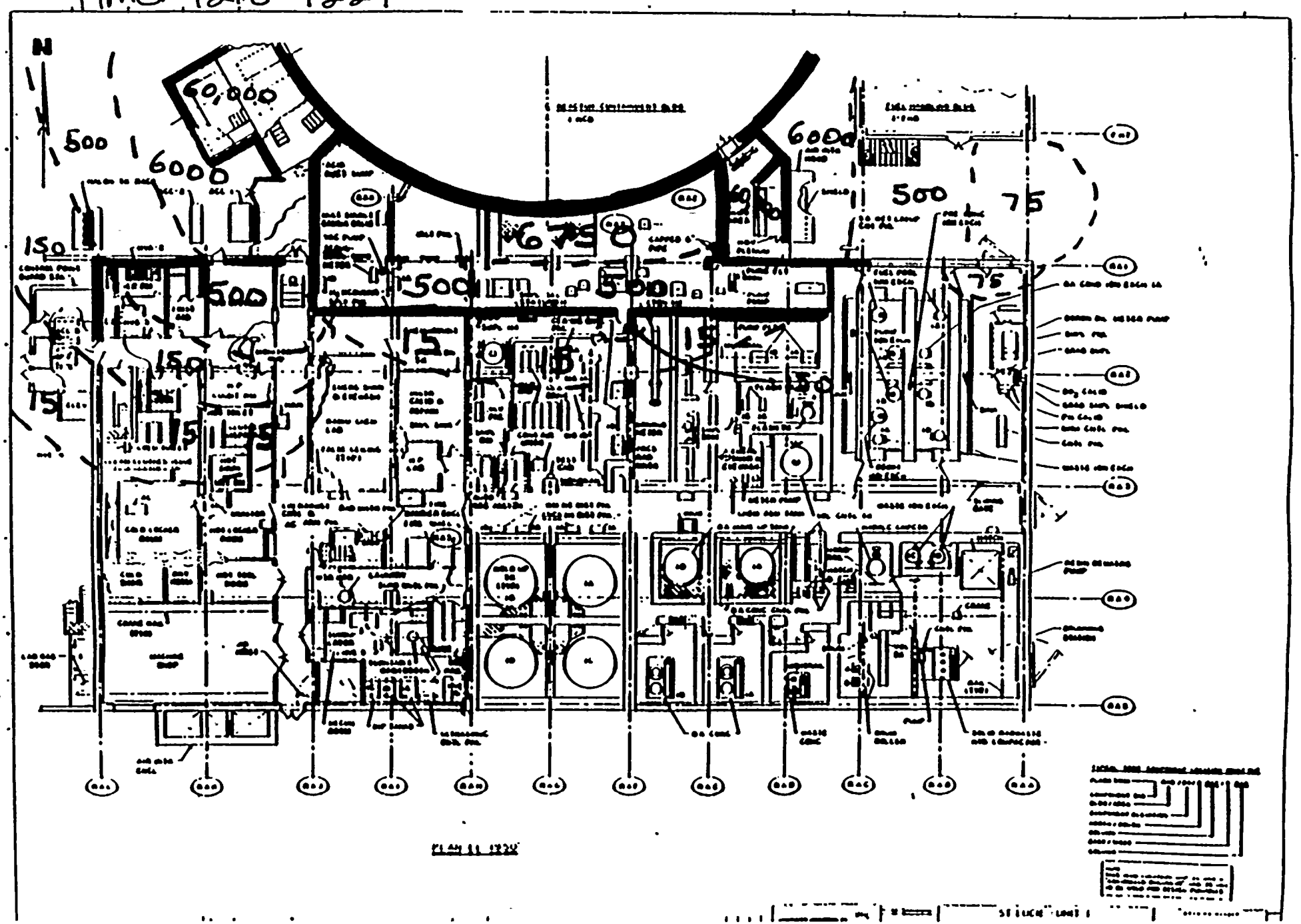
TIME: 1215 - 1229

ALL SHEETS
- DOSE RATES IN MR/HR B & (OPEN WINDOW = CLOSED)
  * EXCEPT FOR STEAM TRESTLE
- CONTAMINATION (CPM) = ACTUAL, EXCEPT STEAM TRESTLE
- AIR ACTIVITY = ACTUAL, EXCEPT FOR STEAM TRESTLE

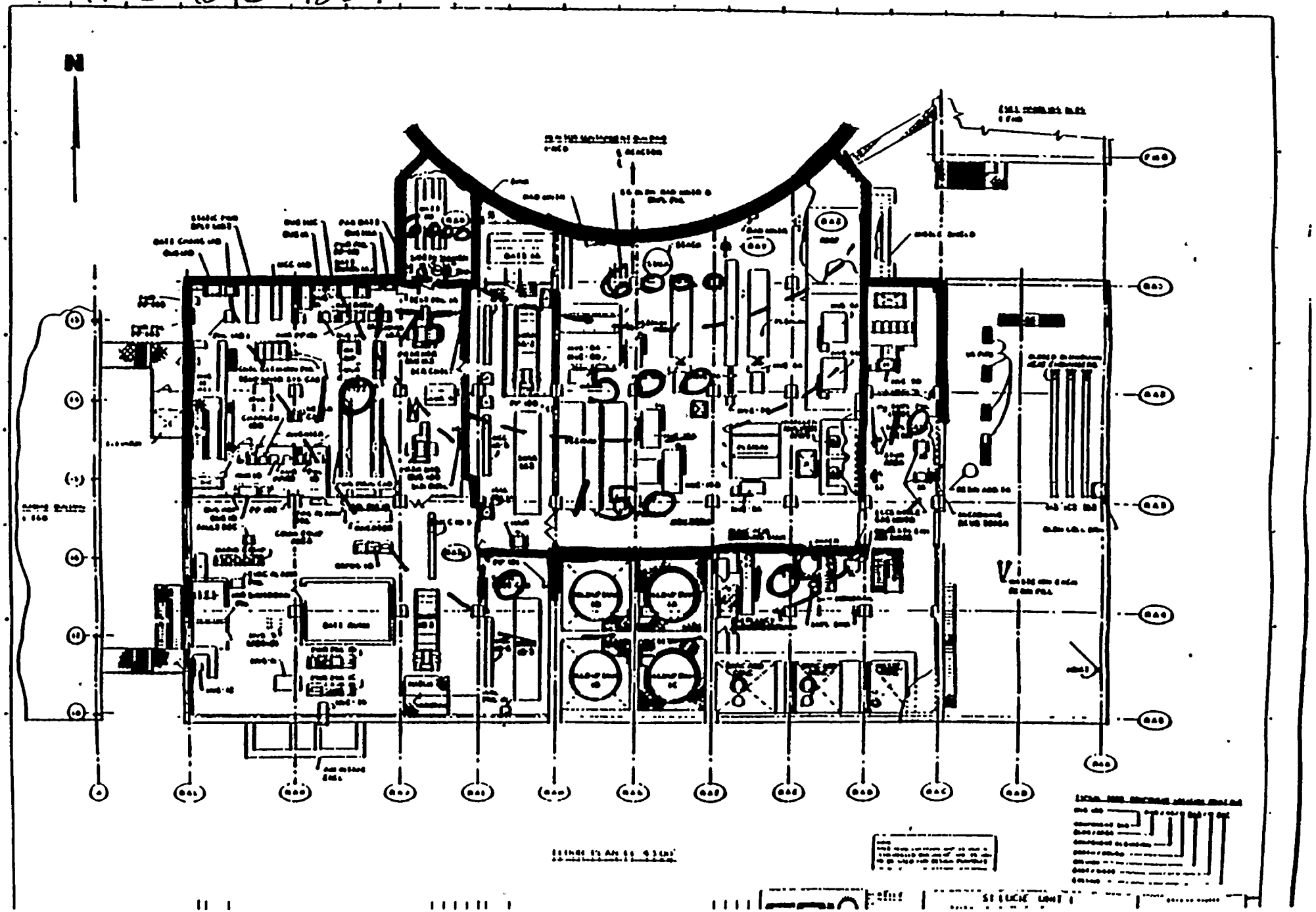


FLOOR PLAN 11-1939

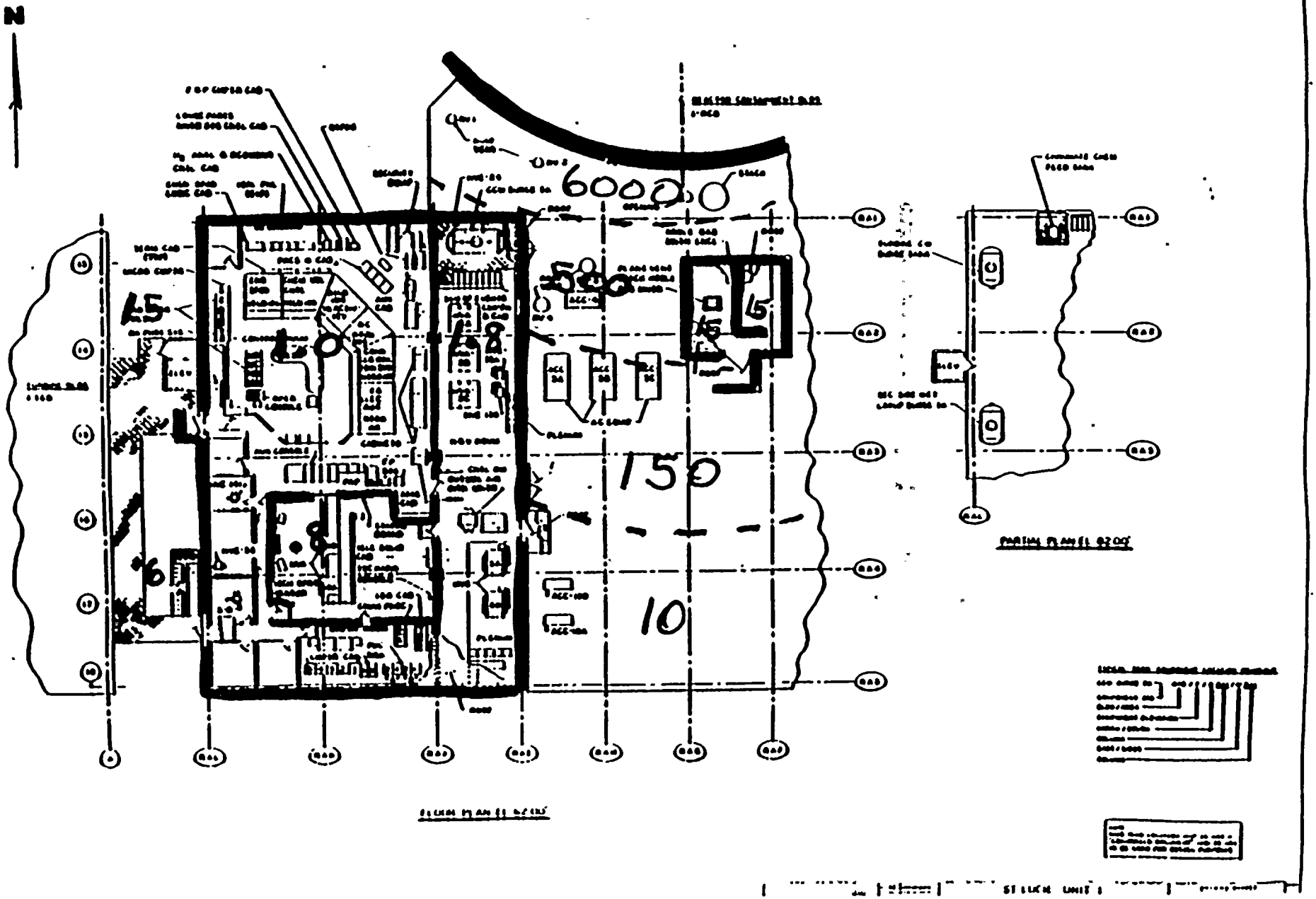
TIME: 1215 - 1229



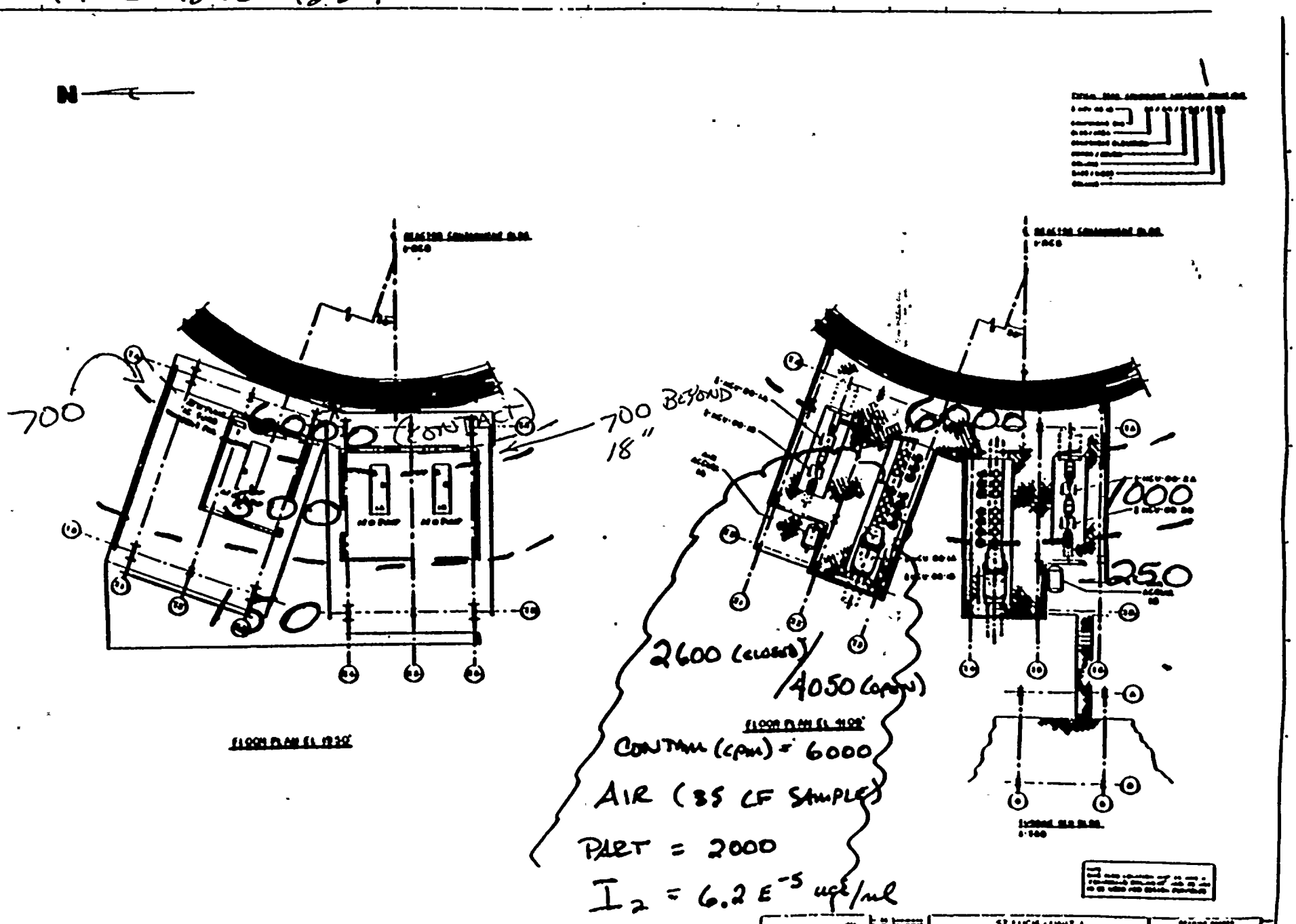
TIME: 1215 - 1229



TIME: 1215 - 1229

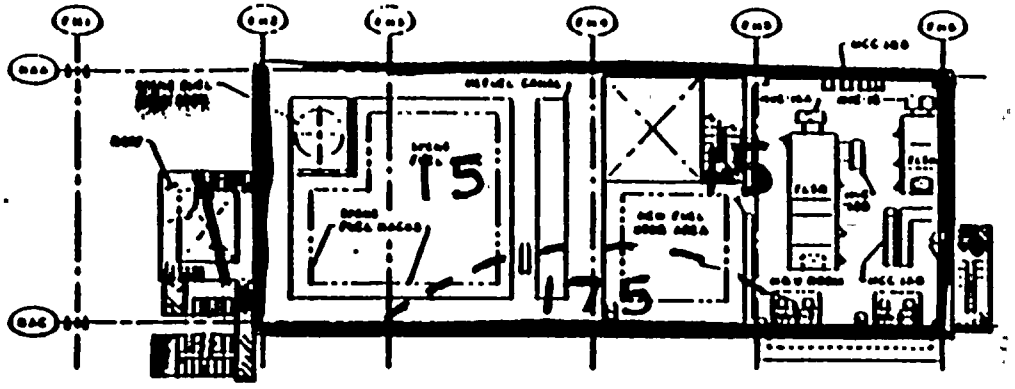


TIME: 1215-1229

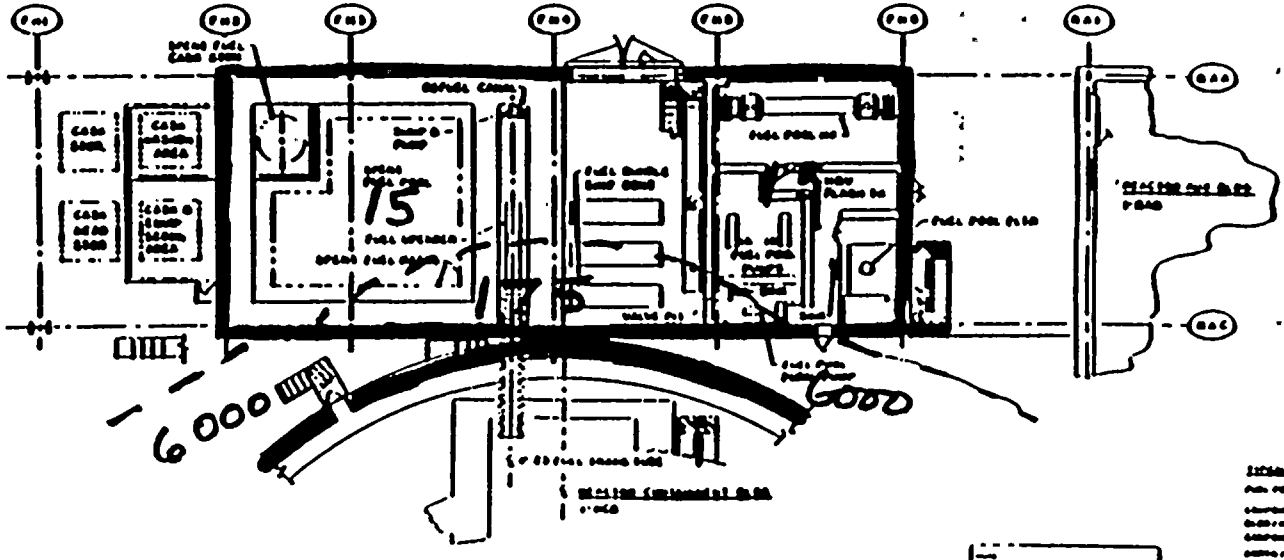


TIME: 1215-1229

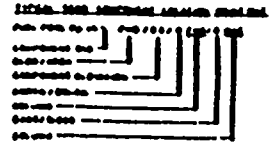
N

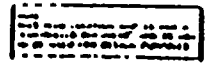


FLOOR PLAN EL 1800'



FLOOR PLAN EL 1930'



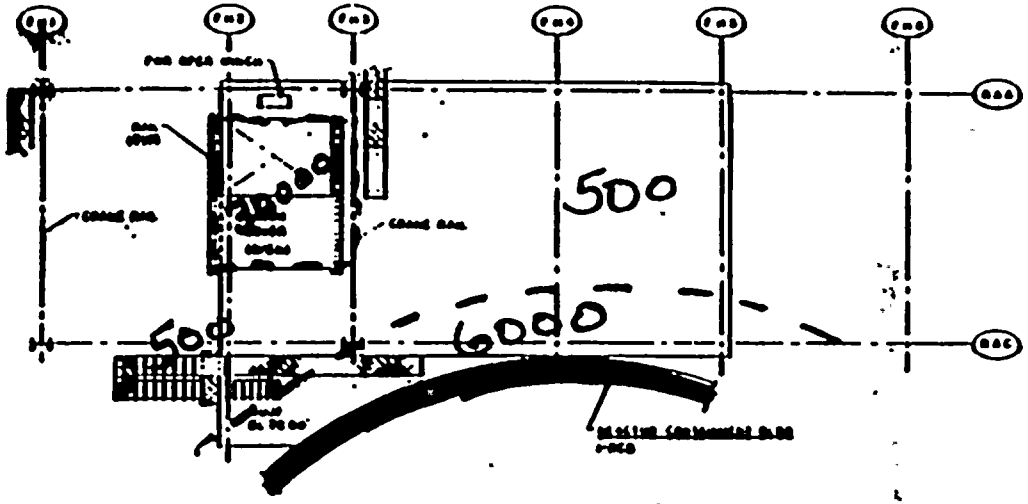


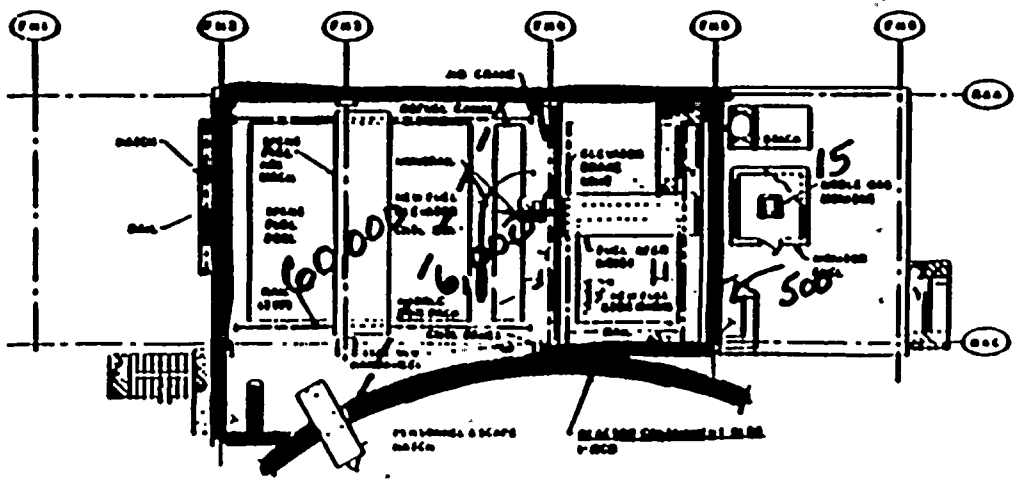
CDI

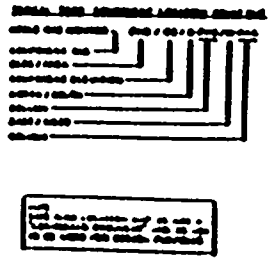
SEABROOK UNIT 1
INITIAL EQUIPMENT DATA BASE

TIME: 1215-1229

N

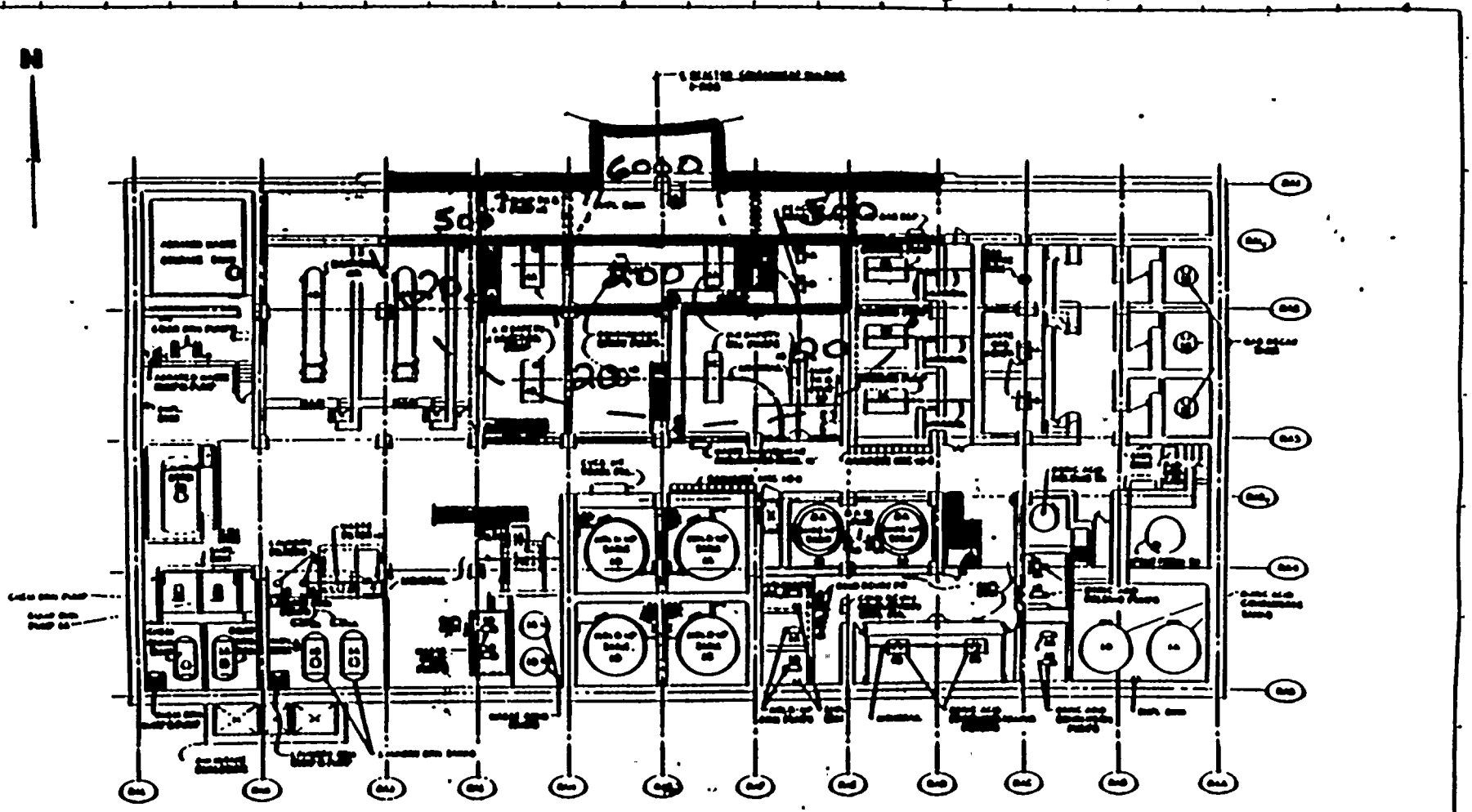




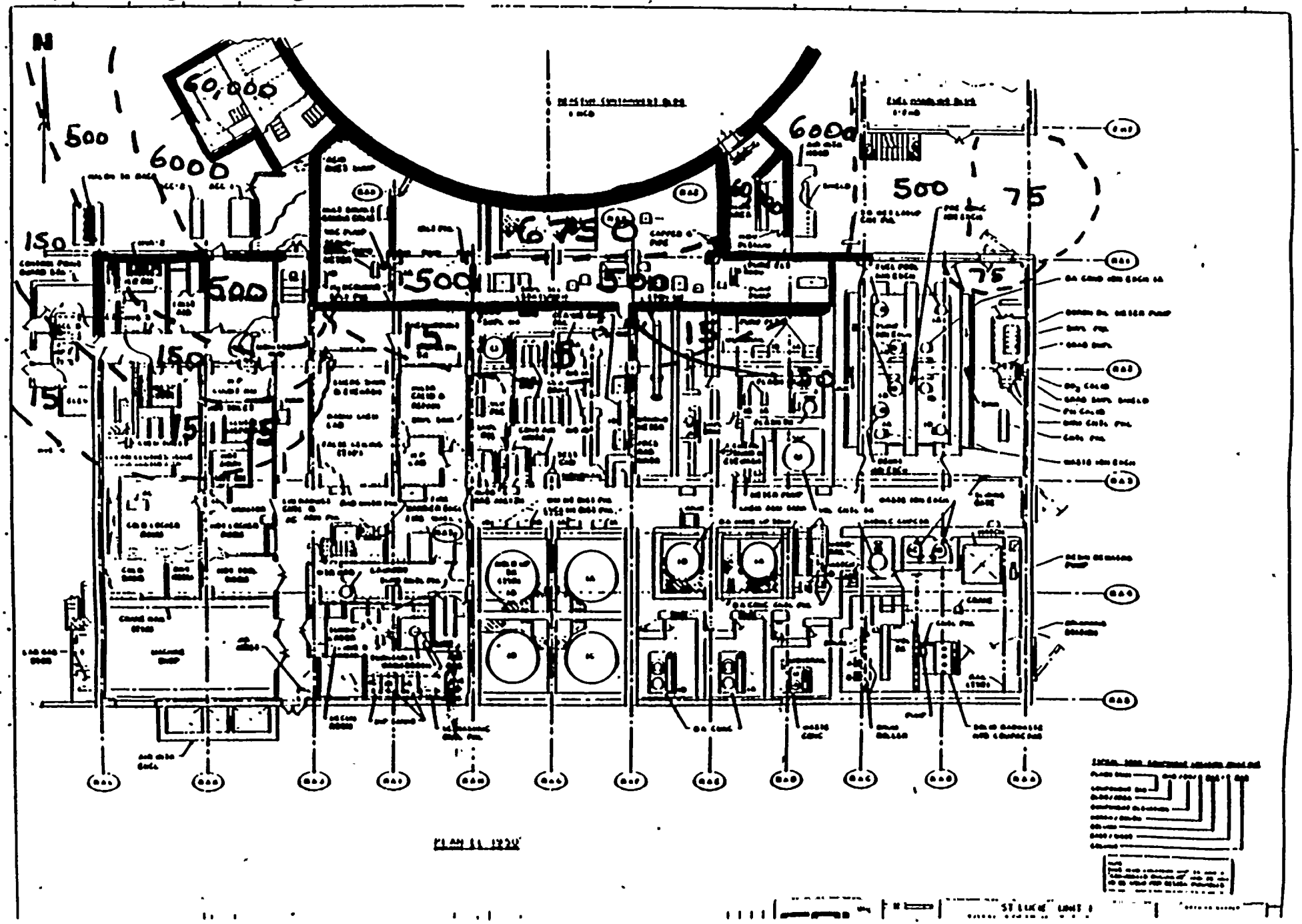


TIME: 1230 - 1244

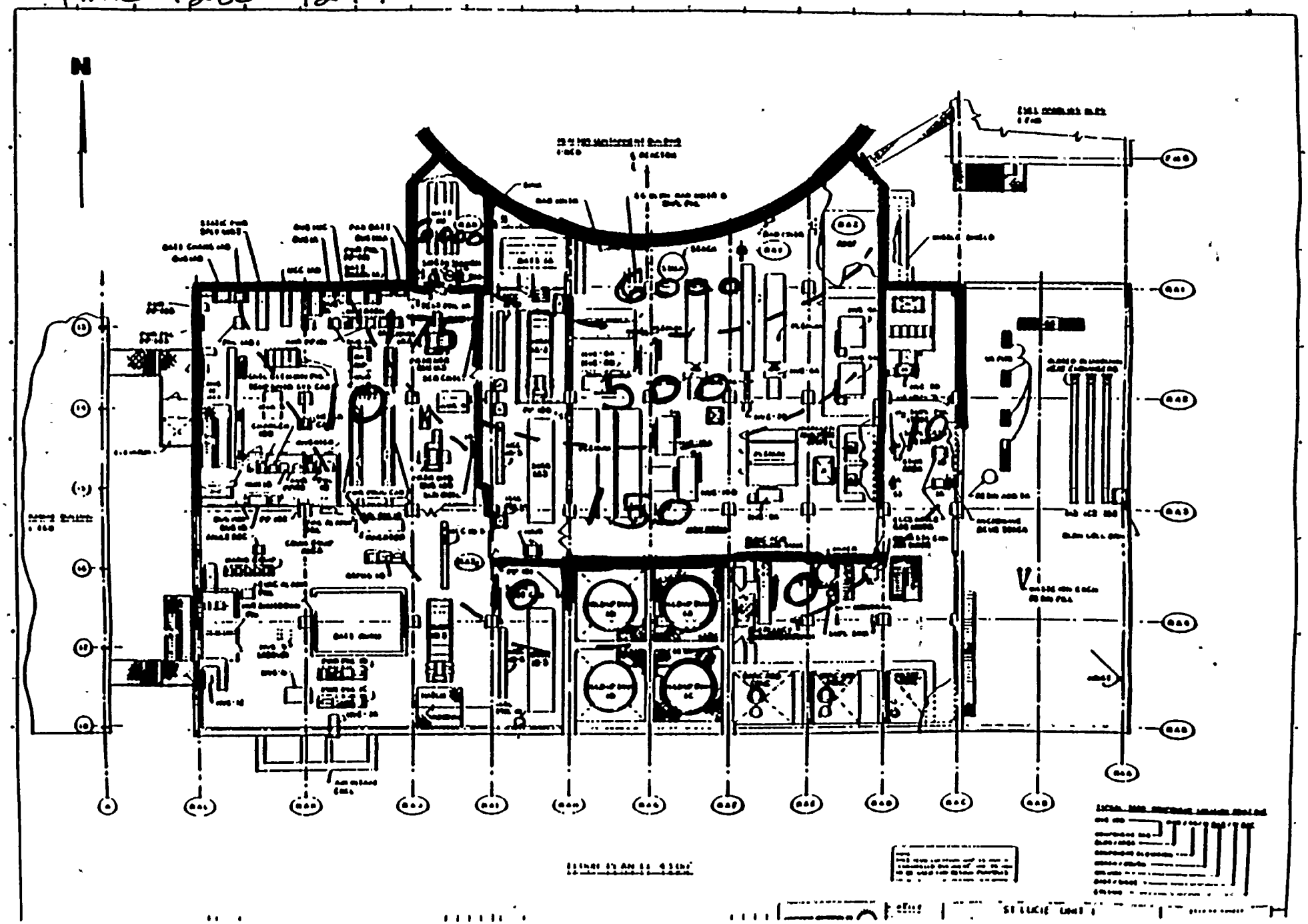
ALL SHEETS
- DOSE RATES IN MR/HR B & (OPEN WINDOW = CLOSED)
  * EXCEPT FOR STEAM TRESTLE
- CONTAMINATION (CPM) = ACTUAL, EXCEPT STEAM TRESTLE
- AIR ACTIVITY = ACTUAL, EXCEPT FOR STEAM TRESTLE



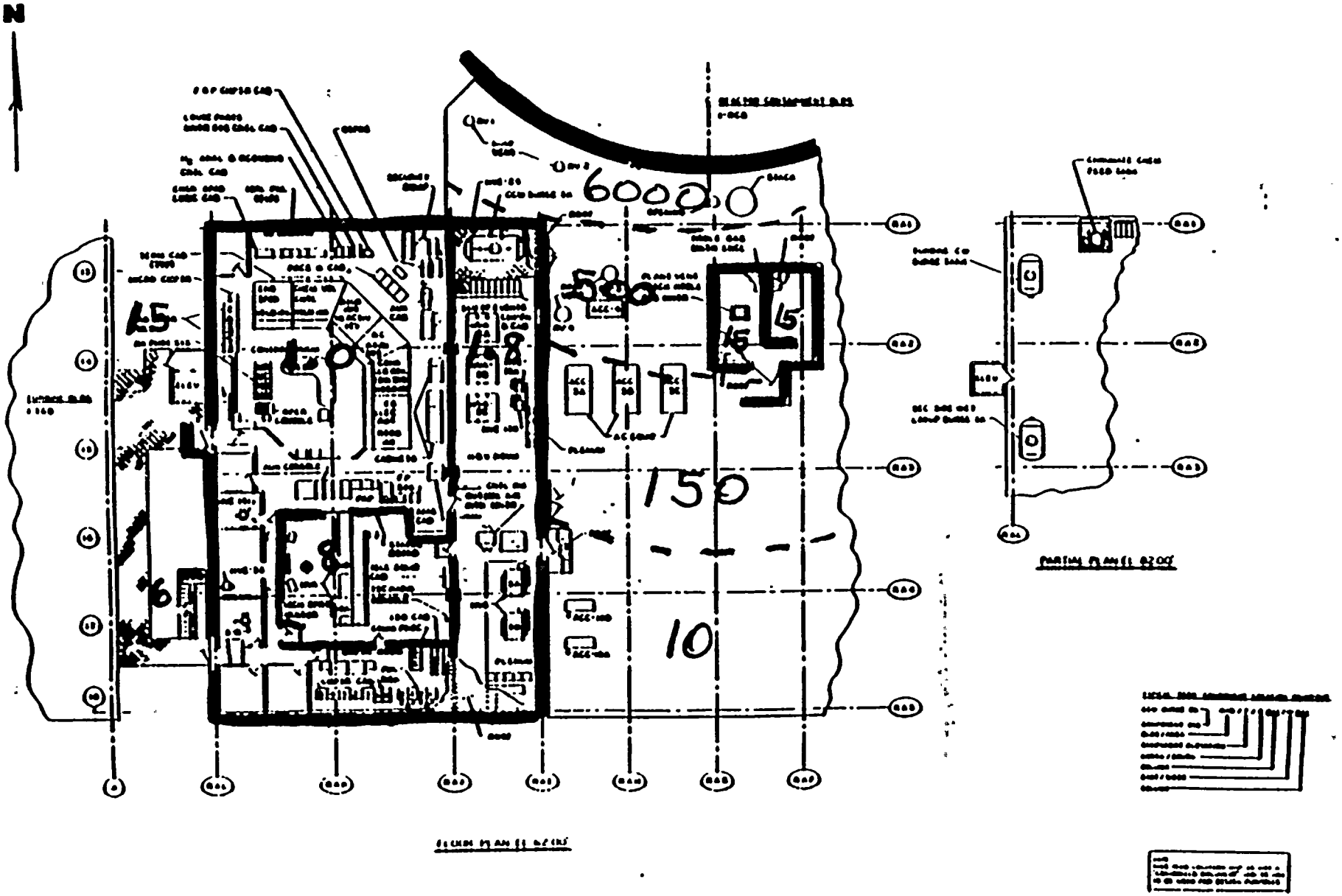
TIME: 1230 - 1244



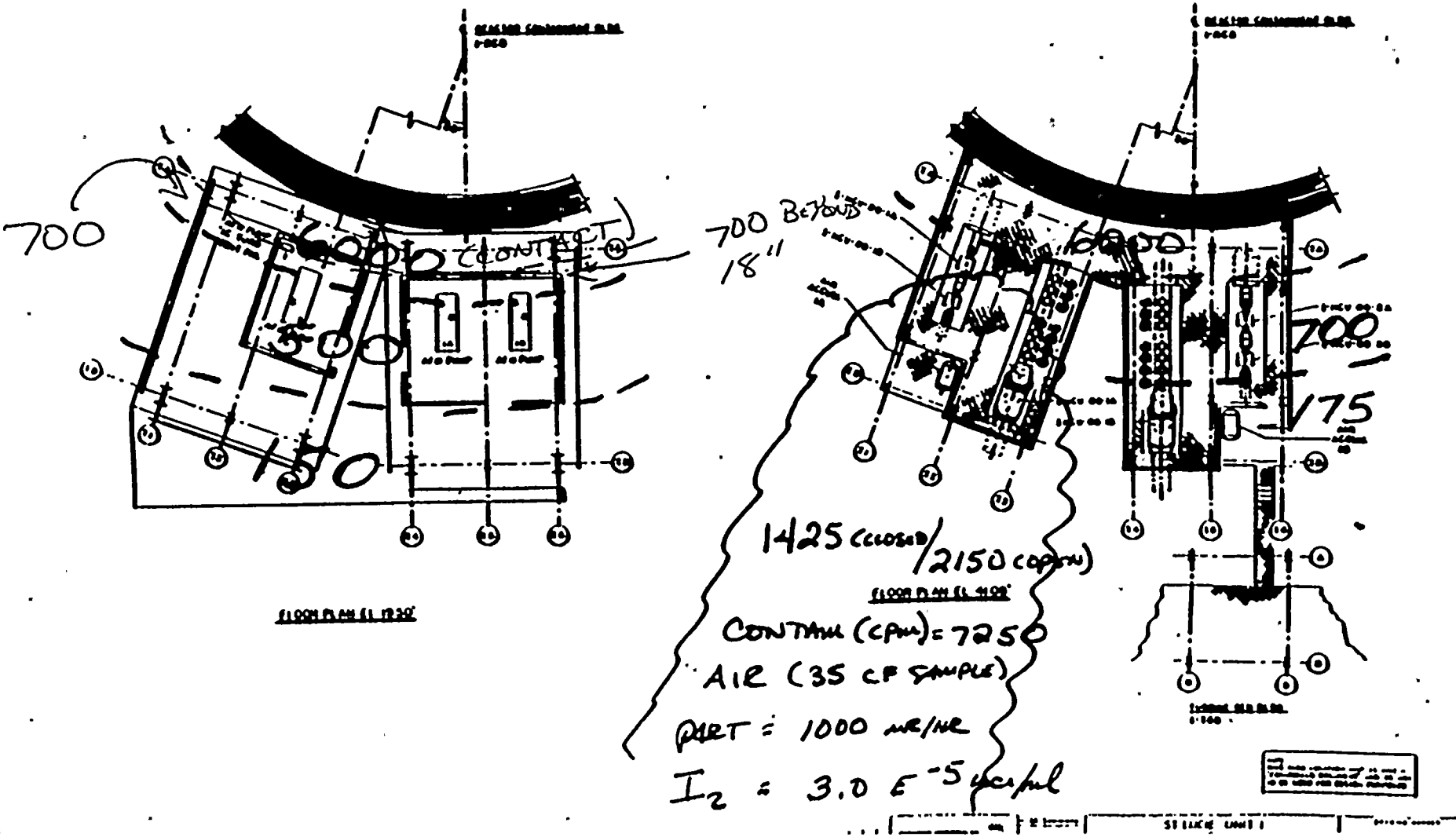
TIME: 1230 - 1244



TIME: 1230 - 1244

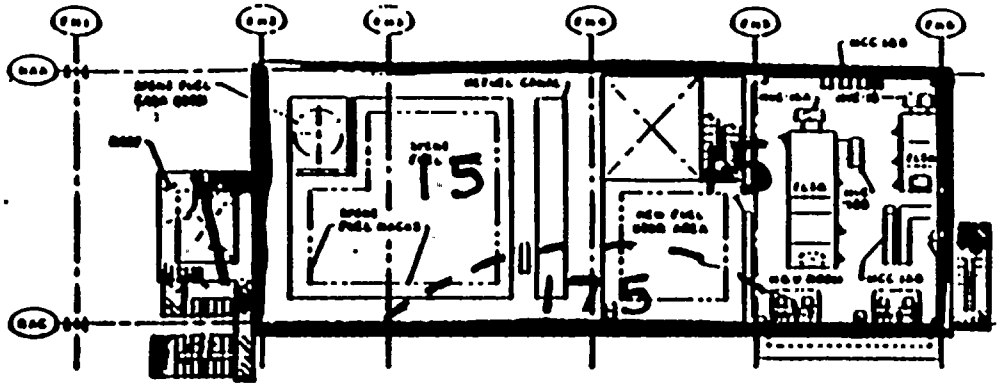


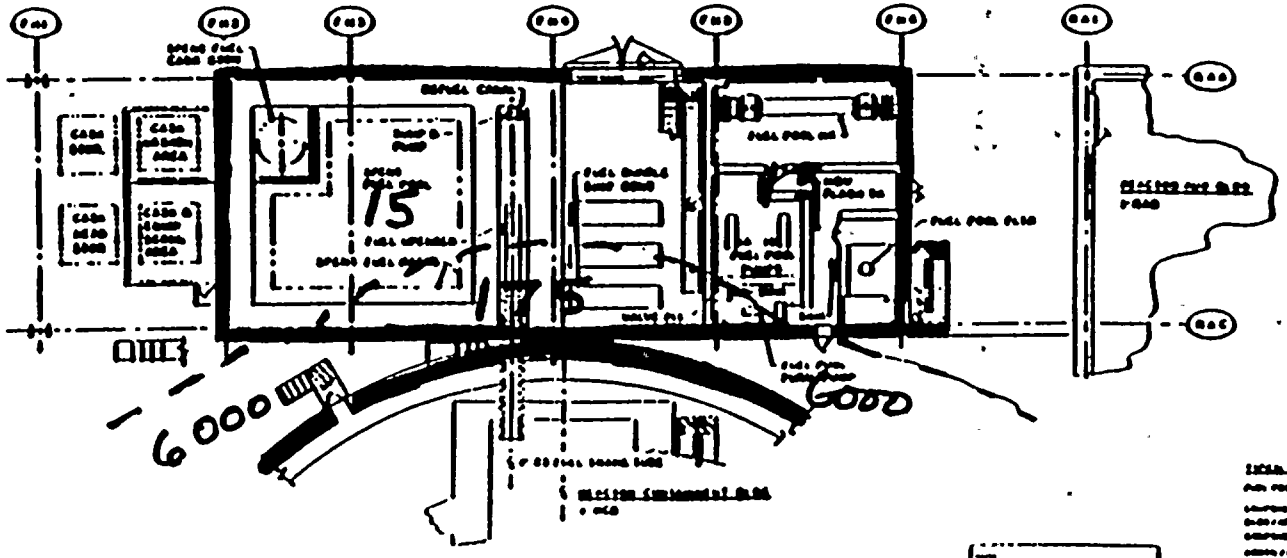
TIME: 1230-1244



TIME: 1230 - 1244

N



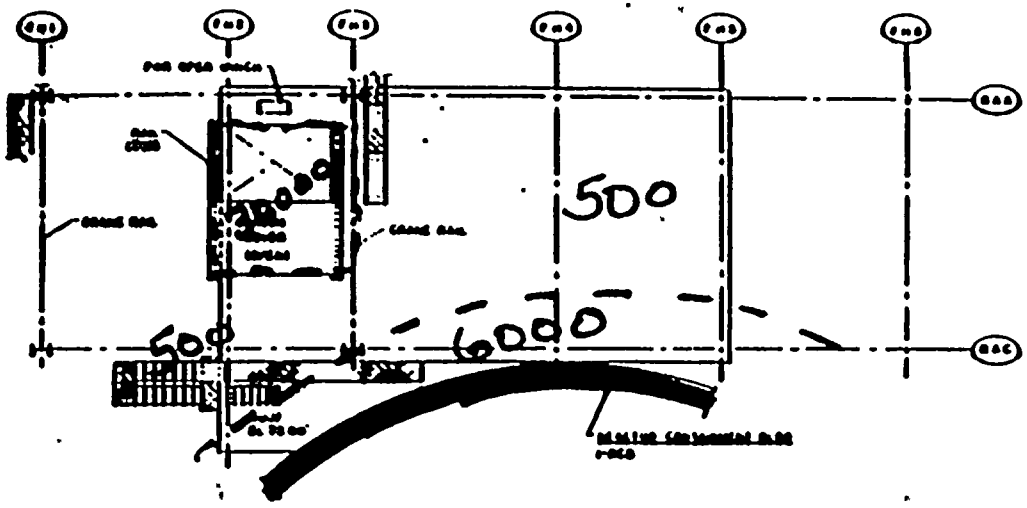


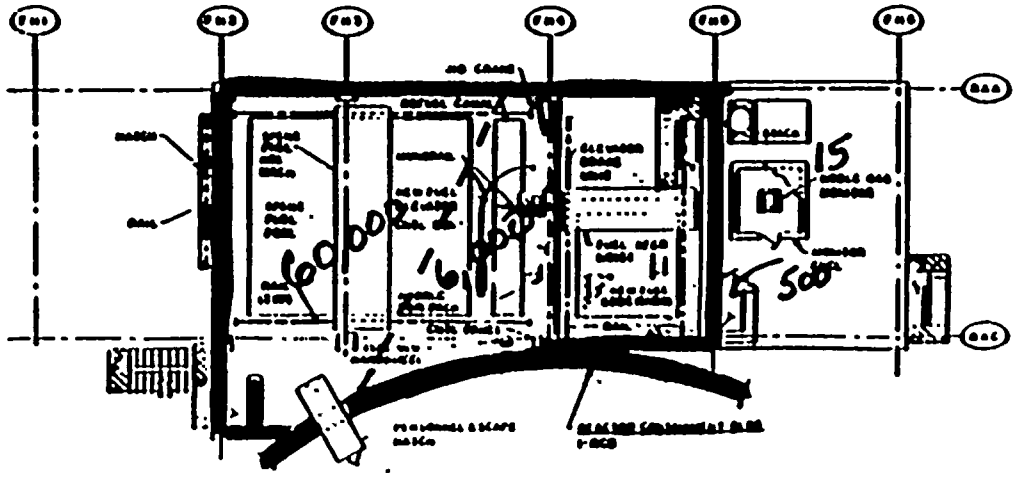
FPL

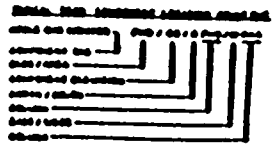
ST LUCIE UNIT 1
INITIAL EQUIPMENT DATA BASE
LOCATION MAP

TIME: 1230 - 1244

N

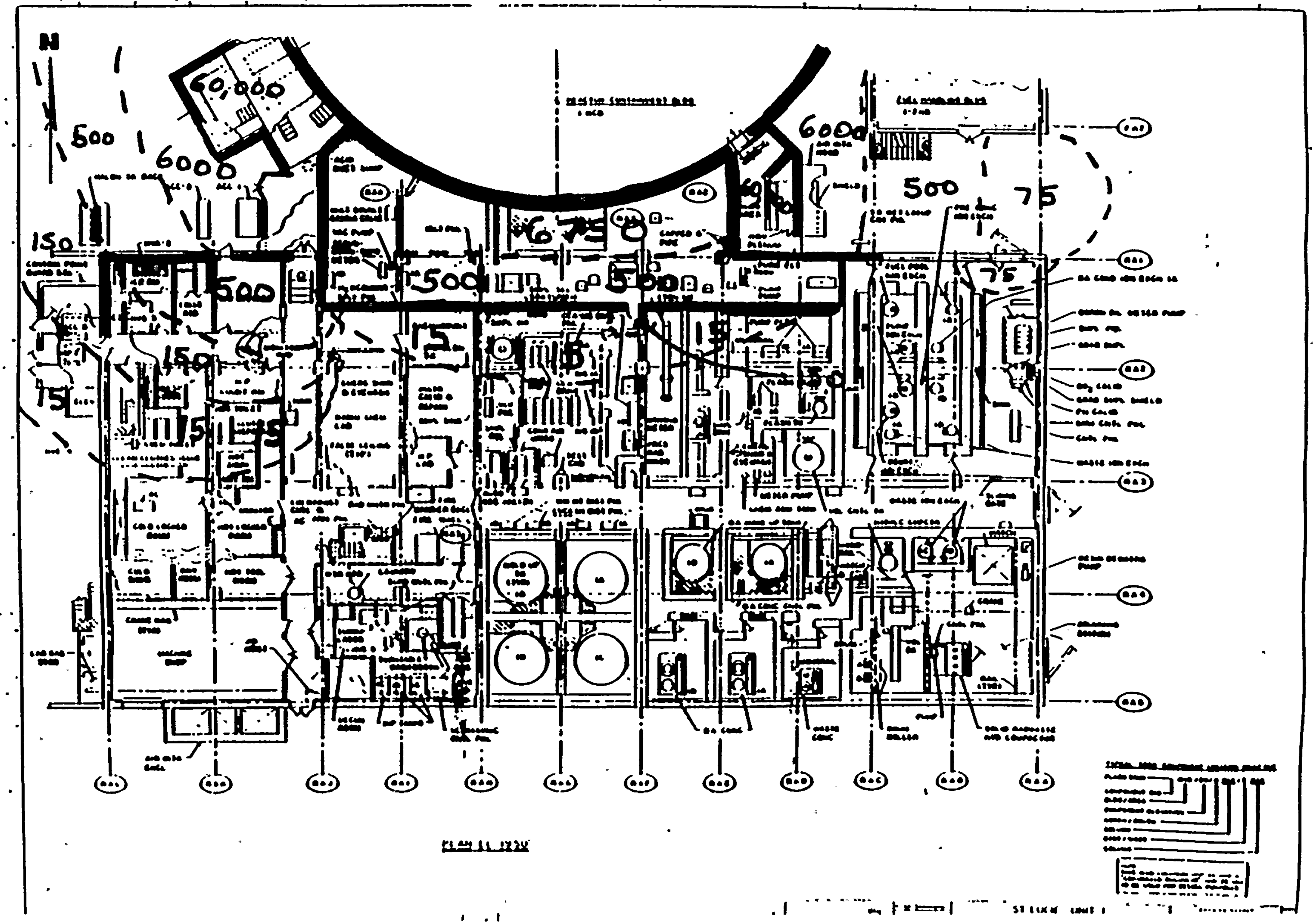




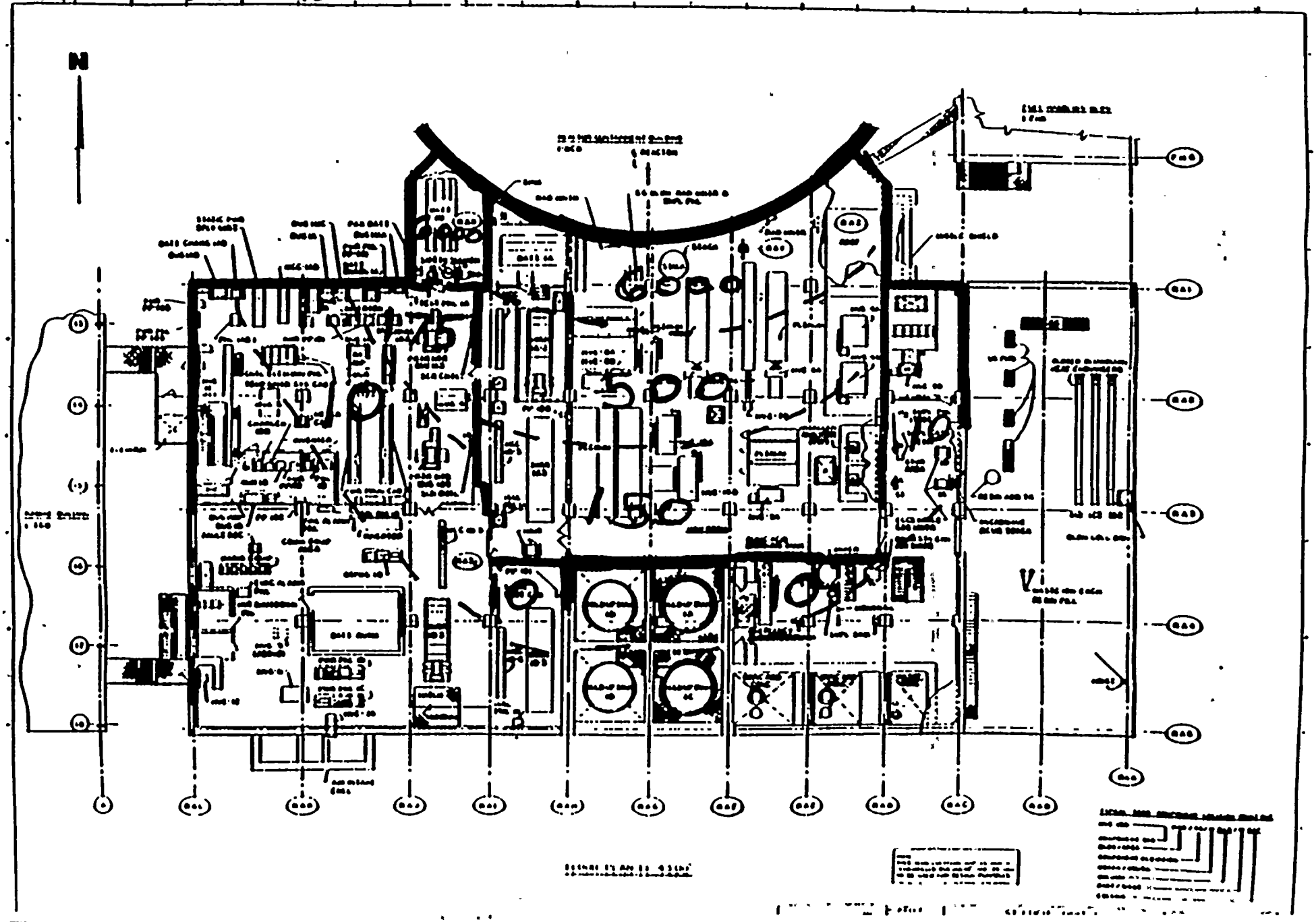


ST LUCIE - UNIT 1
TOTAL EQUIPMENT DATA BASE

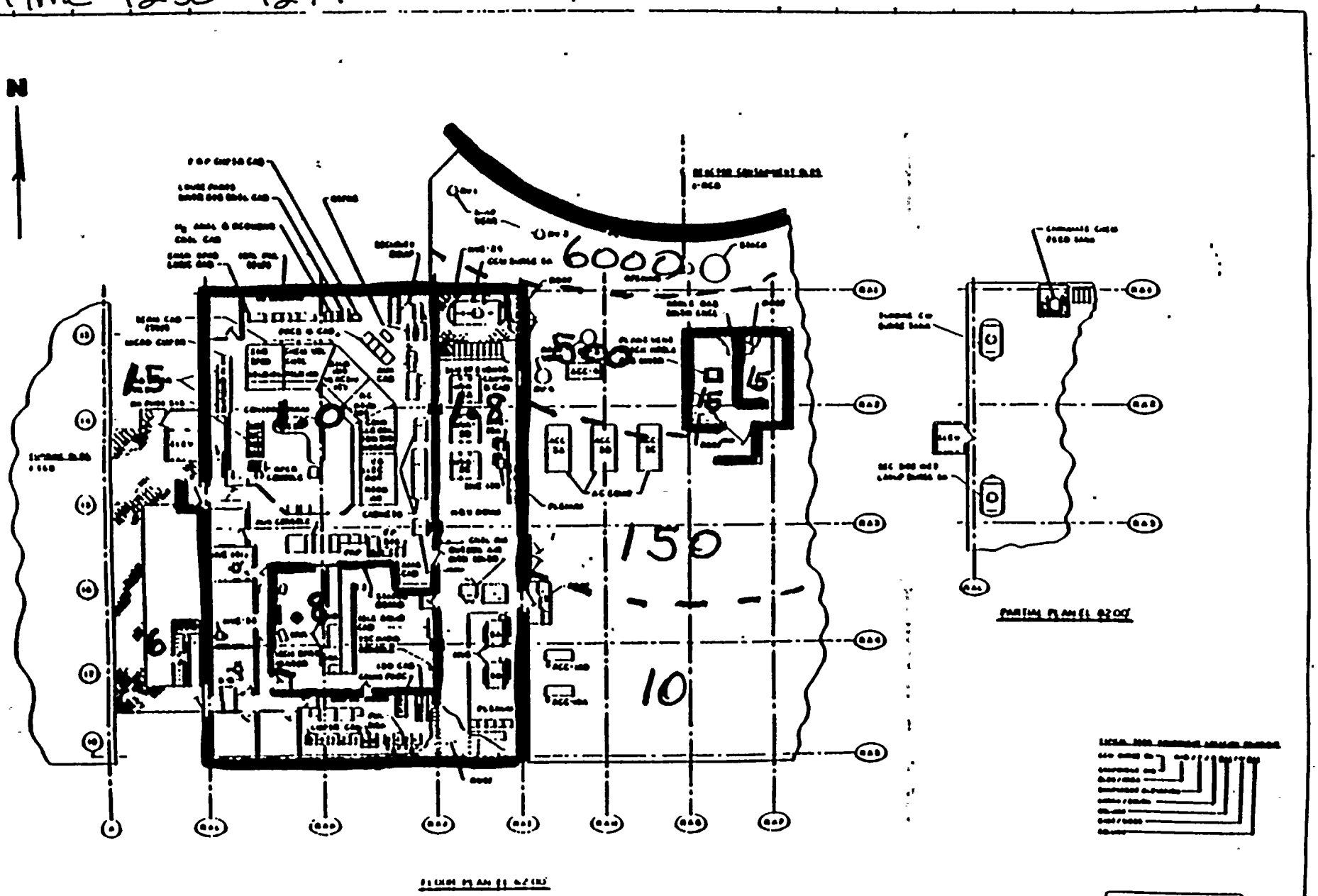
TIME: 1230-1244



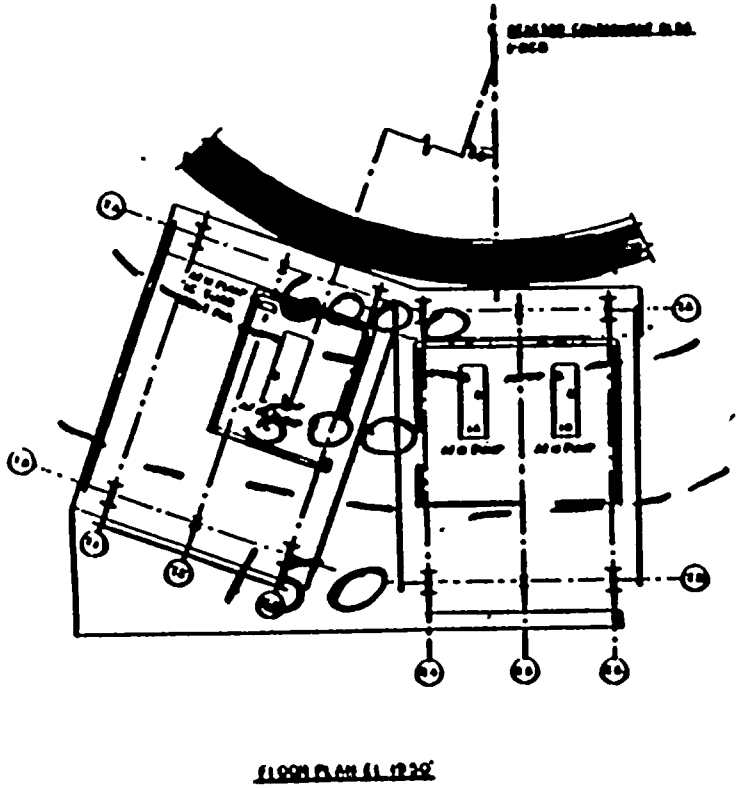
TIME: 1230 - 1244

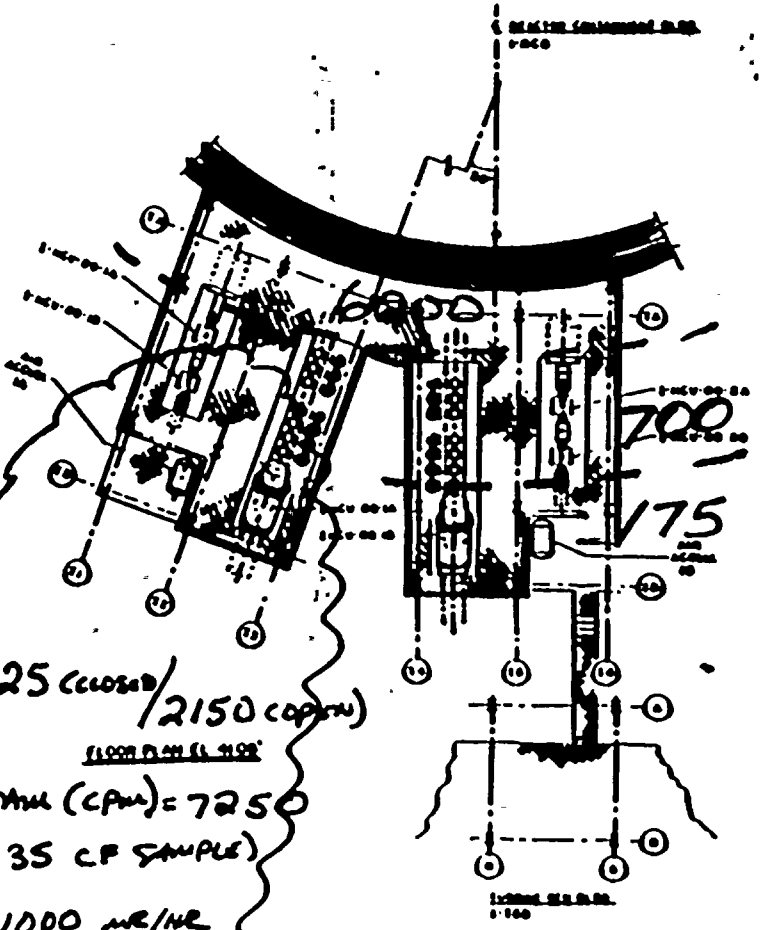


TIME: 1230 - 1244



TIME: 1230-1244





1425 (CLOSED)/2150 (OPEN)

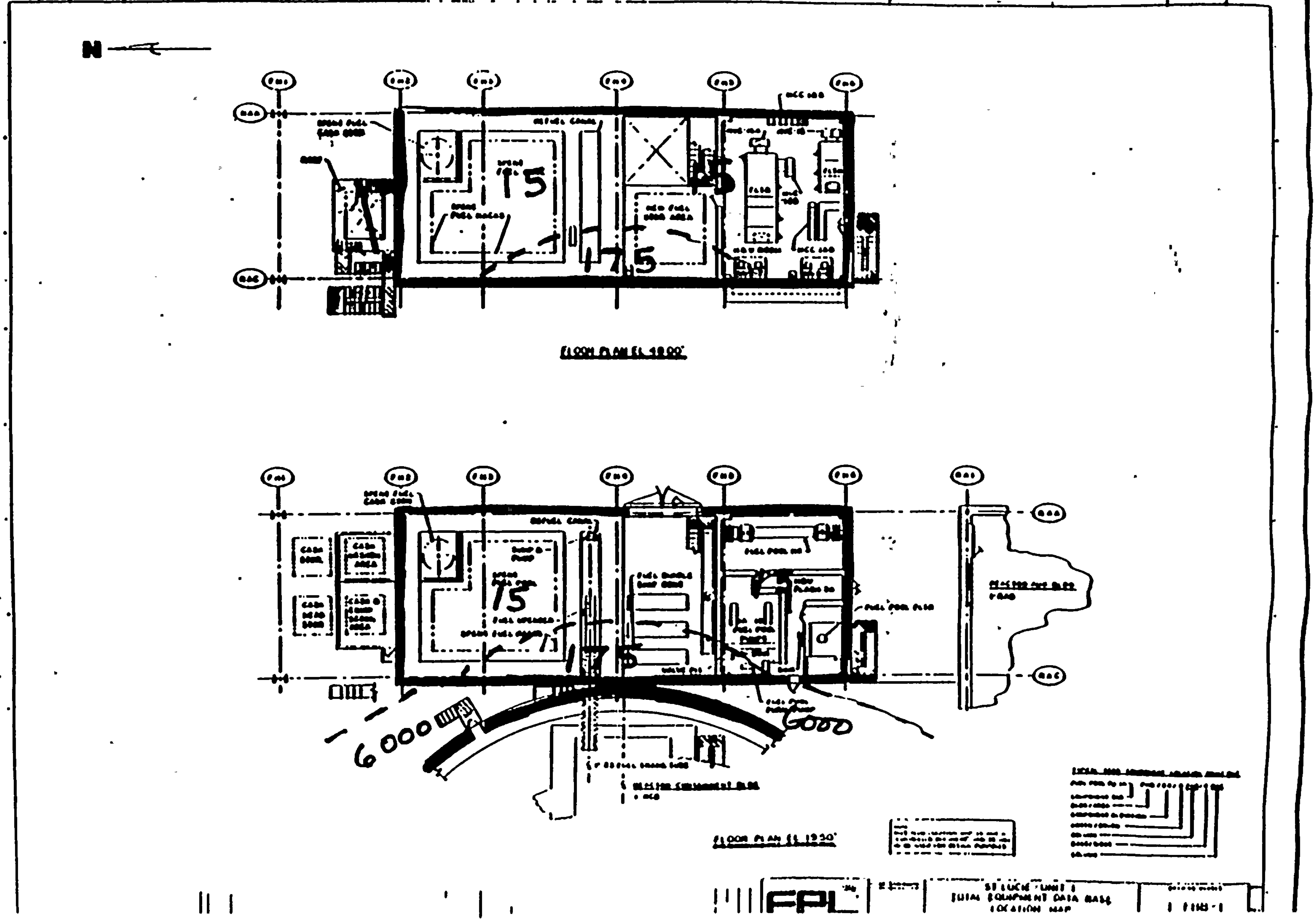
CONTAM (CPM) = 7250

AIR (35 CF SAMPLE)

PART = 1000 MR/HR

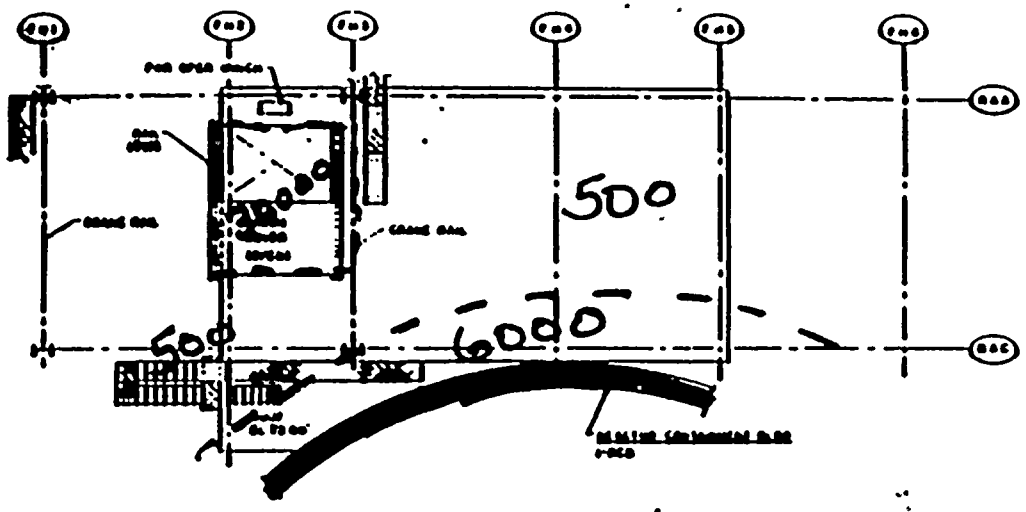
I_2 = 3.0 E^{-5} μCi/ml

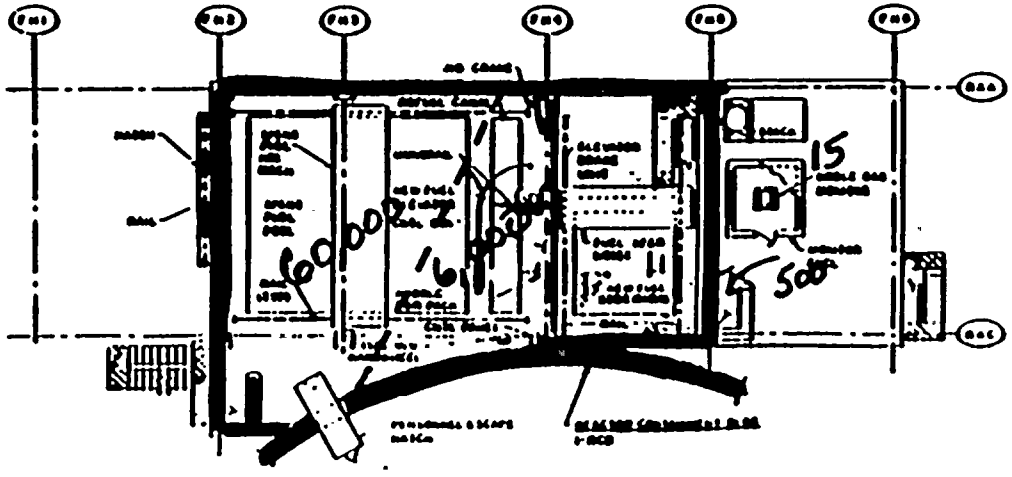
TIME: 1230 - 1244



TIME: 1230 - 1244

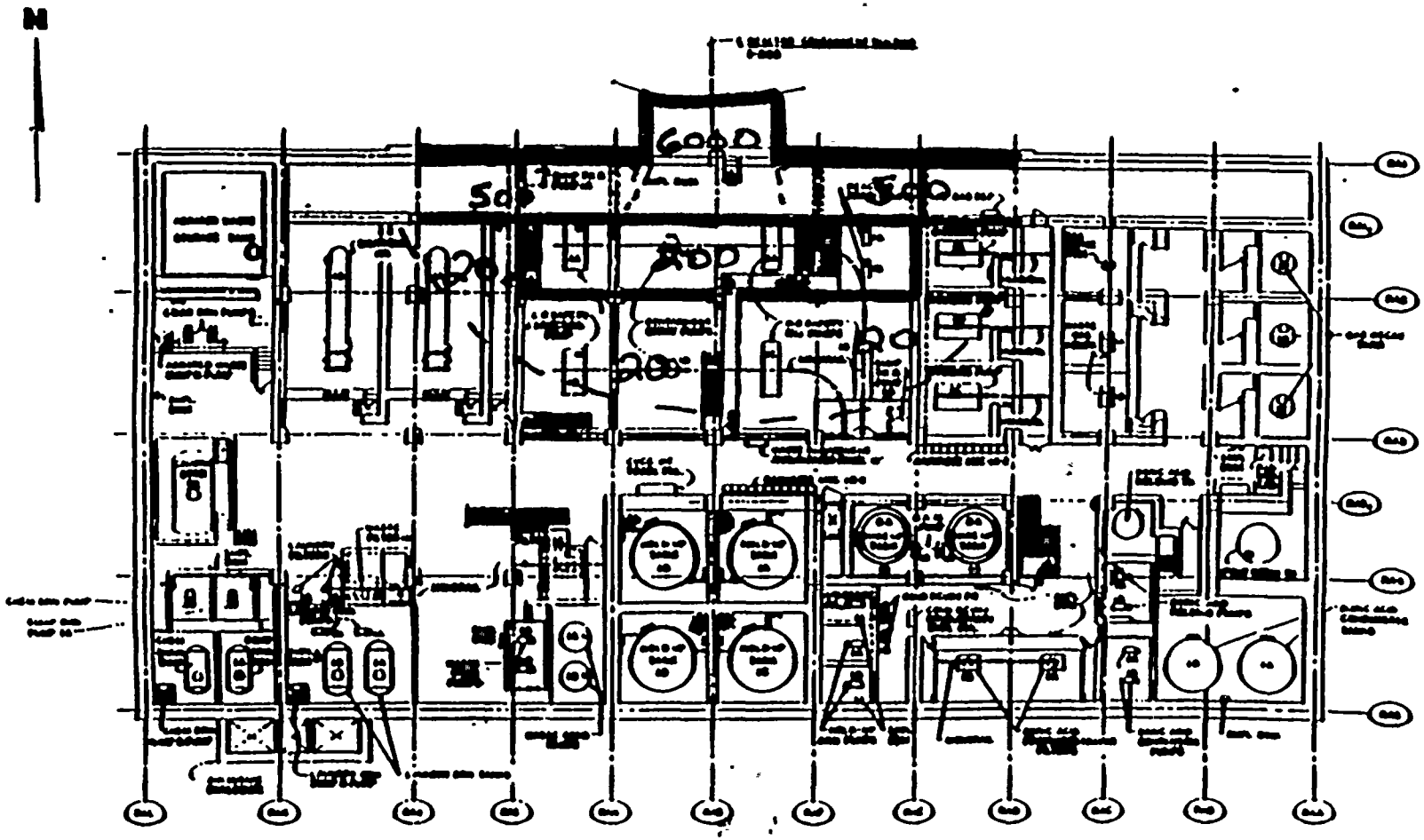
N





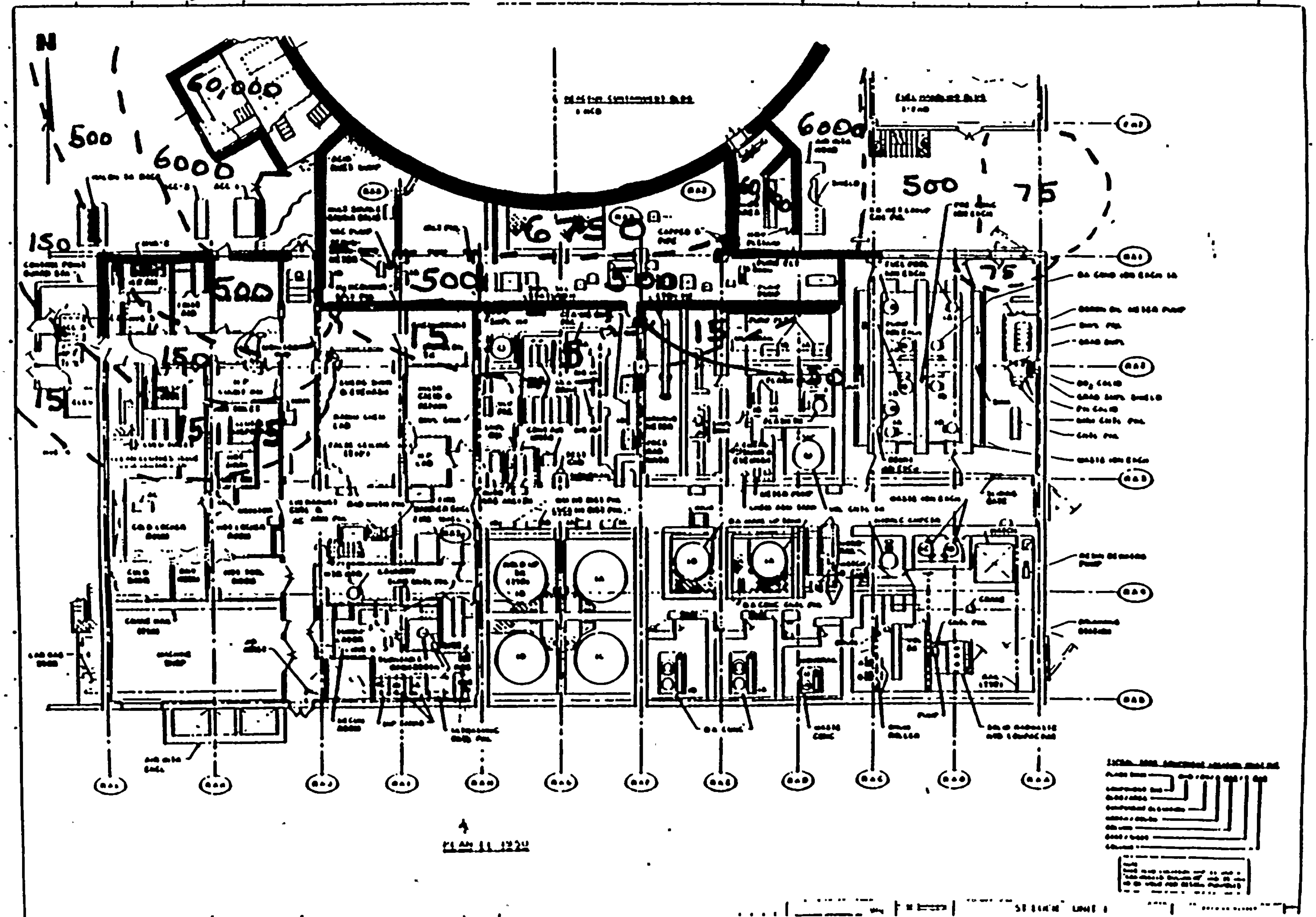
TIME: 1245 - 1259

ALL SHEETS
- DOSE RATES IN MR/HR B & (OPEN WINDOW = CLOSED)
  * EXCEPT FOR STEAM TRESTLE
- CONTAMINATION (CPM) = ACTUAL, EXCEPT STEAM TRESTLE
- AIR ACTIVITY = ACTUAL, EXCEPT FOR STEAM TRESTLE

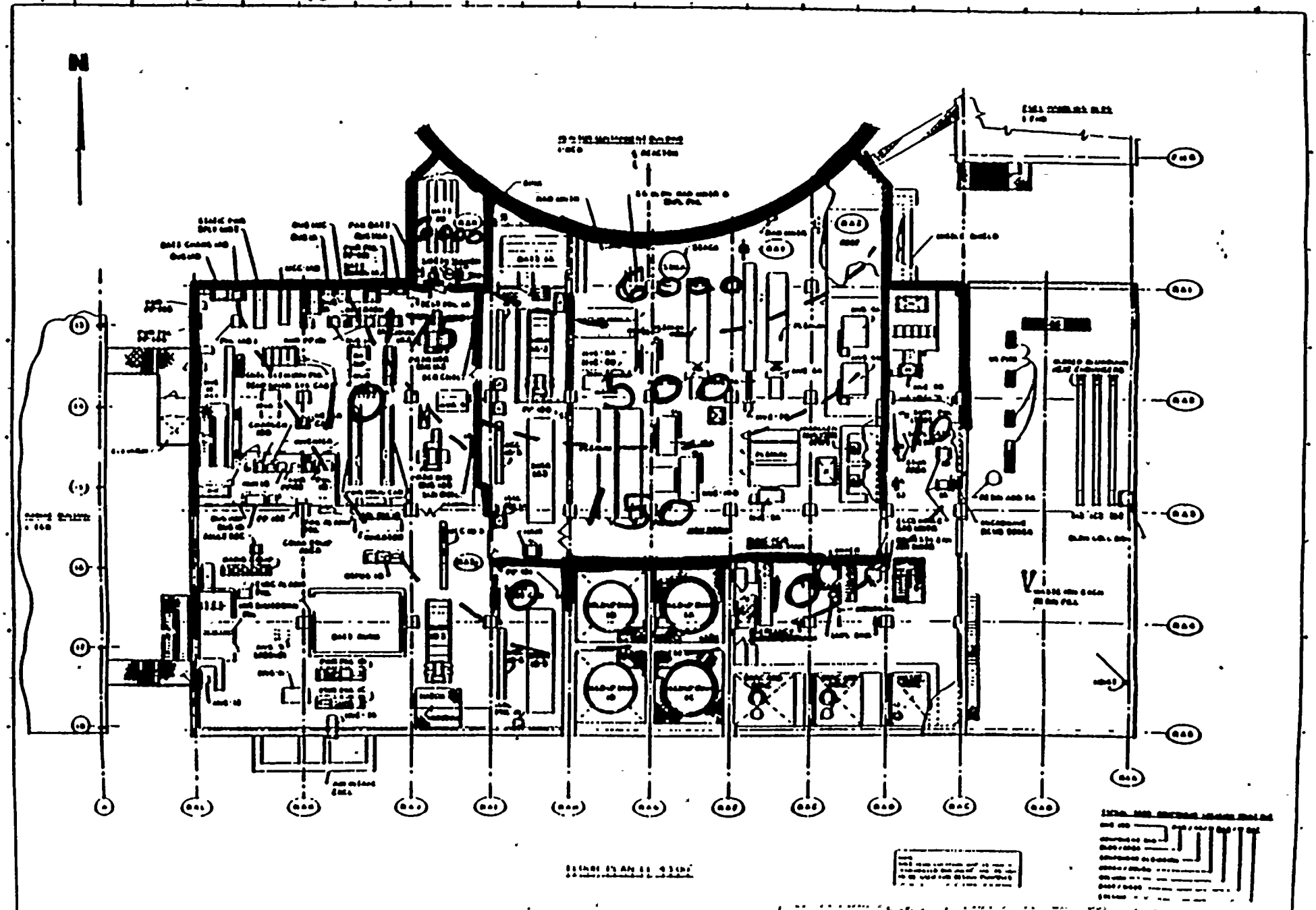


FLOOR PLAN 1:100

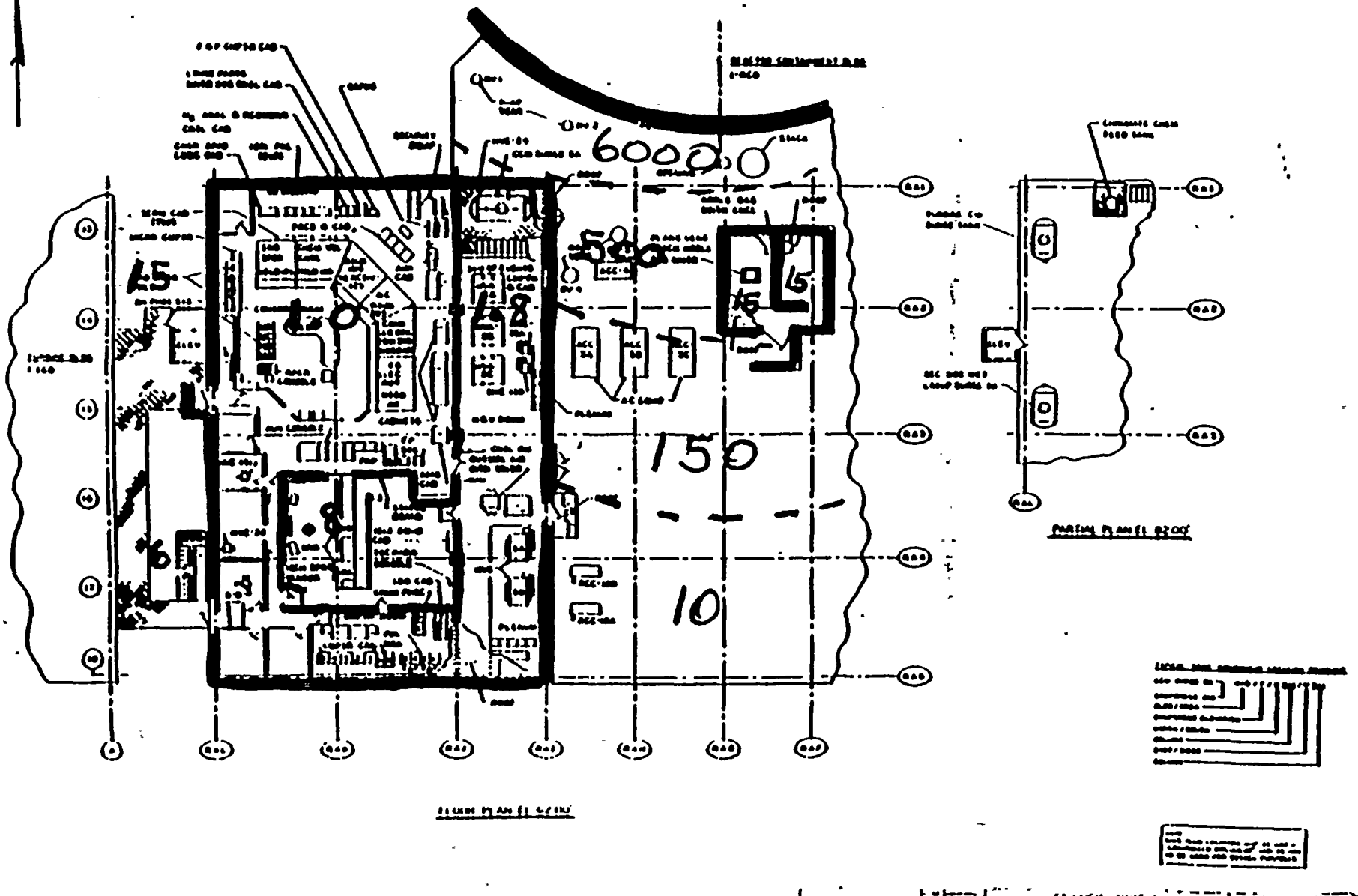
TIME: 1245-1259



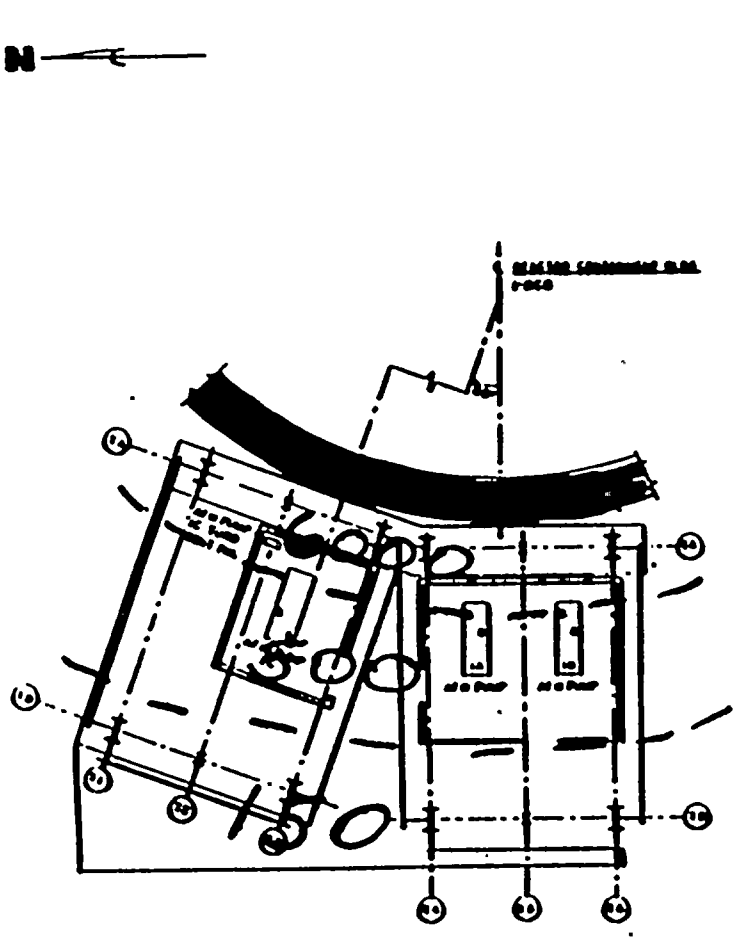
TIME: 1245-1259



TIME: 1245-1259



TIME: 1245-1259



600 (closed) / 900 (open)

CONTAM (CPM) = 8200

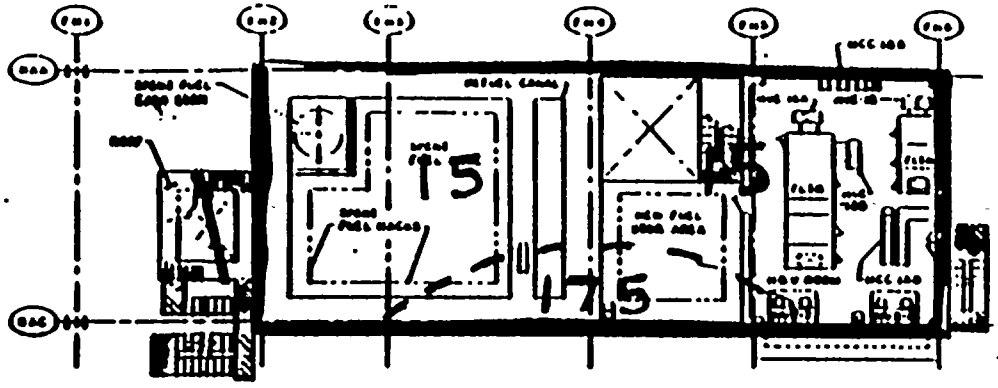
AIR (35 CF SAMPLE)

PART. = 500 NR/MR

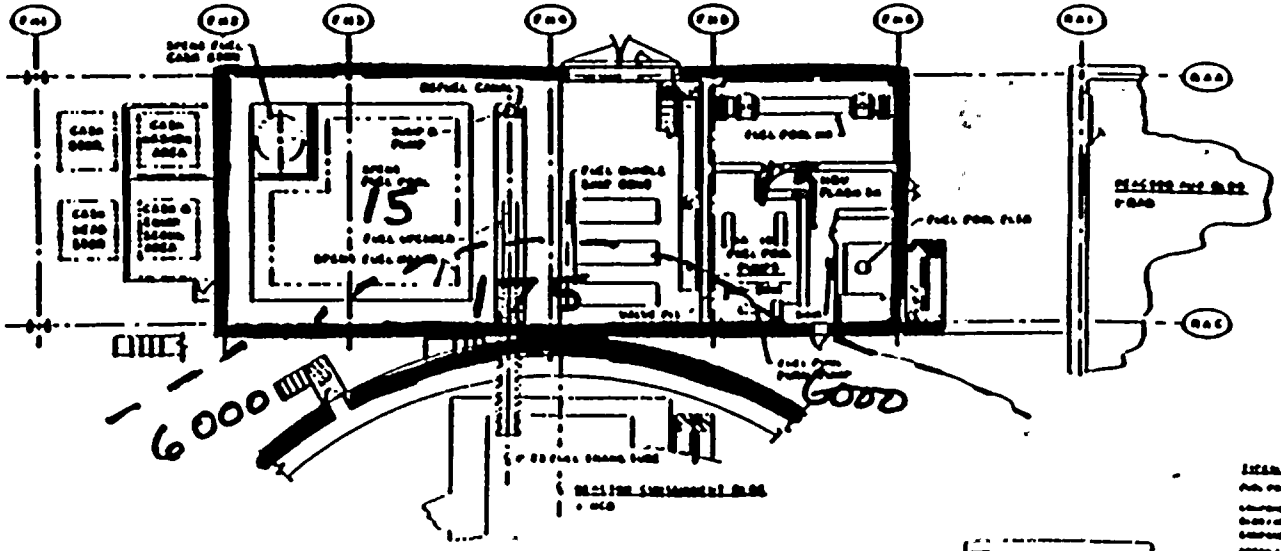
I_2 = 1.25 ε^{-5} gas/ml

TIME: 1245-1259

N



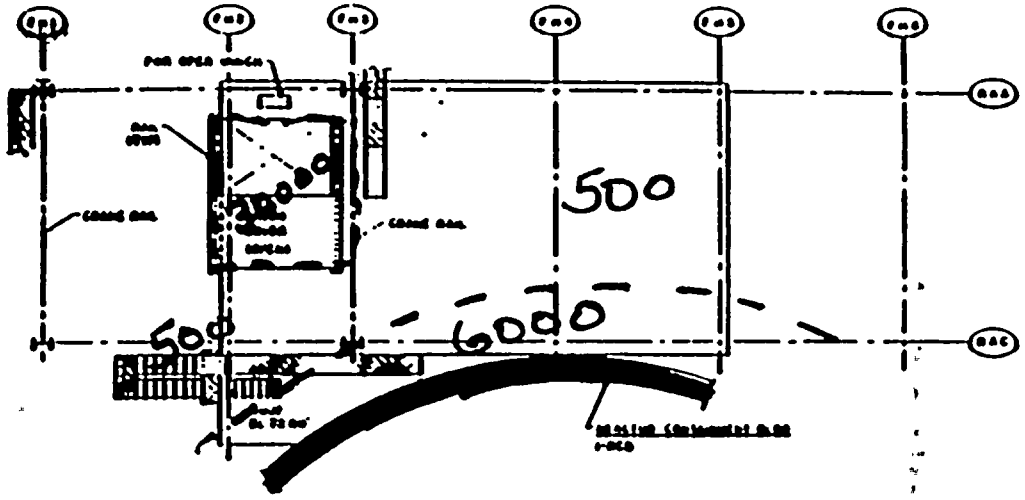
FLOOR PLAN EL 1900'

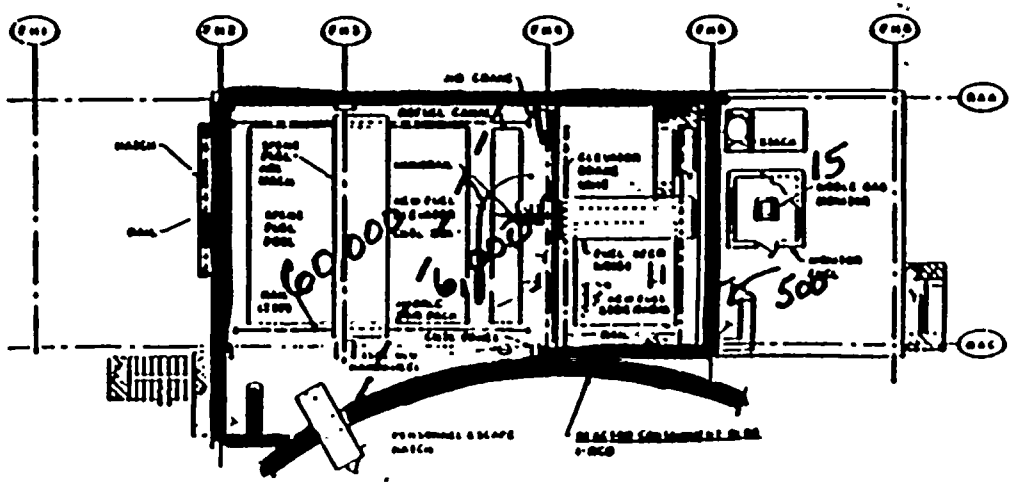


FLOOR PLAN EL 1930'

TIME: 1245 - 1259

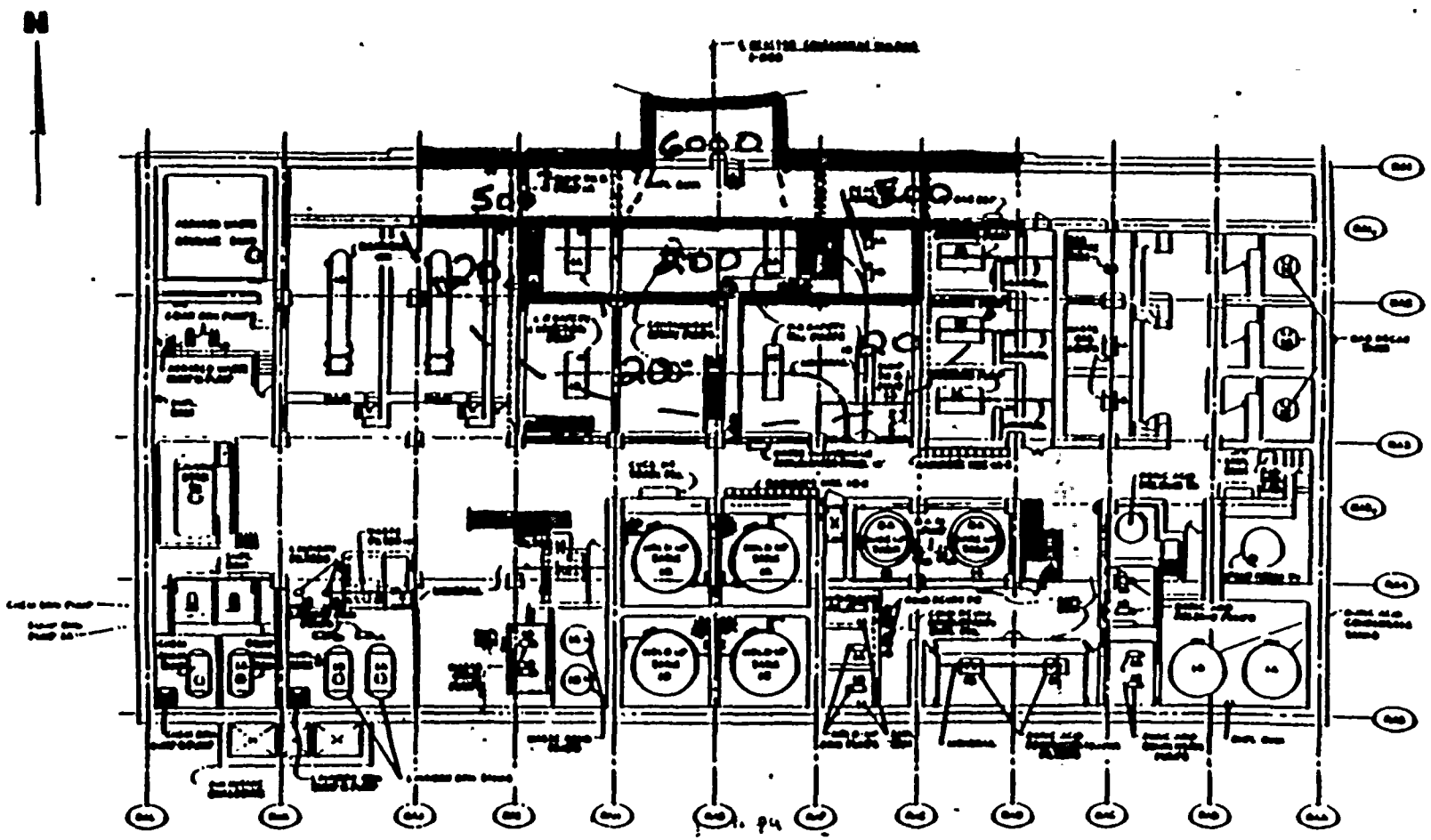
N





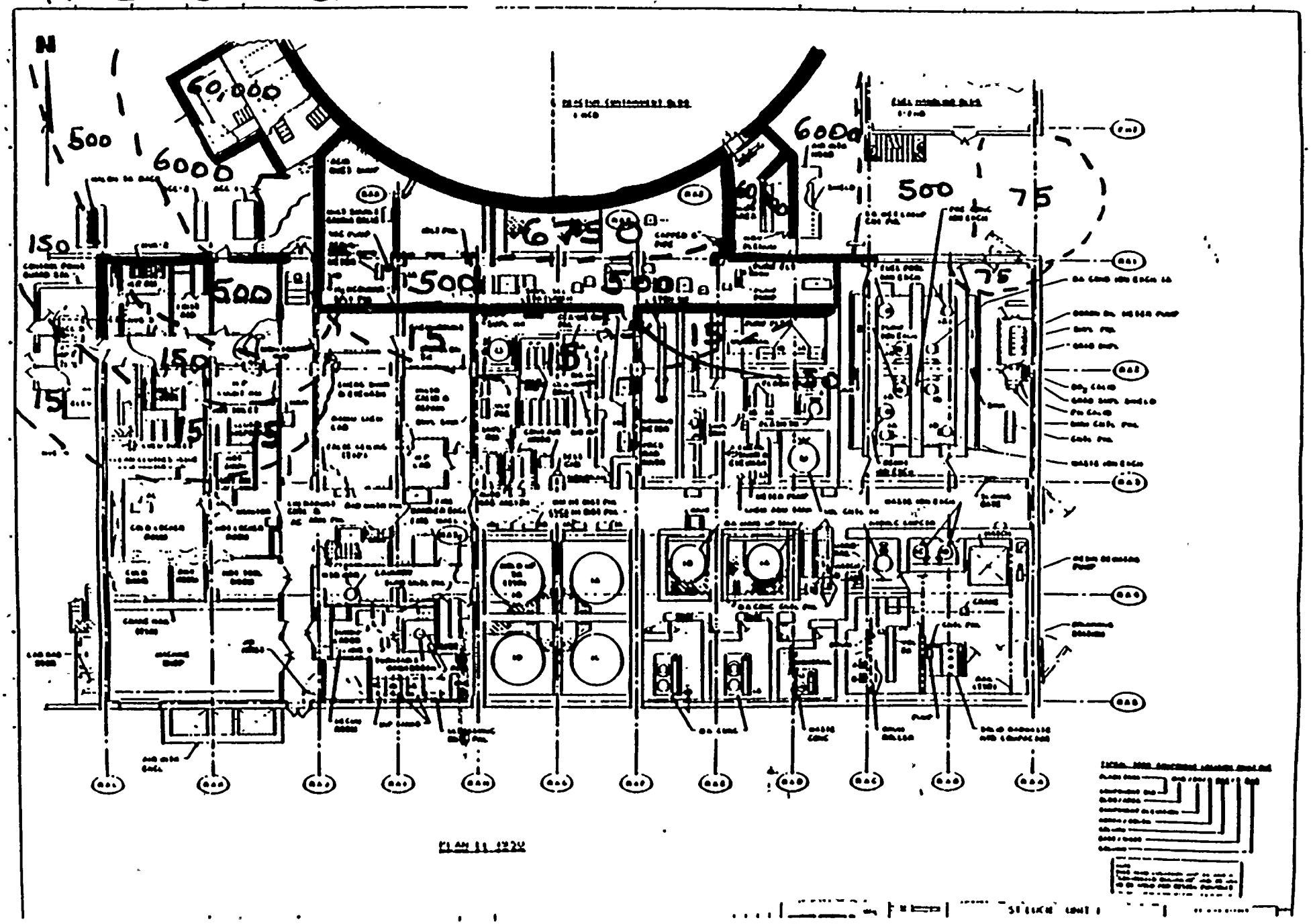
TIME: 1300 - 1314

ALL SHEETS
- DOSE RATES IN MR/HR B & (OPEN WINDOW = CLOSED)
  * EXCEPT FOR STEAM TRESTLE
- CONTAMINATION (CPM) = ACTUAL, EXCEPT STEAM TRESTLE
- AIR ACTIVITY = ACTUAL, EXCEPT FOR STEAM TRESTLE

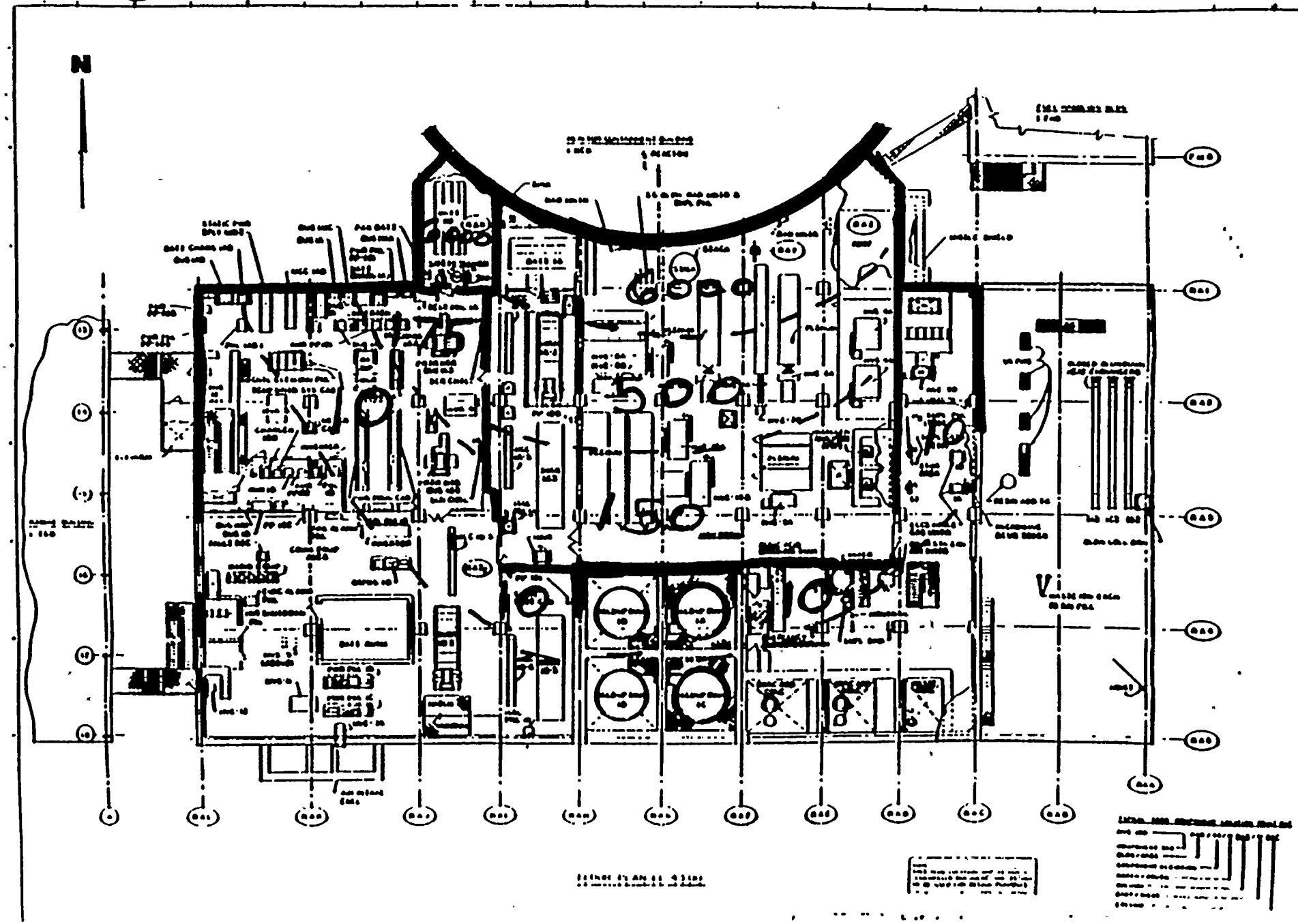


FLOOR PLAN (1:192)

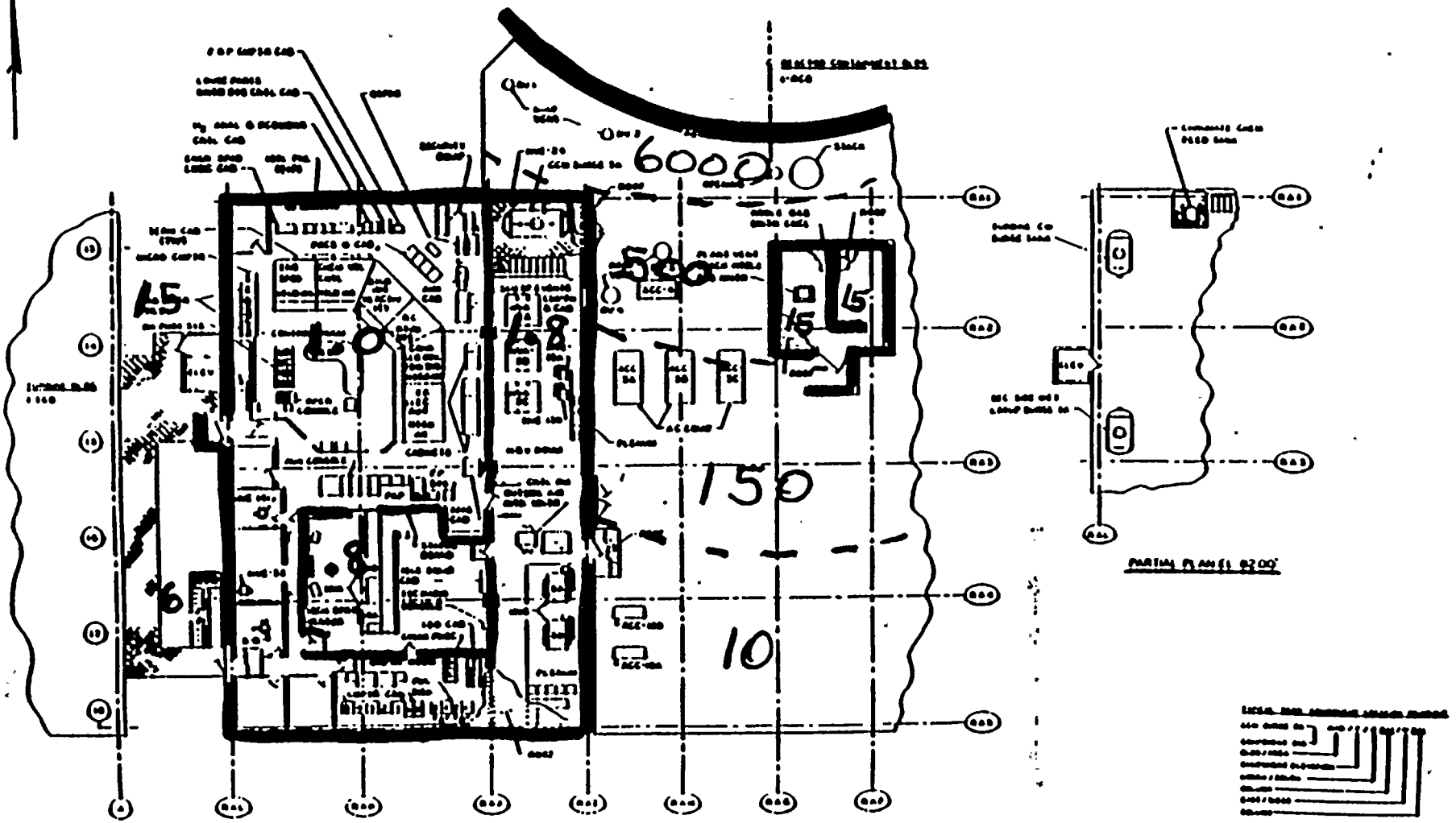
TIME: 1300-1314



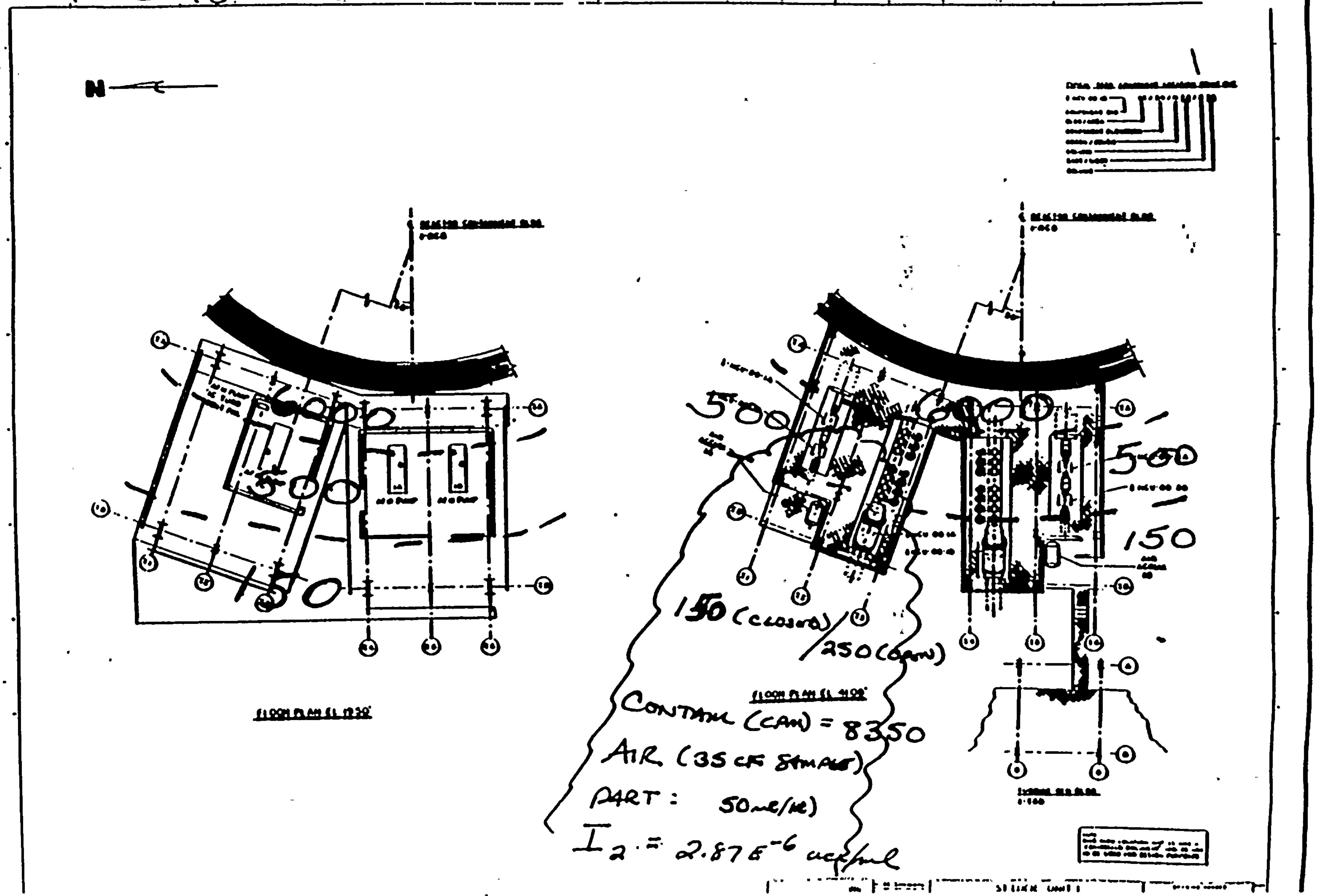
TIME: 1300-1314



TIME: 1300 - 1314

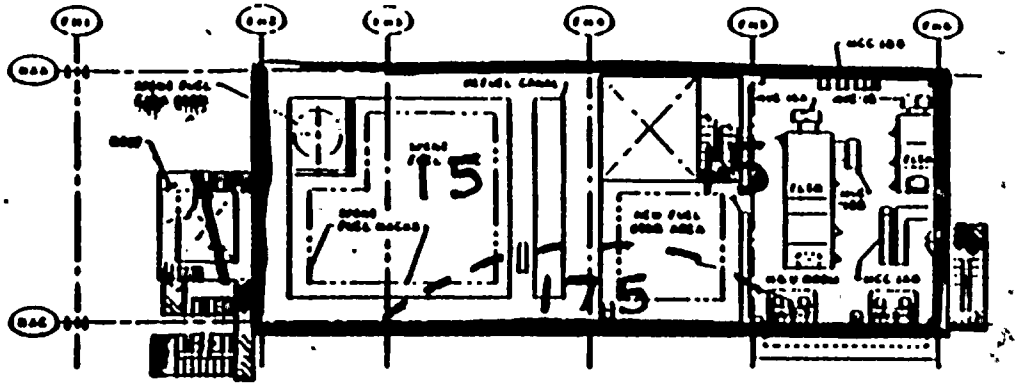


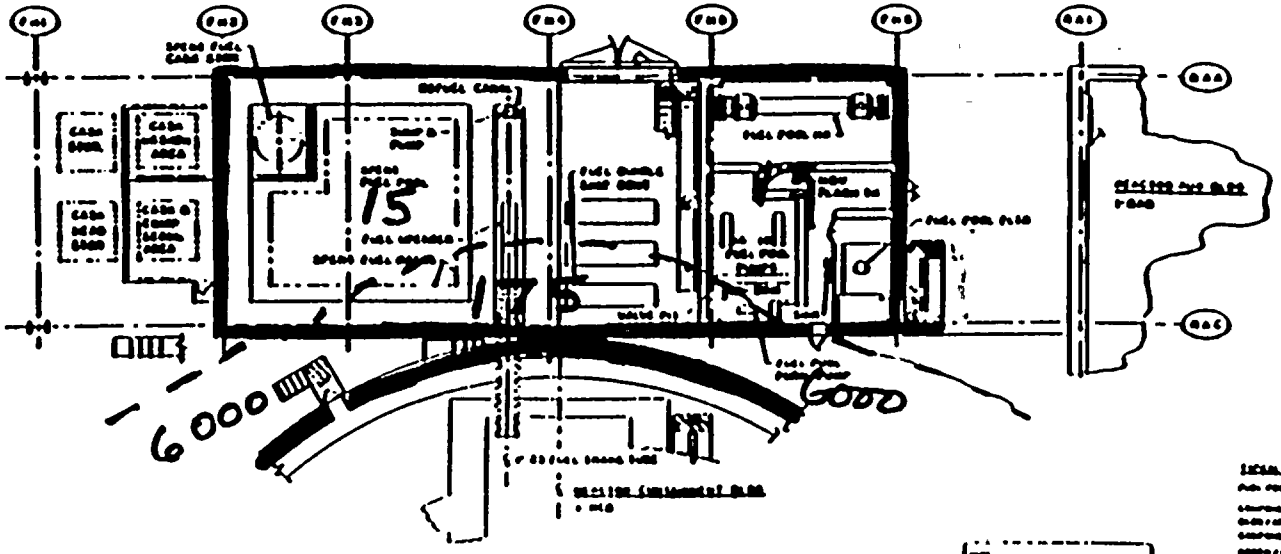
TIME: 1300-1314



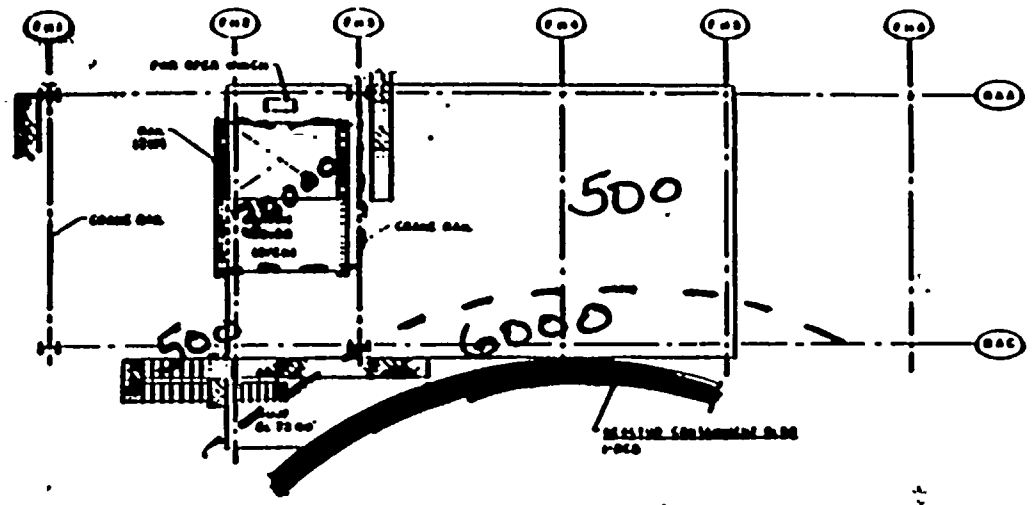
TIME: 1300 - 1314

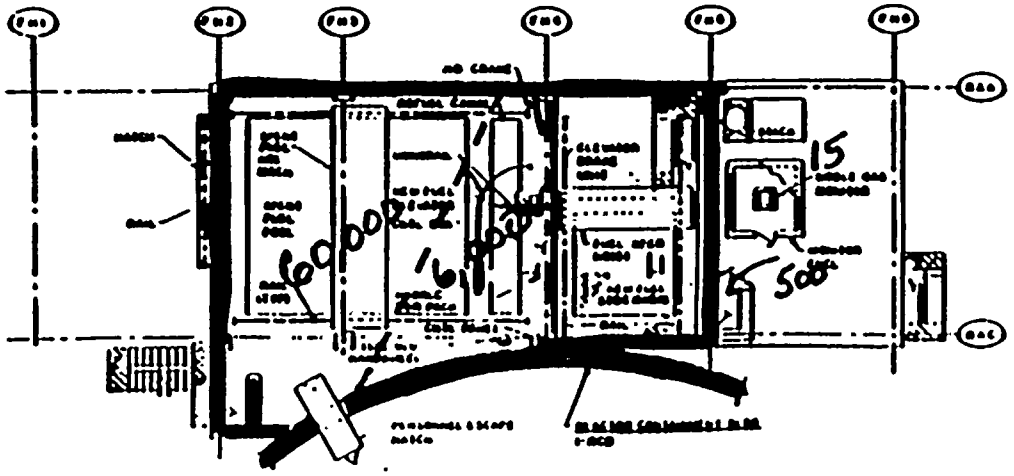
N





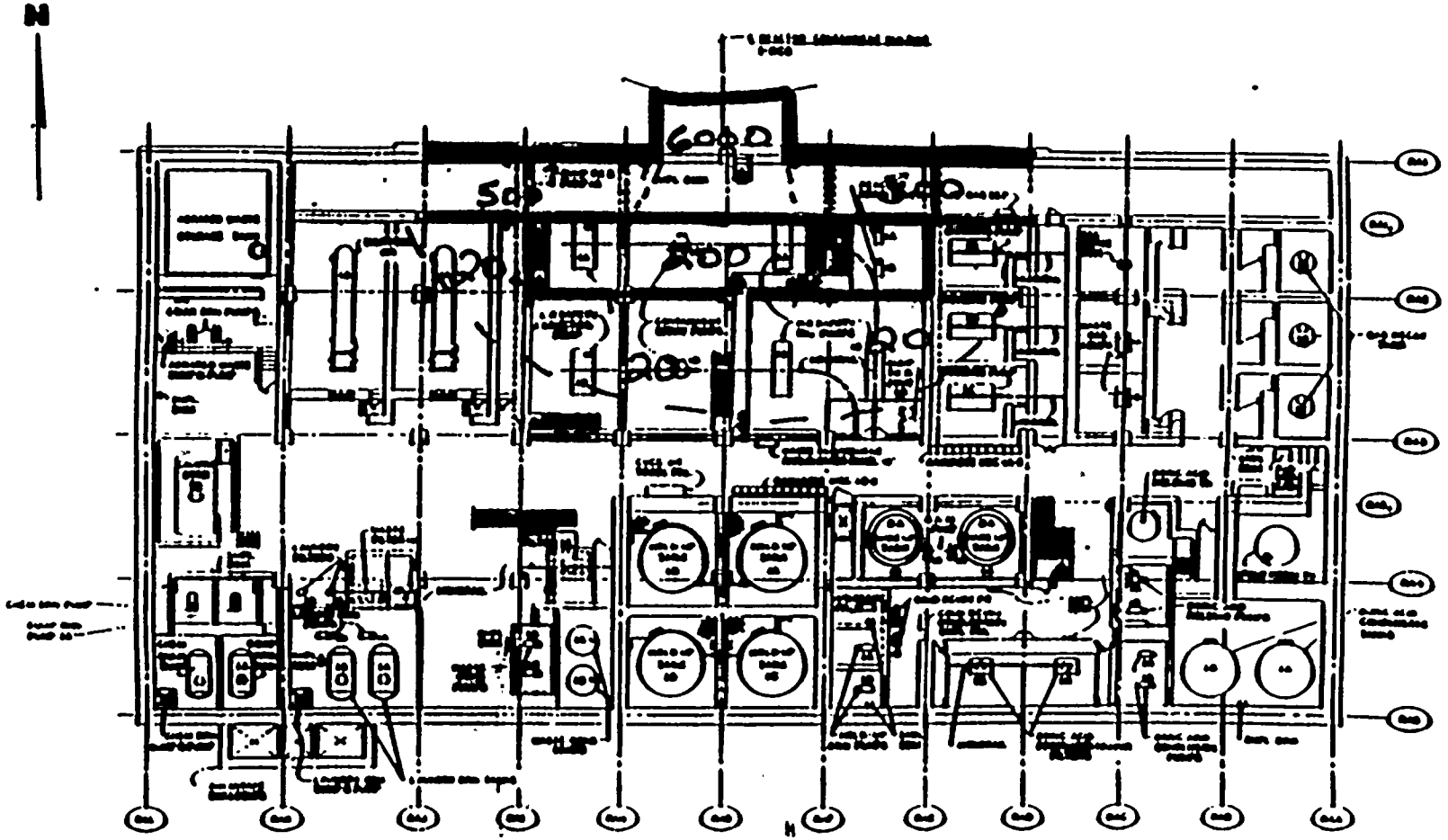
TIME: 1300-1314



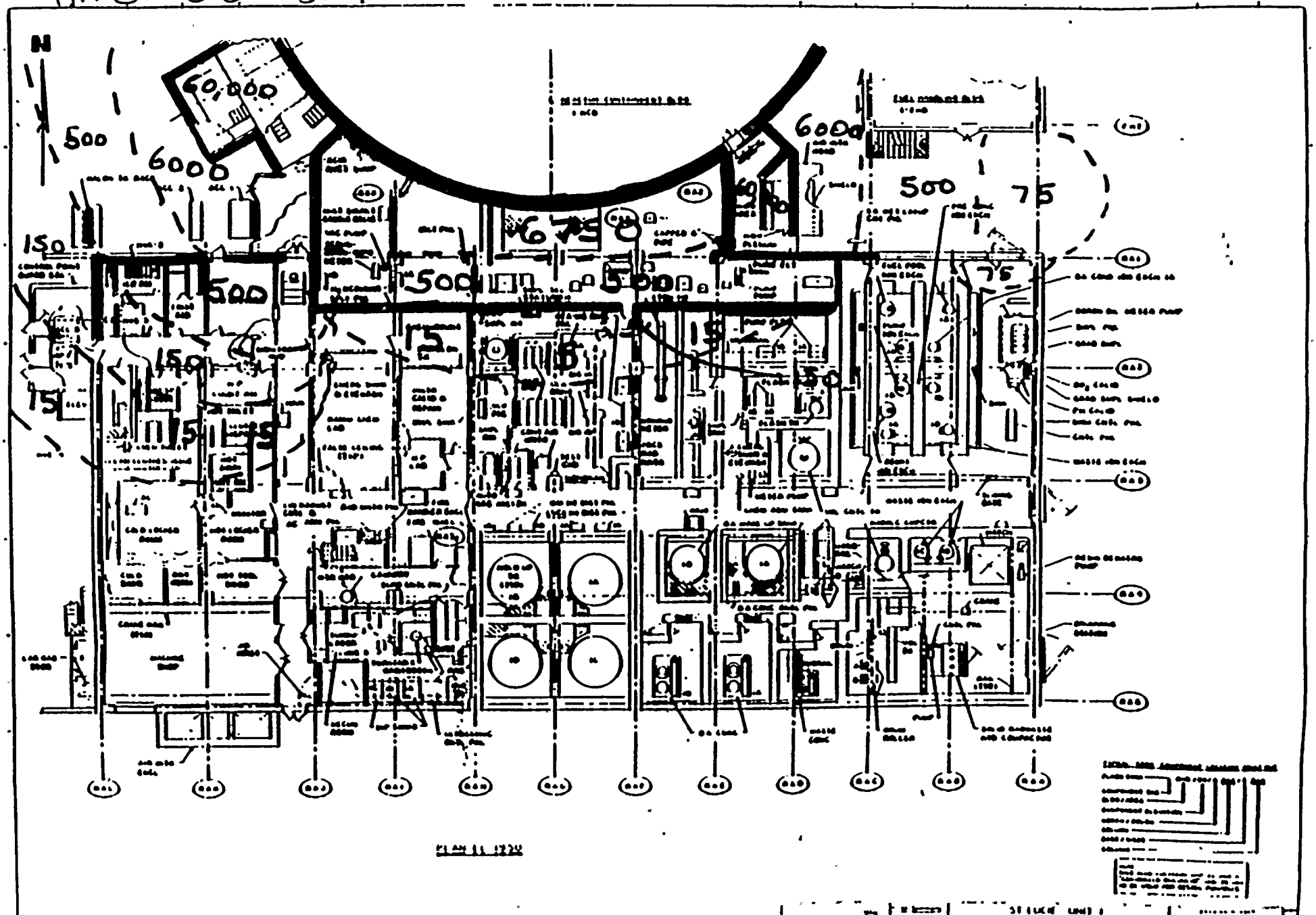


TIME · 1315 - 1329

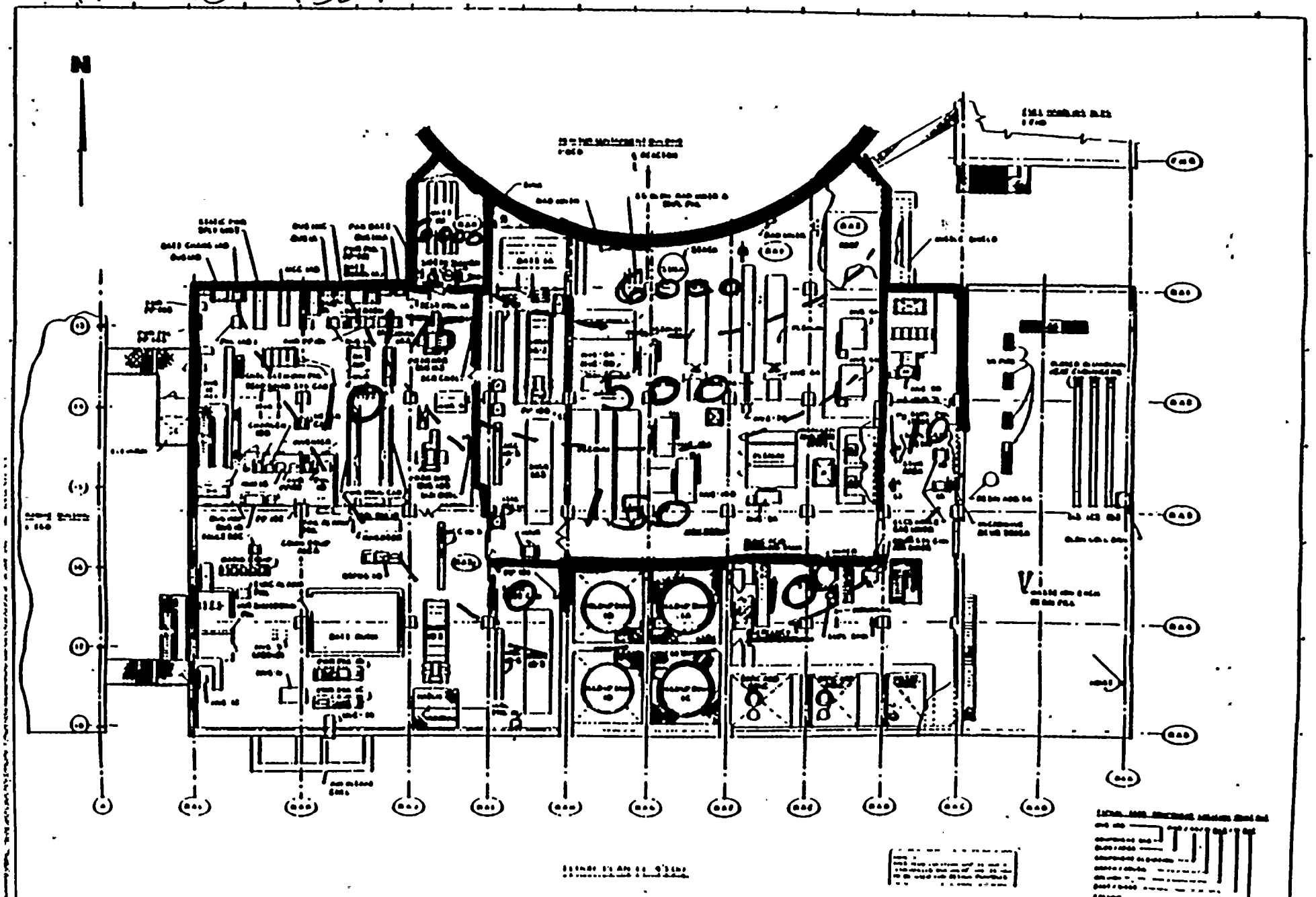
ALL SHEETS
· DOSE RATES IN MR/HR B & (OPEN WINDOW = CLOSED)
★ EXCEPT FOR STEAM TRESTLE
· CONTAMINATION (CPM) = ACTUAL, EXCEPT STEAM TRESTLE
· AIR ACTIVITY = ACTUAL, EXCEPT FOR STEAM TRESTLE



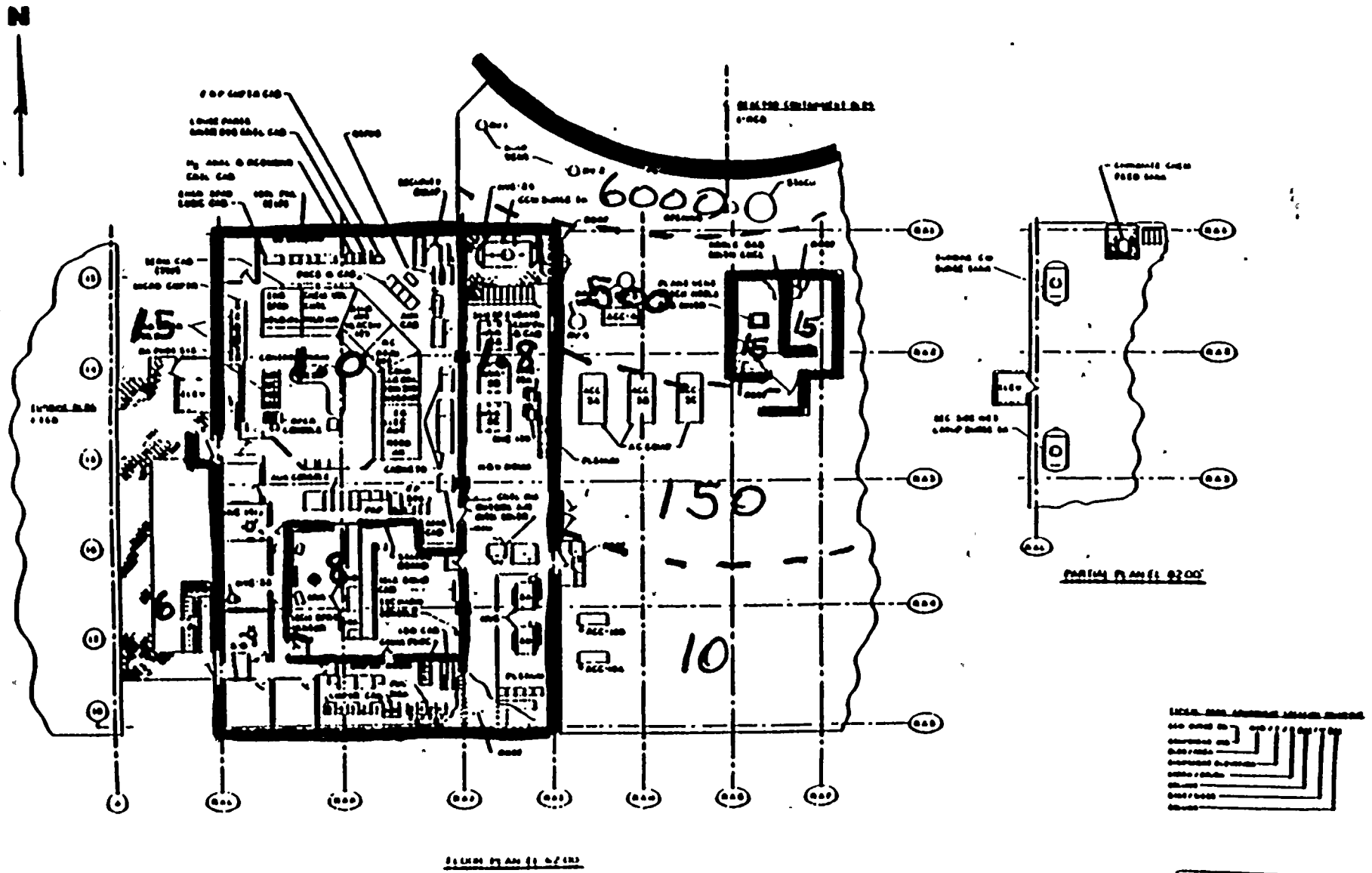
TIME: 1315 - 1329



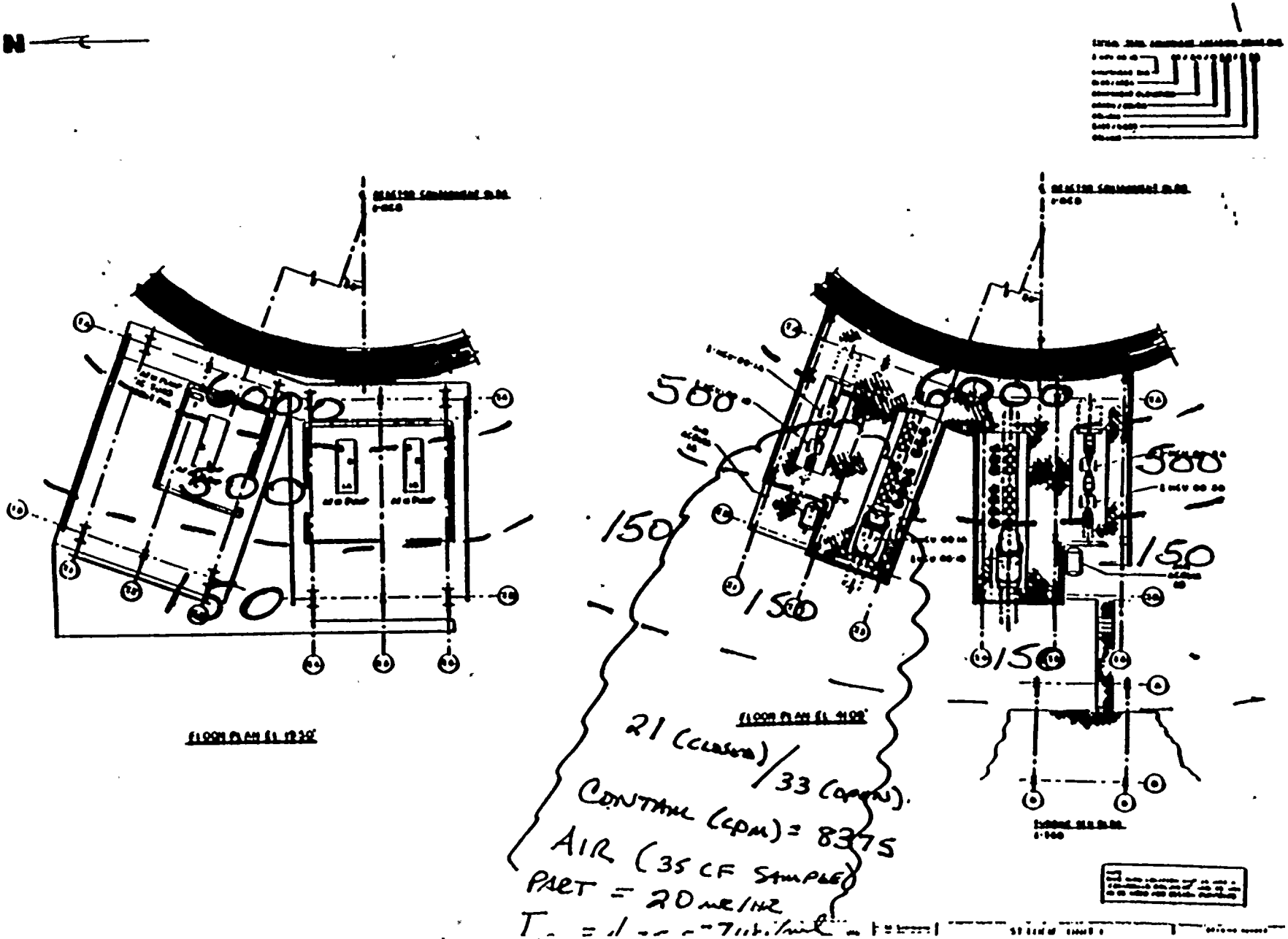
TIME: 1315-1329



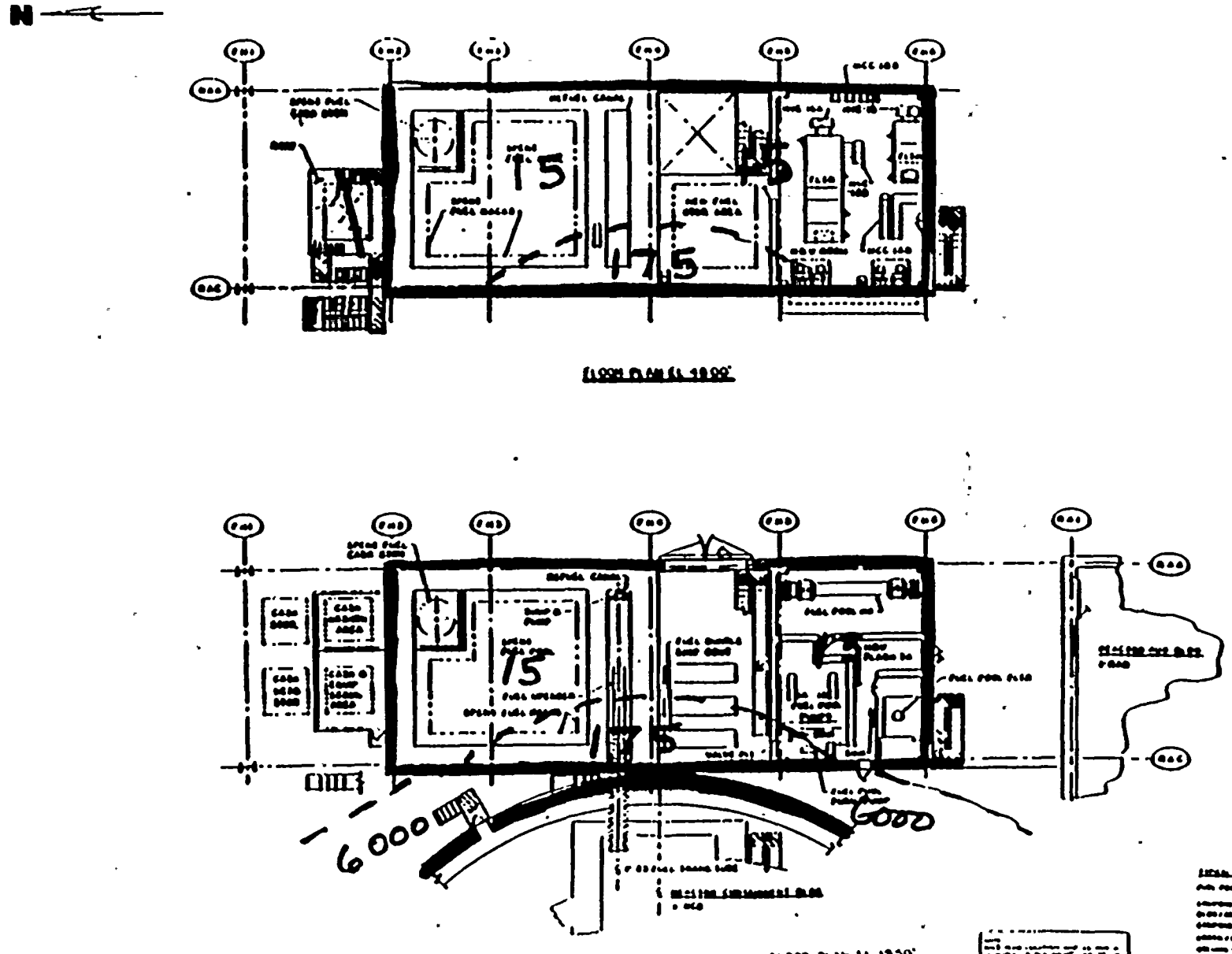
TIME: 1315-1329



TIME: 1315-1329

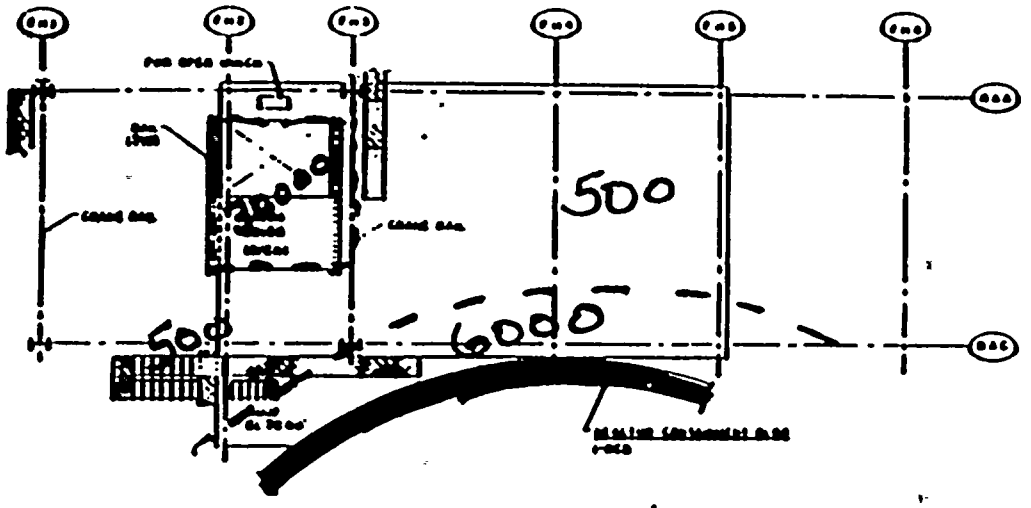


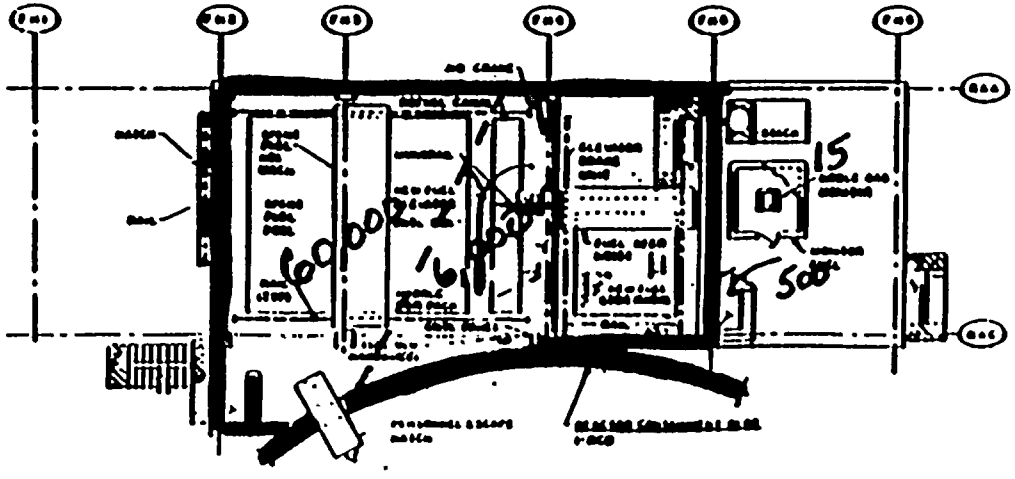
TIME: 1315- 1329



TIME: 1315 - 1329

N

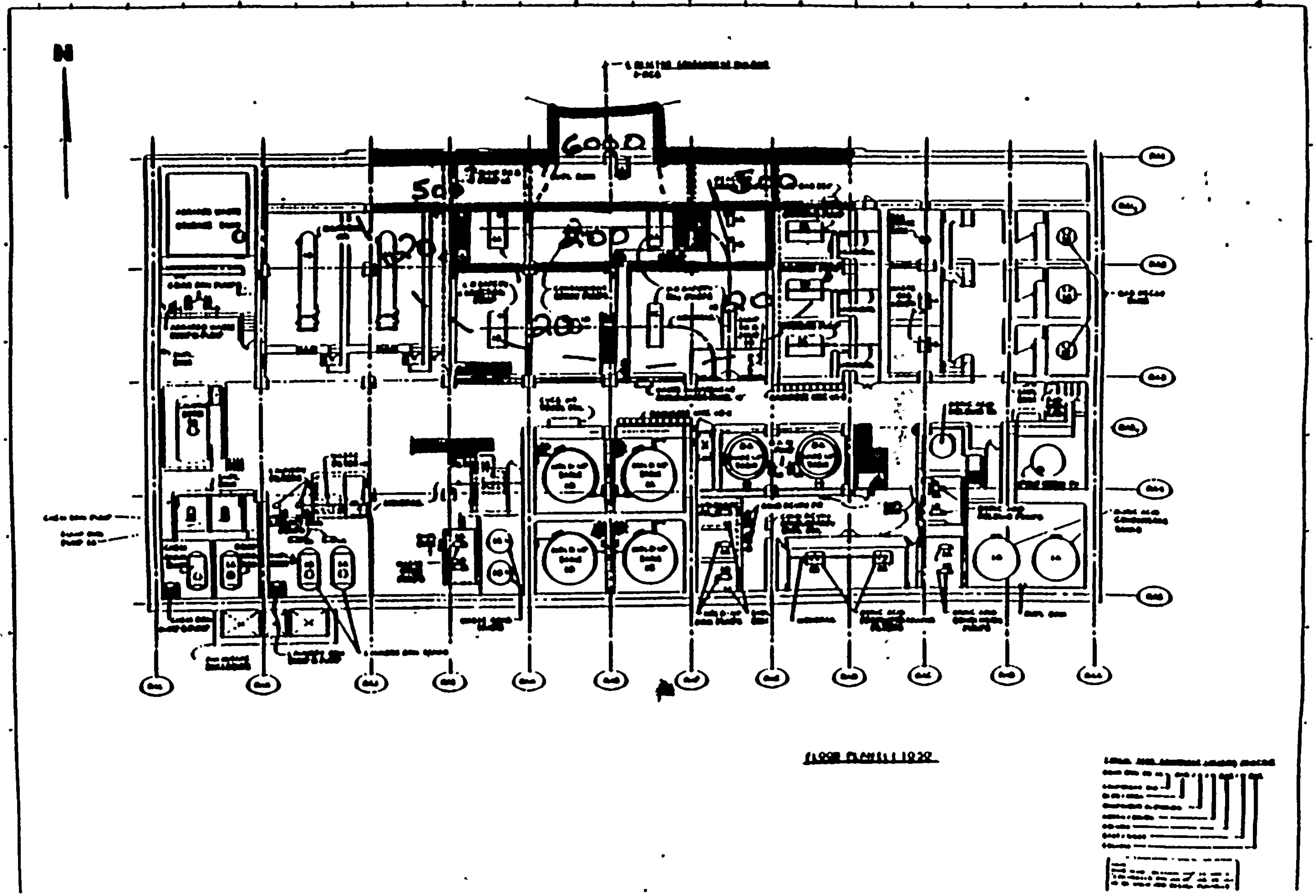




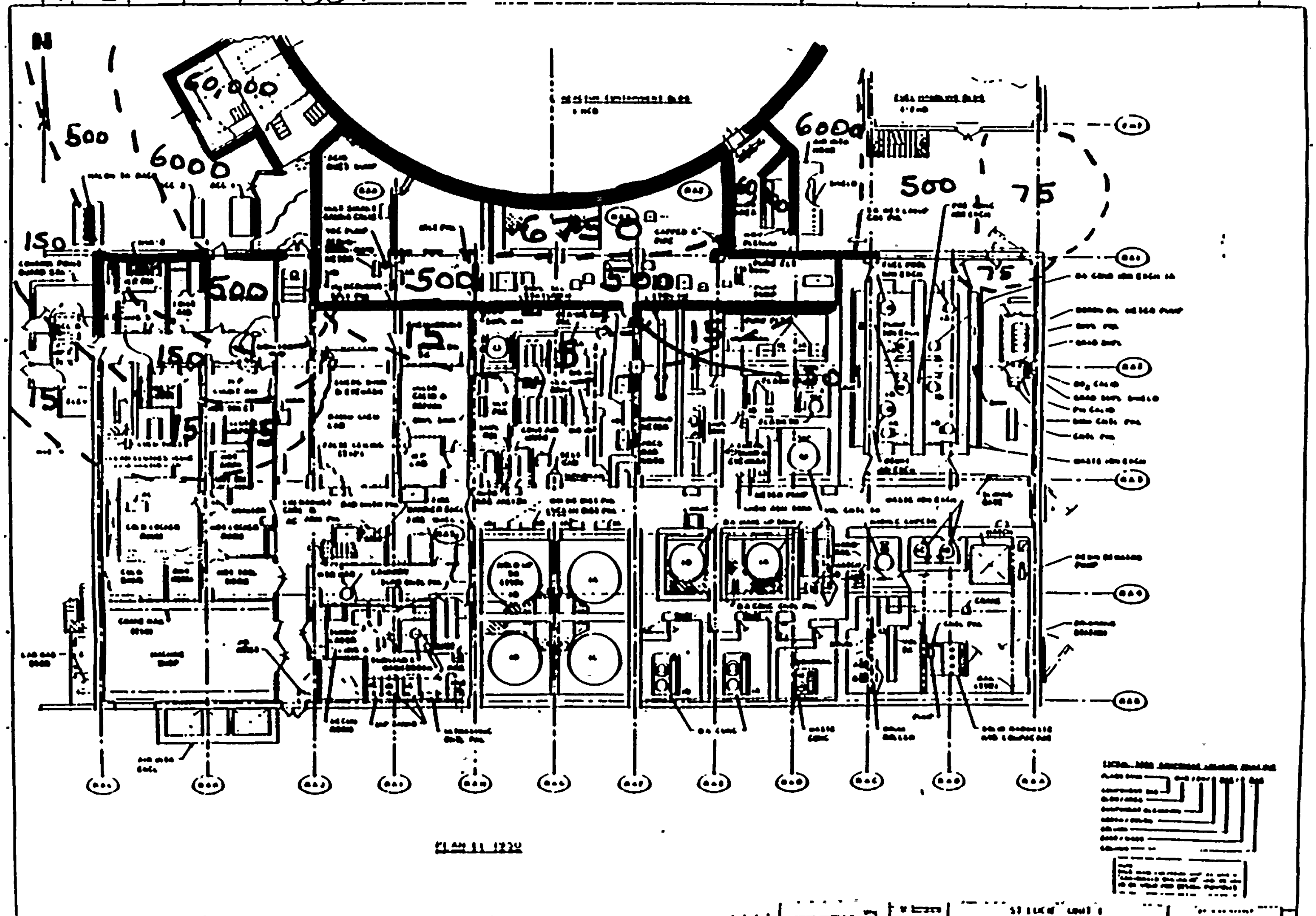
TIME: 1330 - 1359

ALL SHEETS

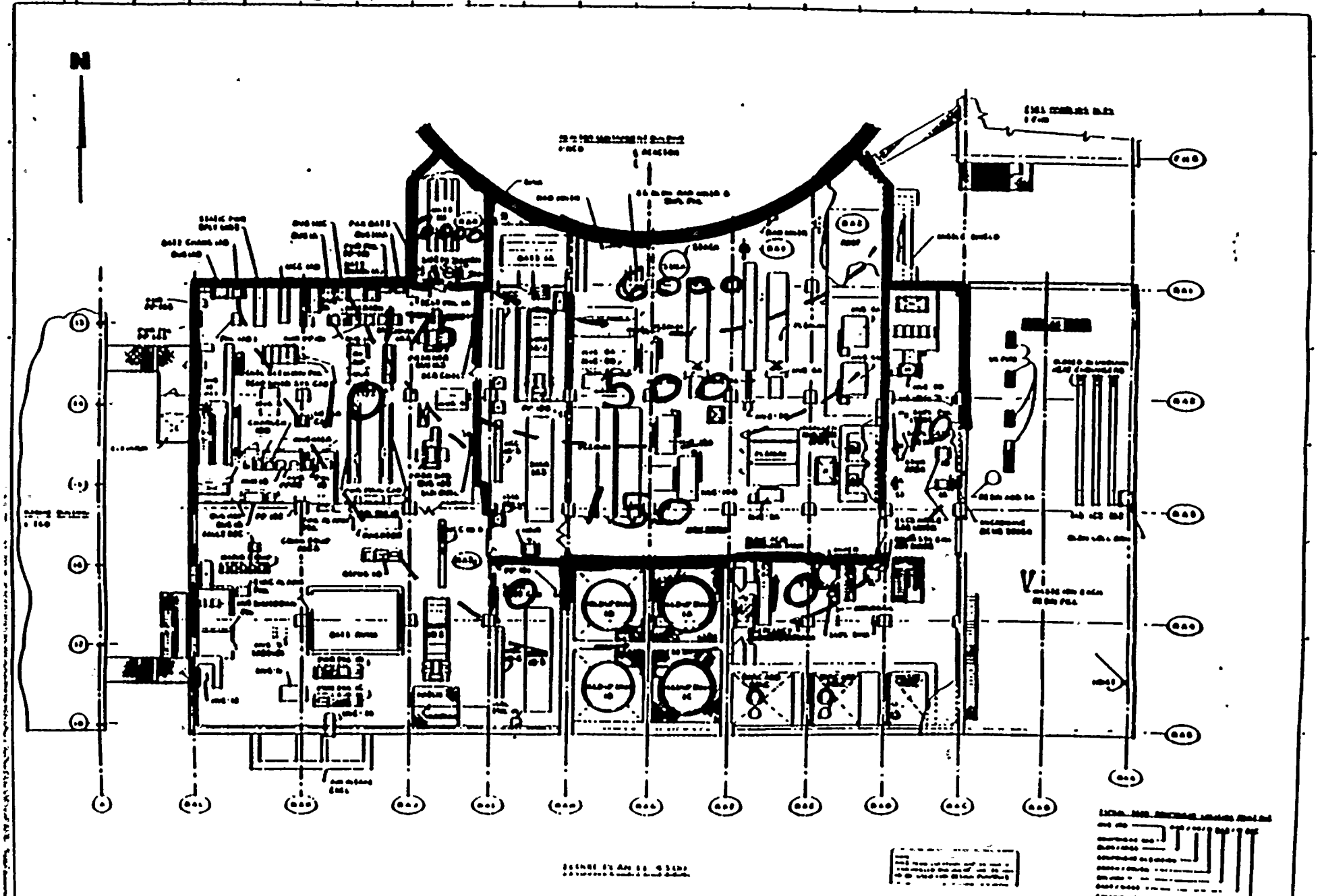
- DOSE RATES IN MR/HR βγ (OPEN WINDOW = CLOSED)
  - ★ EXCEPT FOR STEAM TRESTLE
- CONTAMINATION (CPM) = ACTUAL, EXCEPT STEAM TRESTLE
- AIR ACTIVITY = ACTUAL, EXCEPT FOR STEAM TRESTLE



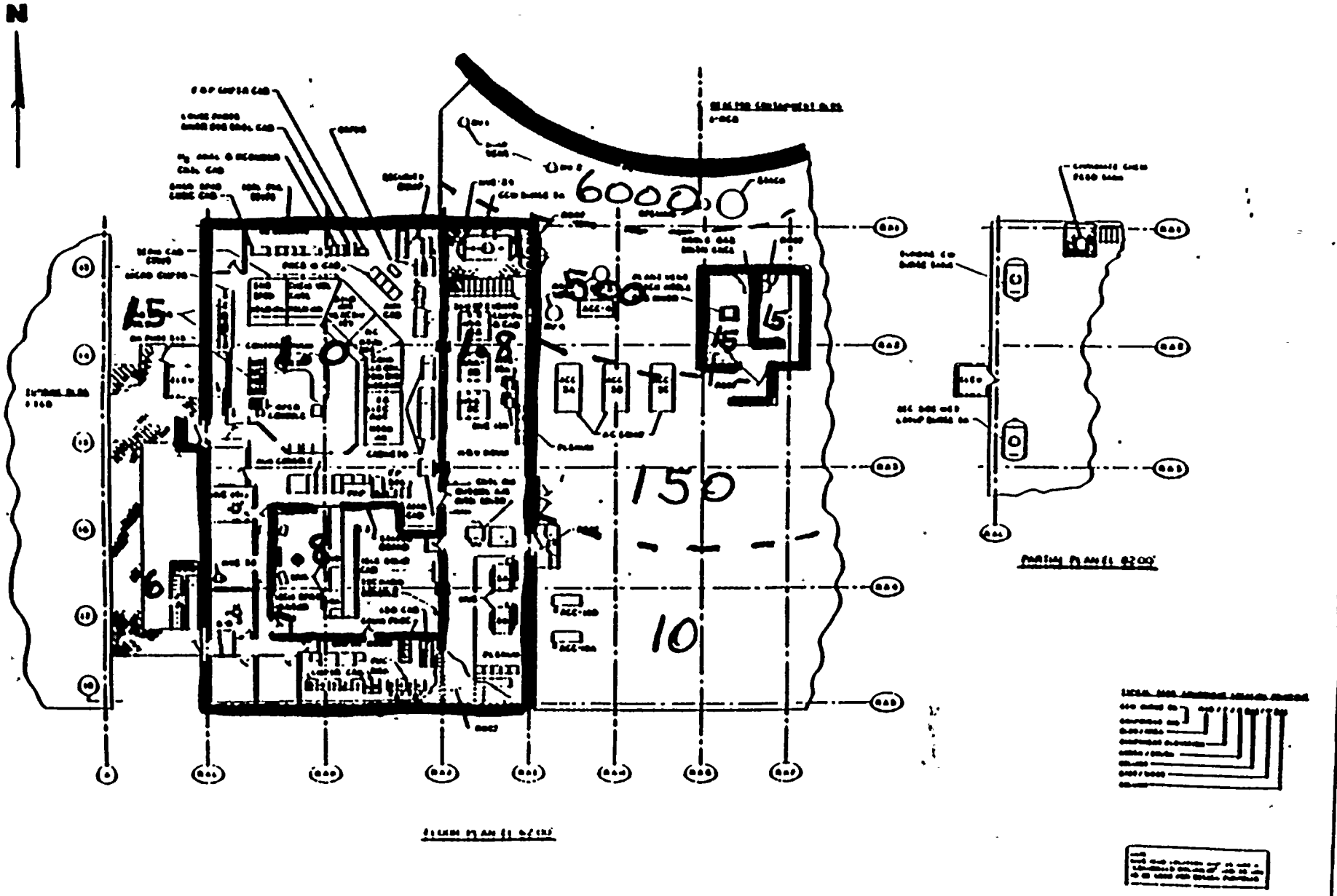
TIME: 1330-1359



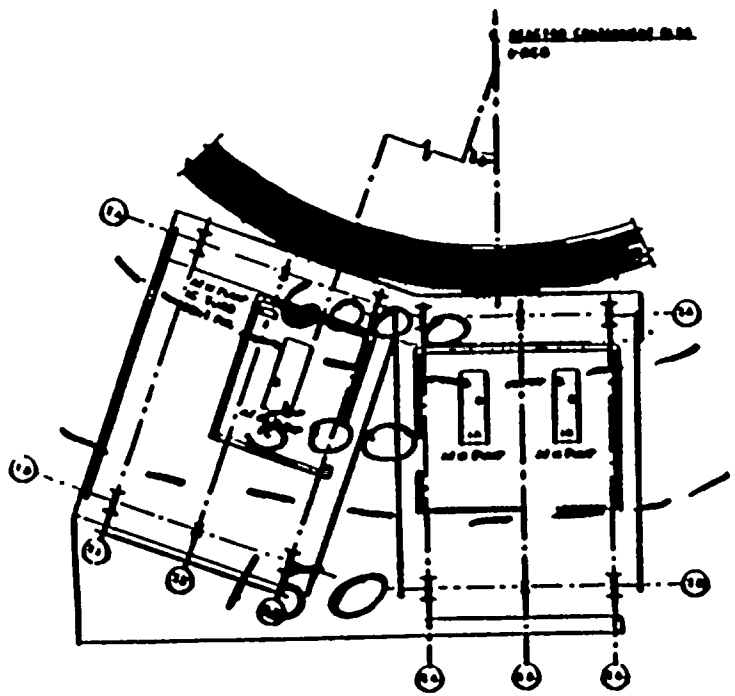
TIME: 1330-1359

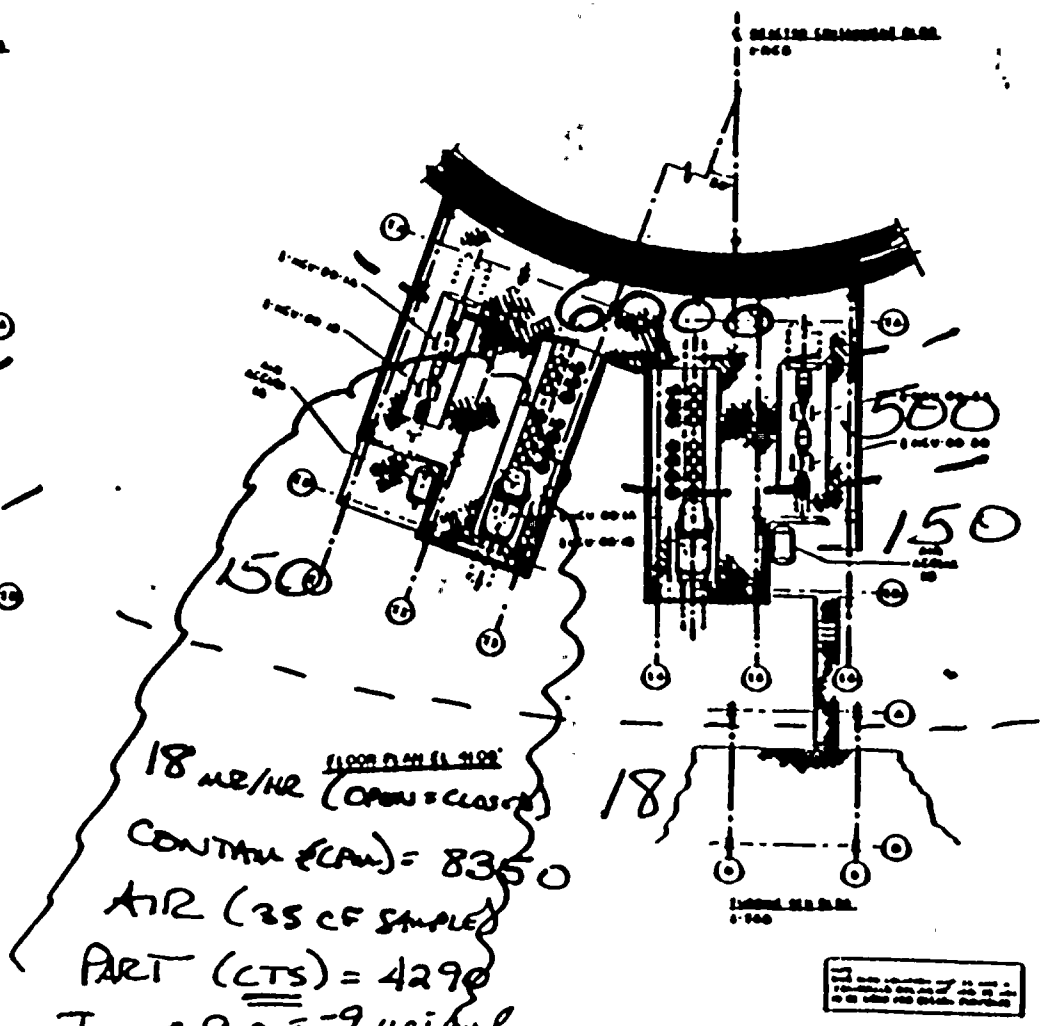


TIME: 1330 - 1359



TIME: 1330-1359





18 MR/HR (OPEN = CLOSED)

CONTAM (CPM) = 8350

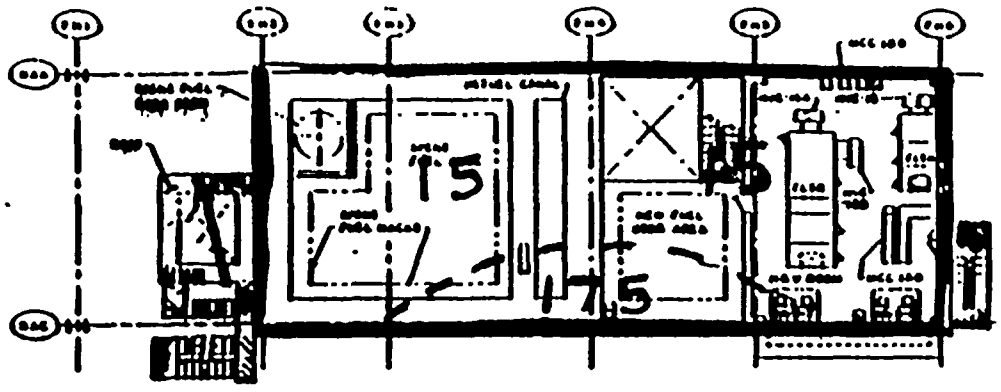
AIR (35 CF SAMPLE)

PART (CTS) = 4290

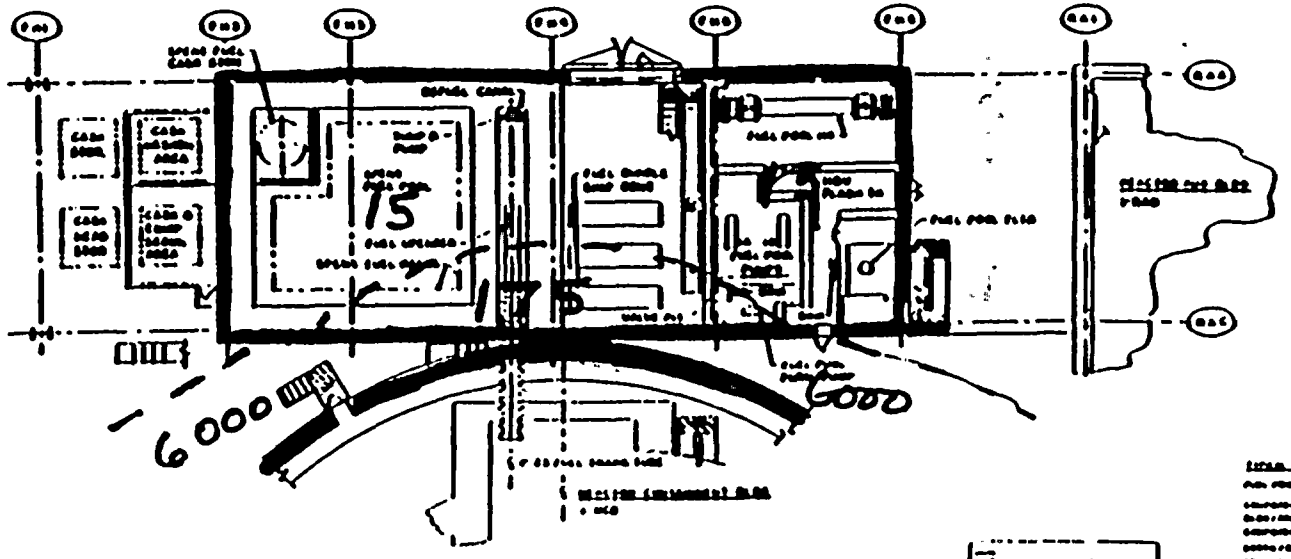
$I_2 = 9.0E-9$ uci/ml

TIME: 1330-1359

N



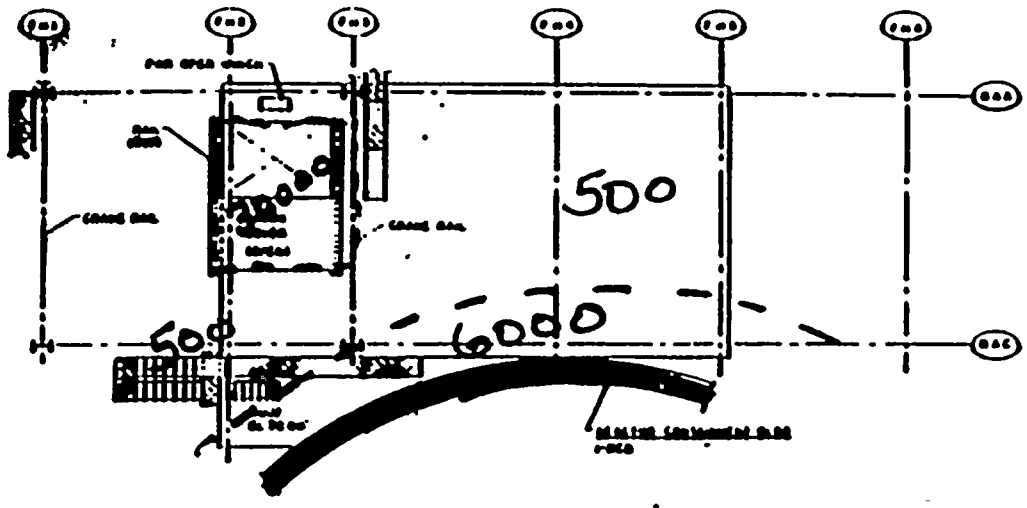
FLOOR PLAN EL 1900'

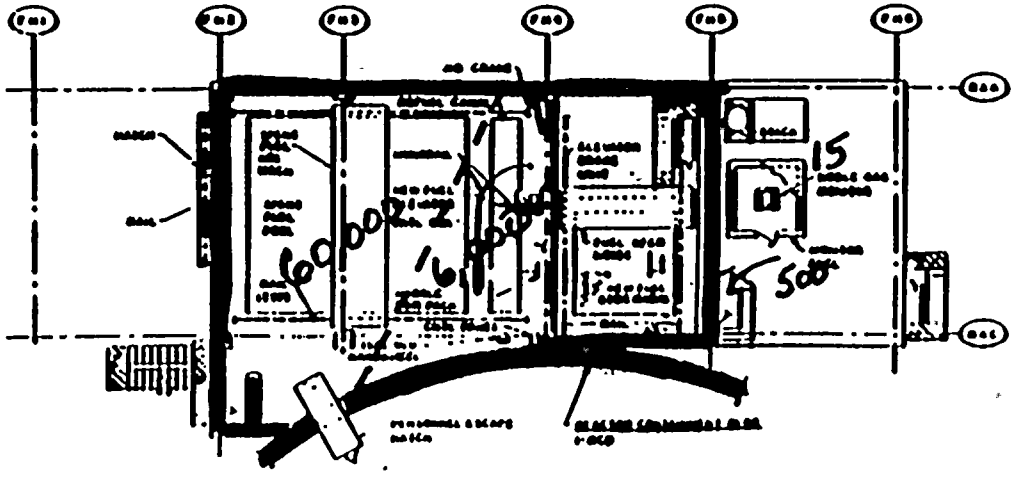


FLOOR PLAN EL 1920'

TIME: 1330-1359

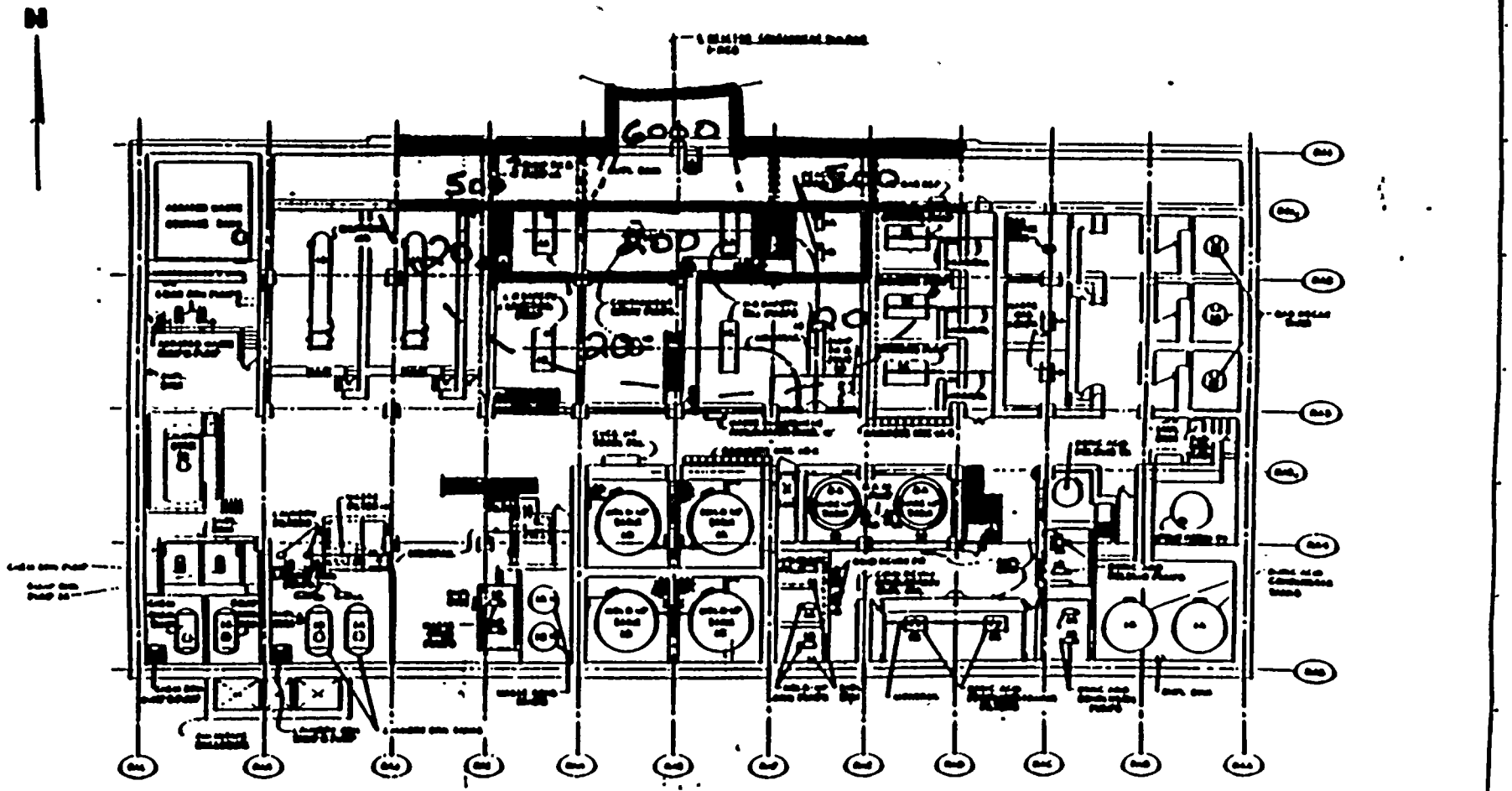
N



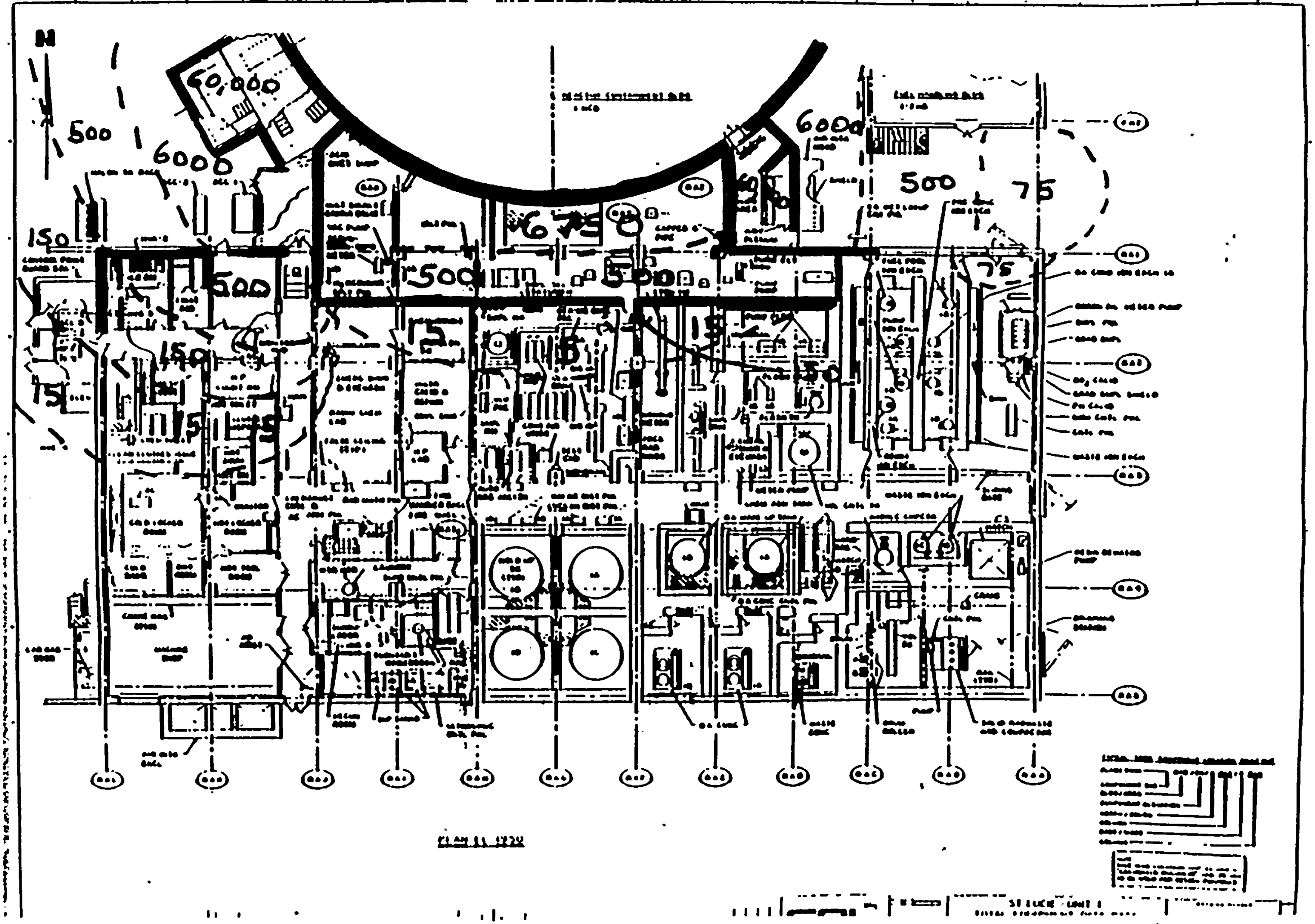


TIME · 1400 - 1600

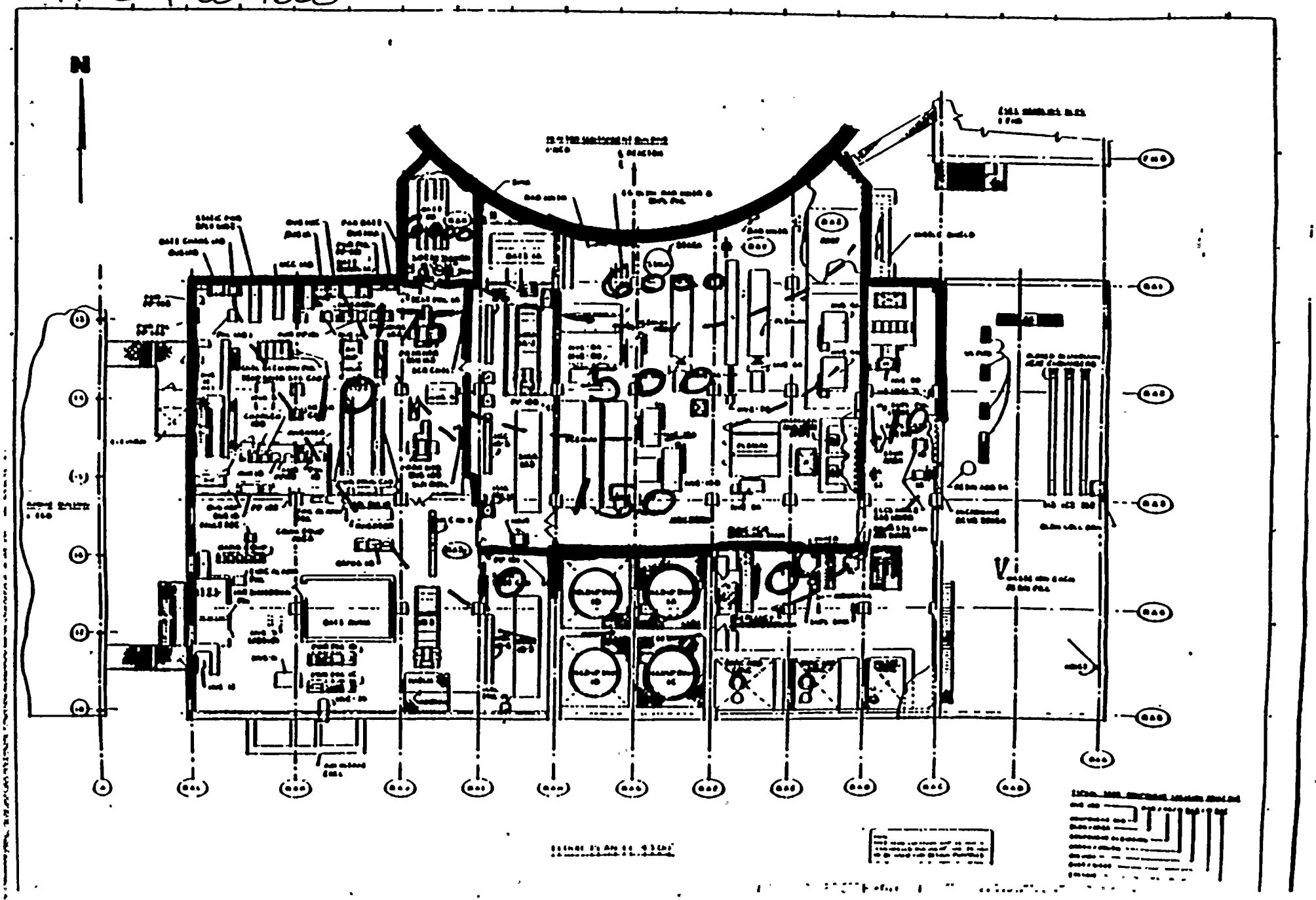
ALL SHEETS
· DOSE RATES IN MR/HR B & (OPEN WINDOW = CLOSED)
  ★ EXCEPT FOR STEAM TRESTLE
· CONTAMINATIONS (CPM) = ACTUAL, EXCEPT STEAM TRESTLE
· AIR ACTIVITY = ACTUAL, EXCEPT FOR STEAM TRESTLE



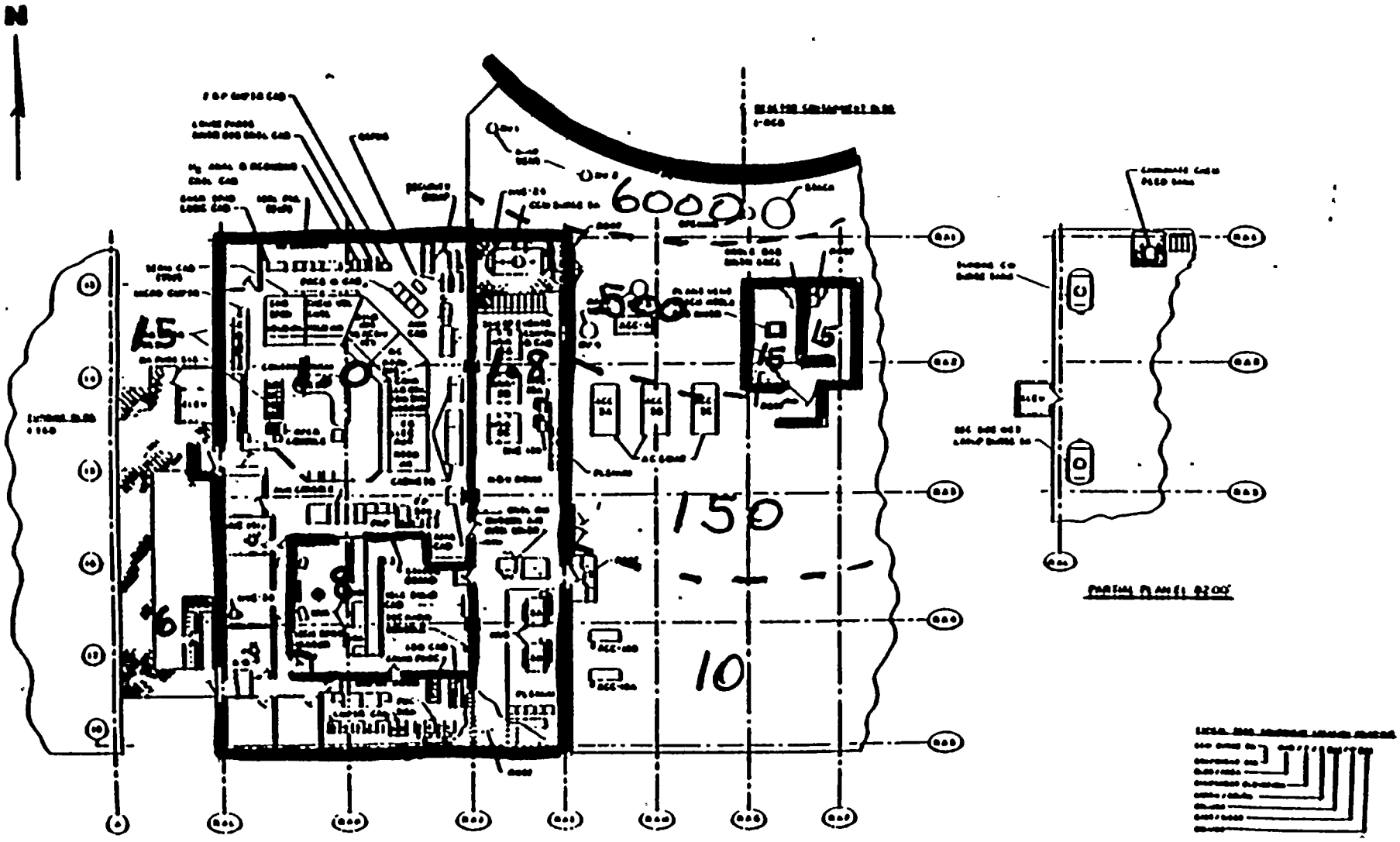
TIME: 1400-1600



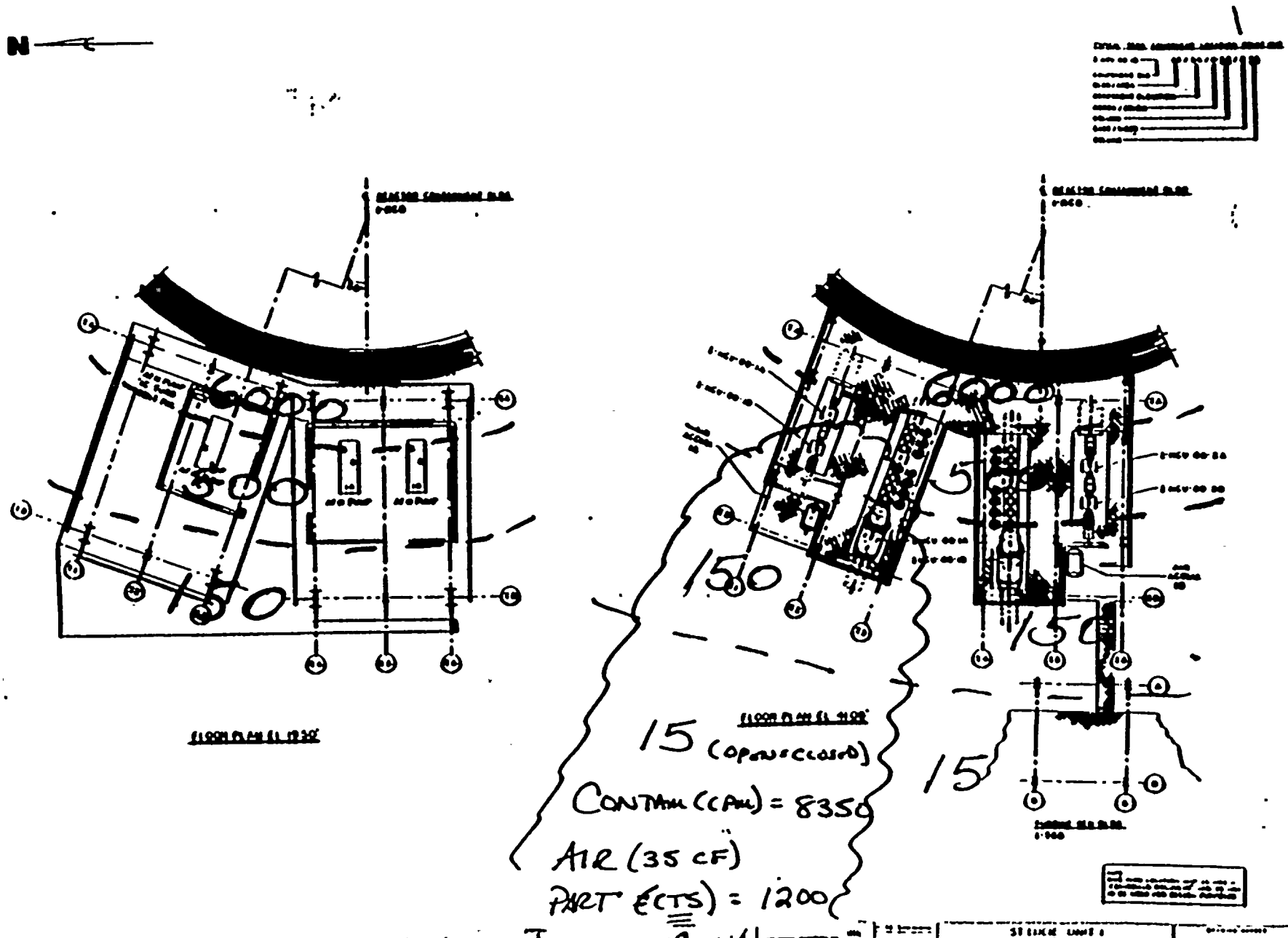
TIME: 1400-1600



TIME: 1400-1600

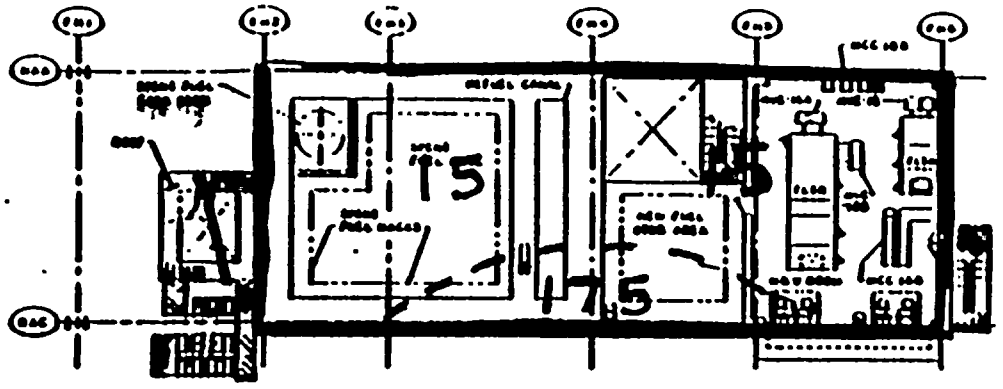


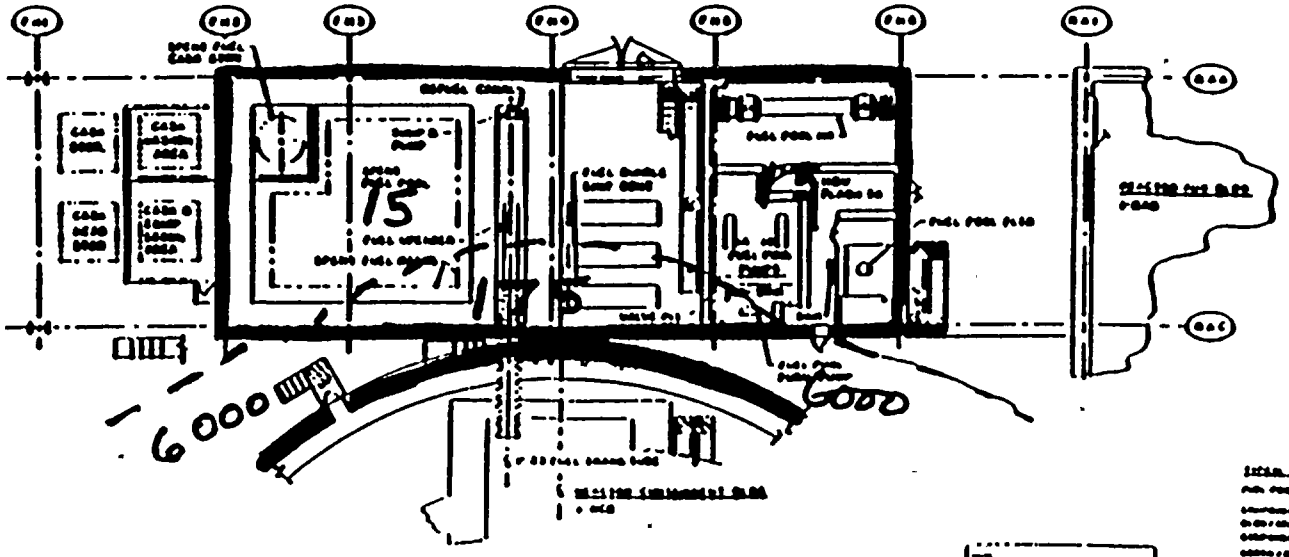
TIME: 1400-1600



TIME: 1400-1600

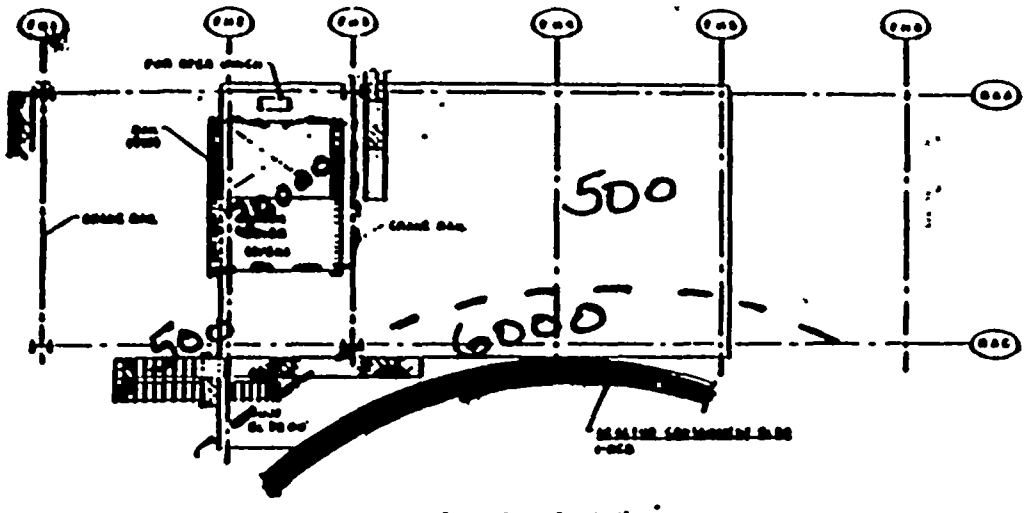
N

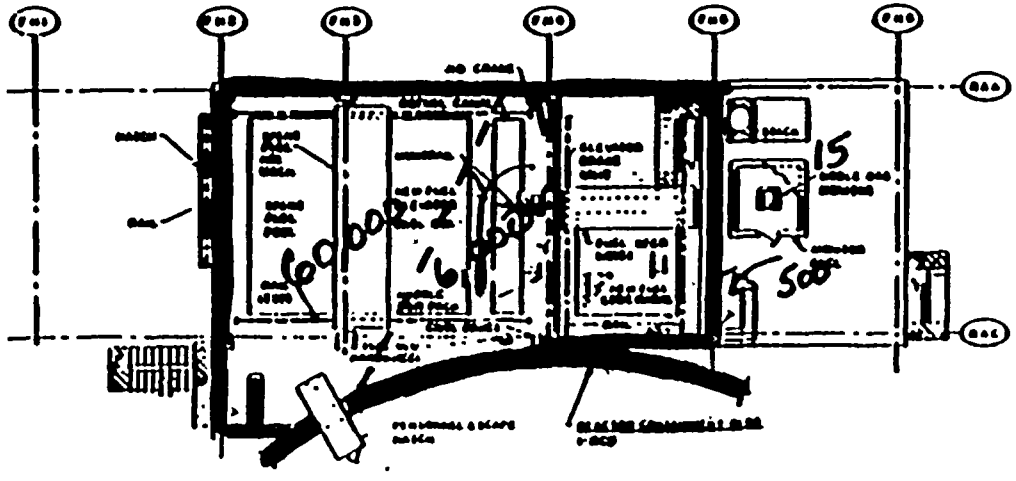




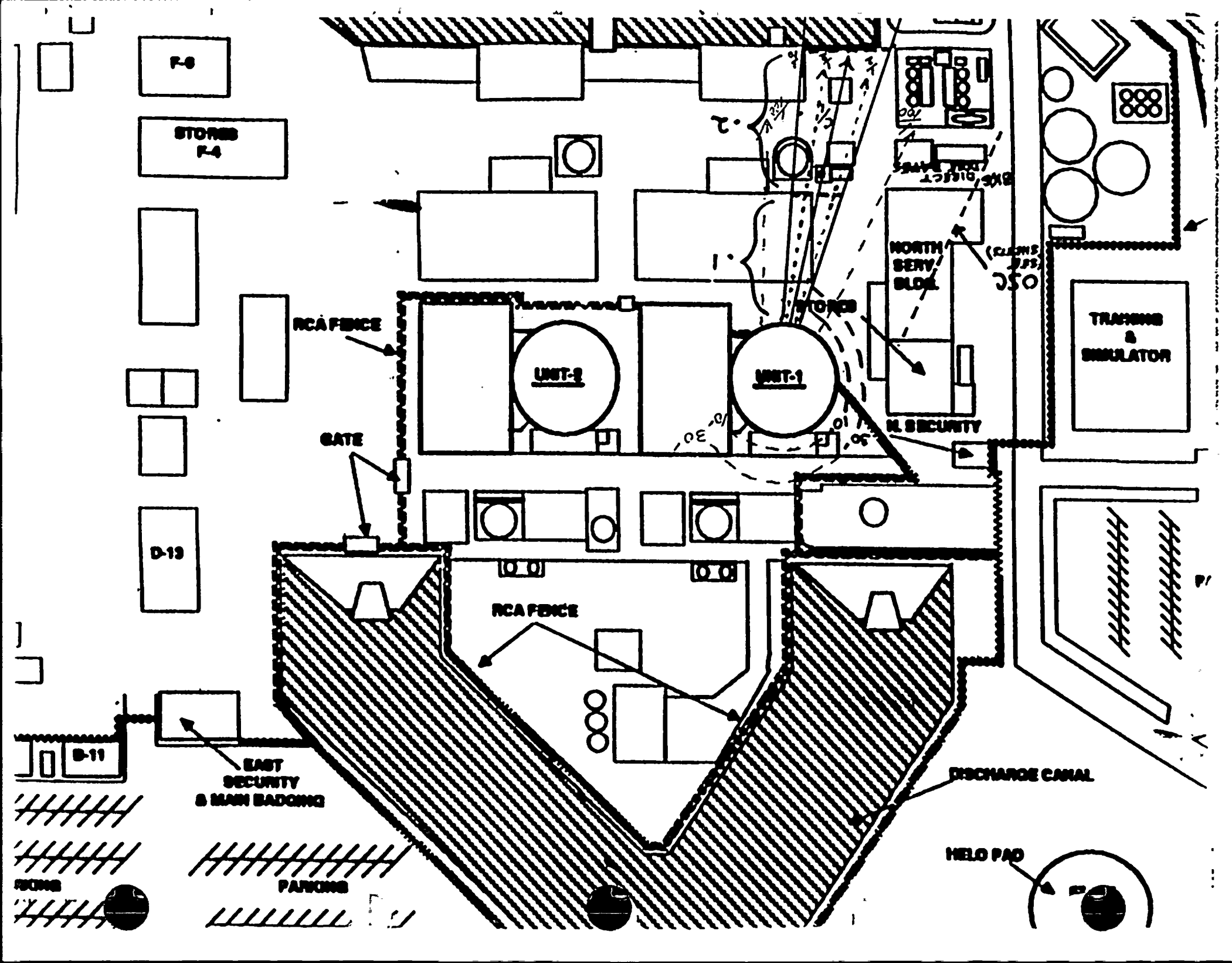
TIME: 1400-1600

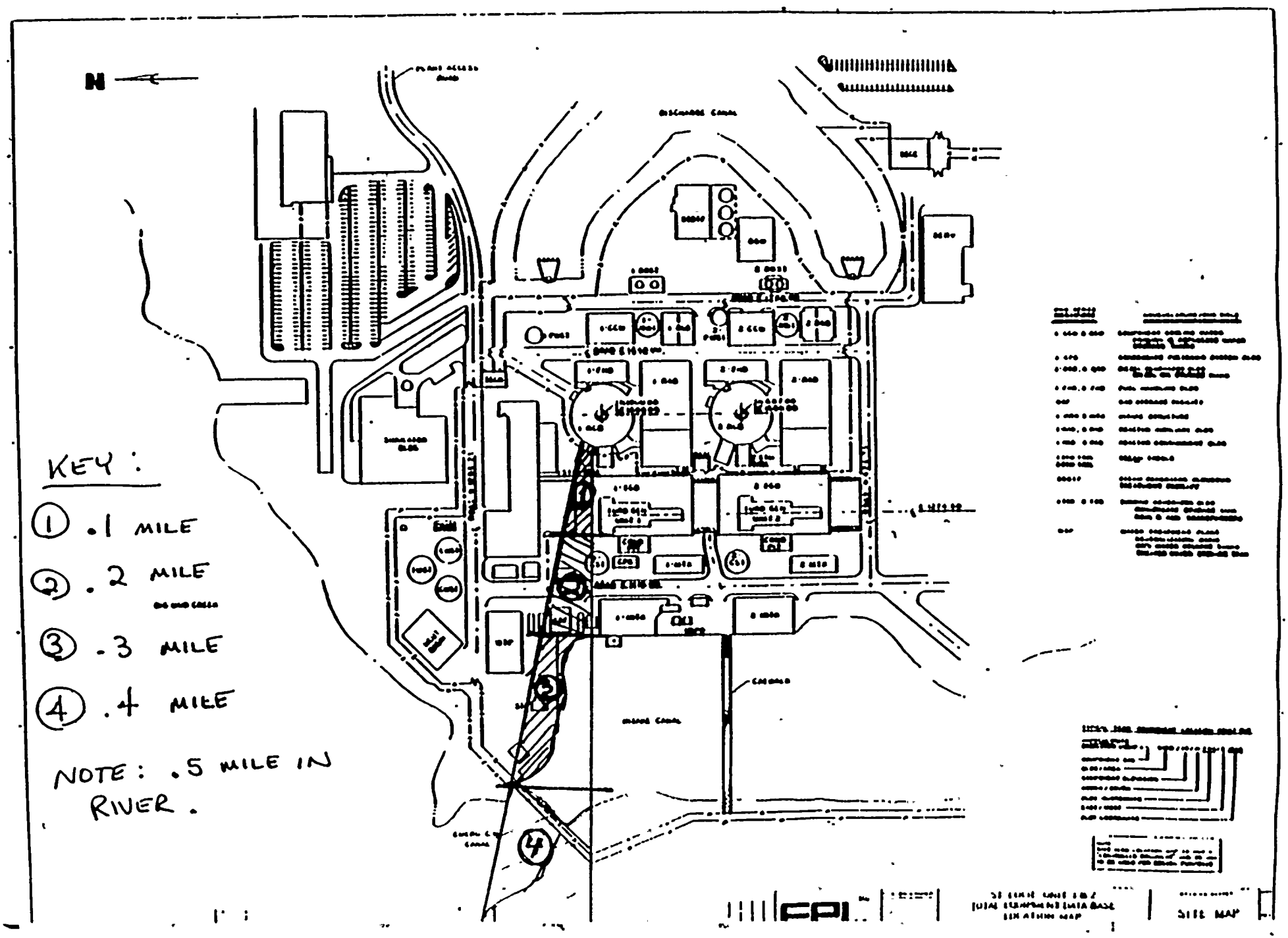
N





7.4.2 ONSITE PLUME MAP


F-6
STORES
F-4
RCA FENCE
UNIT-2
UNIT-1
GATE
D-19
B-11
EAST
SECURITY
& MAIN BADGING
PARKING
RCA FENCE
NORTH
SERV.
BLDG.
STORES
N. SECURITY
TRAINING
&
SIMULATOR
DISCHARGE CANAL
HELO PAD


N
KEY:
① .1 MILE
② .2 MILE
③ .3 MILE
④ .4 MILE
NOTE: .5 MILE IN RIVER.
SITE MAP

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/82 EXERCISE

TIME: 12:00

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	2849	26381	263700	791075	2636888
CART. GROSS TOTAL CNTS, 5 MIN	counts	[illegible]	231341	2309910	6926730	2308400
IODINE AIRBORNE CONC.	uCi/cc	6.00E-08	6.00E-07	6.00E-06	1.80E-05	6.00E-05
OPEN WINDOW @ HEAD HT.	mR/hr	40.2	402.0	1006.1	1206.1	4020.4
CLOSED WINDOW @ HEAD HT.	mR/hr	26.8	268.0	670.1	804.1	2680.3
CART. GROSS TOTAL CNTS, 1 MIN	counts			461782	1385146	4616920
GROUND DEPOSITION	uCi/cm2	1.34E-06	1.34E-05	3.35E-05	4.02E-05	1.34E-04
SMEARABLE CONTAMINATION	dpm/100cm2	298	2975	7436	8925	29751

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	1330	13196	131856	395543
CART. GROSS TOTAL CNTS, 5 MIN	counts	12042	115920	1154706	3463115
IODINE AIRBORNE CONC.	uCi/cc	3.00E-08	3.00E-07	3.00E-06	8.99E-06
OPEN WINDOW @ HEAD HT.	mR/hr	20.1	201.0	502.6	603.1
CLOSED WINDOW @ HEAD HT.	mR/hr	13.4	134.0	335.0	402.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			230941	692623
GROUND DEPOSITION	uCi/cm2	6.70E-07	6.70E-06	1.68E-05	2.01E-05
SMEARABLE CONTAMINATION	dpm/100cm2	149	1488	3719	4463

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	276	2649	26381
CART. GROSS TOTAL CNTS, 5 MIN	counts	2808	23684	231941
IODINE AIRBORNE CONC.	uCi/cc	6.00E-09	6.00E-08	6.00E-07
OPEN WINDOW @ HEAD HT.	mR/hr	4.0	40.2	100.5
CLOSED WINDOW @ HEAD HT.	mR/hr	2.7	26.8	67.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	1.34E-07	1.34E-06	3.35E-06
SMEARABLE CONTAMINATION	dpm/100cm2	30	298	744

OSC AREA

DOSE RATES (MR/HR)		CONTAM	AIR
OSC	1.0 (OPEN = CLOSED) (5000 COUNTS ON Rm. 14)	ACTUAL	ACTUAL
NORTH ENTRANCE FOYER	.1 (BKG ON 2M. 14)	ACTUAL	ACTUAL
SOUTH ENTRANCE	1.2 (1300 CTS)	ACTUAL N SERVICE	ACTUAL

ROADWAY BET. CTMT & MAINT. BLDG

DIST. FROM CTMT.	DOSE RATE (MR/HR)	CONTAM ACTUAL	AIR ACTUAL
5'	500	↓	↓
10'	200		
30'	20		

1/100 C/L PLUME LINE 6 MR/HR (CLOSED · OPEN

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 12:15

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	001	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	263700	527367	791075	1318450	2636868
CART. GROSS TOTAL CNTS, 5 MIN	counts	[illegible]	4617320	6925730	11542650	23084600
IODINE AIRBORNE CONC.	uCi/cc	6.00E-06	1.20E-05	1.80E-05	3.00E-05	6.00E-05
OPEN WINDOW @ HEAD HT.	mR/hr	402.0	804.1	1206.1	2010.2	4020.4
CLOSED WINDOW @ HEAD HT.	mR/hr	268.0	536.1	804.1	1340.1	2680.3
CART. GROSS TOTAL CNTS, 1 MIN	counts	461782	923464	1385146	2308510	4616920
GROUND DEPOSITION	uCi/cm2	1.47E-05	4.02E-05	7.37E-05	1.07E-04	2.68E-04
SMEARABLE CONTAMINATION	dpm/100cm2	3279	8925	16363	23801	59502

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	131866	263700	395543	659231
CART. GROSS TOTAL CNTS, 5 MIN	counts	1154705	2308810	3463118	5771825
IODINE AIRBORNE CONC.	uCi/cc	3.00E-06	6.00E-06	8.99E-06	1.50E-05
OPEN WINDOW @ HEAD HT.	mR/hr	201.0	402.0	603.1	1005.1
CLOSED WINDOW @ HEAD HT.	mR/hr	134.0	268.0	402.0	670.1
CART. GROSS TOTAL CNTS, 1 MIN	counts	230941	461782	692823	1154305
GROUND DEPOSITION	uCi/cm2	7.37E-06	2.01E-05	3.69E-05	5.36E-05
SMEARABLE CONTAMINATION	dpm/100cm2	1636	4463	8181	11900

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	26361	52750	79118
CART. GROSS TOTAL CNTS, 5 MIN	counts	231941	462152	693029
IODINE AIRBORNE CONC.	uCi/cc	6.00E-07	1.20E-06	1.80E-06
OPEN WINDOW @ HEAD HT.	mR/hr	40.2	80.4	120.6
CLOSED WINDOW @ HEAD HT.	mR/hr	26.8	53.6	80.4
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	1.47E-06	4.02E-06	7.37E-06
SMEARABLE CONTAMINATION	dpm/100cm2	327	893	1636

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN = 35 CFM

OSC AREA

	Dose Rates (MR/HR)	CONTAM	AIR
OSC	1.0 (open=closed) (5000 cts. on RM-14)	ACTUAL	ACTUAL
NORTH ENTRANCE FOYER	5.1 (Bkg on RM 14)	ACTUAL	ACTUAL
SOUTH ENTRANCE	1.5 (7500 CTS)	ACTUAL	ACTUAL

N SERVICE ROADWAY BET. CTMT & MAINT BLDG.

DIST FROM CTMT	DOSE RATE (MR/HR)	CONTAM.	AIR
5'	550	ACTUAL	ACTUAL
10'	210	↓	↓
30'	25		

1/100 C/L LINE 8.0 MR/HR (CLOSED = OPEN)

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 12:30

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	135076	270141	405206	675334	1350656
CART. GROSS TOTAL CNTS, 5 MIN	counts	[illegible]	[illegible]	[illegible]	[illegible]	[illegible]
IODINE AIRBORNE CONC.	uCi/cc	3.07E-06	6.14E-06	9.21E-06	1.54E-05	3.07E-05
OPEN WINDOW @ HEAD HT.	mR/hr	213.0	426.0	639.0	1065.0	2130.0
CLOSED WINDOW @ HEAD HT.	mR/hr	142.0	284.0	426.0	710.0	1420.0
CART. GROSS TOTAL CNTS, 1 MIN	counts	236580	473060	709540	1182499	2364899
GROUND DEPOSITION	uCi/cm2	2.18E-05	5.44E-05	9.50E-05	1.43E-04	3.39E-04
SMEARABLE CONTAMINATION	dpm/100cm2	4849	12078	21092	31682	75264

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	67544	135076	202609	337673
CART. GROSS TOTAL CNTS, 5 MIN	counts	591700	[illegible]	[illegible]	[illegible]
IODINE AIRBORNE CONC.	uCi/cc	1.54E-06	3.07E-06	4.61E-06	7.68E-06
OPEN WINDOW @ HEAD HT.	mR/hr	106.5	213.0	319.5	532.5
CLOSED WINDOW @ HEAD HT.	mR/hr	71.0	142.0	213.0	355.0
CART. GROSS TOTAL CNTS, 1 MIN	counts		236580	354820	591300
GROUND DEPOSITION	uCi/cm2	1.09E-05	2.72E-05	4.75E-05	7.14E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2424	6039	10546	15841

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	13518	27025	40531
CART. GROSS TOTAL CNTS, 5 MIN	counts	118740	[illegible]	[illegible]
IODINE AIRBORNE CONC.	uCi/cc	3.07E-07	6.14E-07	9.21E-07
OPEN WINDOW @ HEAD HT.	mR/hr	21.3	42.6	63.9
CLOSED WINDOW @ HEAD HT.	mR/hr	14.2	28.4	42.6
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.18E-06	5.44E-06	9.50E-06
SMEARABLE CONTAMINATION	dpm/100cm2	485	1208	2109

OSC AREA

	DOSE RATES (MR/HR)	CONTAM	AIR
OSC	.6 (3000 CTS ON RM 14)	ACTUAL	ACTUAL
N. ENTRANCE FOYER	<.1 (BKG ON RM 14)	ACTUAL	ACTUAL
S. ENTRANCE	1.0 (OPEN/CLOSED) (5000 CTS)	ACTUAL	ACTUAL

ROADWAY BET. CTMT & N. SERVICE ~~MAINT.~~ BLDG.

DIST. FROM CTMT	DOSE RATE (MR/HR)	CONTAM	AIR
5'	550	ACTUAL	ACTUAL
10'	210		
30'	25		

1/100 C/L LINE A MR/HR (CLOSED = OPEN)

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 12:45

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	54864	109915	164867	274770	549529
CART. GROSS TOTAL CNTS, 5 MIN	counts	[illegible]	962891	1442597	2406526	4811156
IODINE AIRBORNE CONC.	uCi/cc	1.25E-06	2.50E-06	3.75E-06	6.25E-06	1.25E-05
OPEN WINDOW @ HEAD HT.	mR/hr	49.9	175.7	299.8	449.3	898.6
CLOSED WINDOW @ HEAD HT.	mR/hr	59.9	119.8	179.7	299.5	599.1
CART. GROSS TOTAL CNTS, 1 MIN	counts		192528	288739	481166	962231
GROUND DEPOSITION	uCi/cm2	2.48E-05	6.04E-05	1.04E-04	1.58E-04	3.09E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5514	13408	23096	35006	81913

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	27488	54964	82440	137391
CART. GROSS TOTAL CNTS, 5 MIN	counts	241033	481298	720988	1203164
IODINE AIRBORNE CONC.	uCi/cc	6.25E-07	1.25E-06	1.87E-06	3.12E-06
OPEN WINDOW @ HEAD HT.	mR/hr	44.9	89.9	134.8	224.7
CLOSED WINDOW @ HEAD HT.	mR/hr	30.0	59.9	89.9	149.8
CART. GROSS TOTAL CNTS, 1 MIN	counts				240633
GROUND DEPOSITION	uCi/cm2	1.24E-06	3.02E-06	5.20E-06	7.88E-06
SMEARABLE CONTAMINATION	dpm/100cm2	2757	6704	11543	17503

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	5507	11002	16496
CART. GROSS TOTAL CNTS, 5 MIN	counts	48507	96713	144820
IODINE AIRBORNE CONC.	uCi/cc	1.25E-07	2.50E-07	3.75E-07
OPEN WINDOW @ HEAD HT.	mR/hr	9.0	18.0	27.0
CLOSED WINDOW @ HEAD HT.	mR/hr	6.0	12.0	18.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.48E-06	6.04E-06	1.04E-05
SMEARABLE CONTAMINATION	dpm/100cm2	551	1341	2309

OSC AREA

	DOSE RATE (mR/hr)	CONTAM	AIR
OSC	.3 (OPEN=CLOSED)	ACTUAL	ACTUAL

(1500 CTS ON RM 14)

N. ENTRANCE FOYER	<.1	ACTUAL ↓	ACTUAL ↓
S. ENTRANCE	.4 (2000 CTS)		

ROADWAY BET. CTMT & N SERVICE BLDG.

5'	500	ACTUAL ↓	ACTUAL ↓
10'	200		
30'	20		

1/100 C/L LINE 2.0 mR/HR (CLOSED=OPEN)

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 13:00

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	12883	25254	37674	63116	126220
CART. GROSS TOTAL CNTS, 5 MIN	counts	[illegible]	221473	331958	552932	1105365
IODINE AIRBORNE CONC.	uCi/cc	2.87E-07	5.74E-07	8.61E-07	1.43E-06	2.87E-06
OPEN WINDOW @ HEAD HT.	mR/hr	21.4	42.9	64.3	107.2	214.5
CLOSED WINDOW @ HEAD HT.	mR/hr	14.3	28.6	42.9	71.5	142.9
CART. GROSS TOTAL CNTS, 1 MIN	counts					221073
GROUND DEPOSITION	uCi/cm2	2.56E-05	6.18E-05	1.06E-04	1.61E-04	3.76E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5572	13725	23582	35799	83499

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	6322	12633	18943	31564
CART. GROSS TOTAL CNTS, 5 MIN	counts	56745	113269	169950	278715
IODINE AIRBORNE CONC.	uCi/cc	1.43E-07	2.87E-07	4.30E-07	7.17E-07
OPEN WINDOW @ HEAD HT.	mR/hr	10.7	21.4	32.2	53.6
CLOSED WINDOW @ HEAD HT.	mR/hr	7.2	14.3	21.4	35.7
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.28E-05	3.09E-05	5.31E-05	8.06E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2696	6662	11781	17900

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	1274	2536	3798
CART. GROSS TOTAL CNTS, 5 MIN	counts	11540	22687	33646
IODINE AIRBORNE CONC.	uCi/cc	2.87E-06	5.74E-06	8.61E-06
OPEN WINDOW @ HEAD HT.	mR/hr	2.2	4.3	6.4
CLOSED WINDOW @ HEAD HT.	mR/hr	1.4	2.9	4.3
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.56E-06	6.18E-06	1.06E-05
SMEARABLE CONTAMINATION	dpm/100cm2	567	1372	2356

OSC AREA

	DOSE RATE (MR/HR)	CONTAM AIR
OSC	.1 (OPEN = CLOSED) (500 CTS)	ACTUAL ACTUAL
N. ENTRANCE	<.1 (BKG.)	ACTUAL ACTUAL
S ENTRANCE	.2 (1000 CTS)	ACTUAL ACTUAL

ROADWAY BET. CNMT & N SERVICE BLDG.

	DOSE RATE (MR/HR)	CONTAM. AIR
5'	500	ACTUAL ACTUAL
10'	200	↓ ↓
30'	20	

1/100 C/L LINE .4 MR/HR (CLOSED = OPEN)

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 13:15

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	001	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	1878	3744	5610	9342	18671
CART. GROSS TOTAL CNTS, 5 MIN	counts	[illegible]	35170	49505	82175	163651
IODINE AIRBORNE CONC.	uCi/cc	4.24E-08	8.49E-08	1.27E-07	2.12E-07	4.24E-07
OPEN WINDOW @ HEAD HT.	mR/hr	3.3	6.6	9.9	16.5	32.9
CLOSED WINDOW @ HEAD HT.	mR/hr	2.2	4.4	6.6	11.0	21.9
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.20E-05	1.06E-04	1.62E-04	3.77E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5697	13773	23535	35921	83743

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	945	1878	2811	4677
CART. GROSS TOTAL CNTS, 5 MIN	counts	[illegible]	18836	25009	41336
IODINE AIRBORNE CONC.	uCi/cc	2.12E-08	4.24E-08	6.36E-08	1.06E-07
OPEN WINDOW @ HEAD HT.	mR/hr	1.7	3.3	4.9	8.2
CLOSED WINDOW @ HEAD HT.	mR/hr	1.1	2.2	3.3	5.5
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.29E-05	3.10E-05	5.32E-05	8.09E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2848	6887	11818	17961

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	199	365	572
CART. GROSS TOTAL CNTS, 5 MIN	counts	2134	3710	5401
IODINE AIRBORNE CONC.	uCi/cc	4.24E-09	8.49E-09	1.27E-08
OPEN WINDOW @ HEAD HT.	mR/hr	0.3	0.7	1.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.2	0.4	0.7
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.20E-06	1.06E-05
SMEARABLE CONTAMINATION	dpm/100cm2	570	1377	2364

OSC AREA

OSC	DOSE RATES (MR/HR) ACTUAL	CONTAM ACTUAL	AIR ACTUAL
N ENTRANCE	↓	↓	↓
S. ENTRANCE			

ROADWAY BET. CTMT & N. SERVICE BLDG.

	DOSE RATE (MR/HR)	CONTAM ACTUAL	AIR ACTUAL
5'	500	↓	↓
10'	180		
30'	185		

1/100 C/L LINE < 0.1 MR/NR (CLOSED + OPEN)

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 13:30

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	55	99	142	228	445
CART. GROSS TOTAL CNTS, 5 MIN	counts	875	1258	1697	2364	4269
IODINE AIRBORNE CONC.	uCi/cc	9.84E-10	1.97E-09	2.95E-09	4.92E-09	9.84E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.8	1.4	2.1	4.0	7.9
CLOSED WINDOW @ HEAD HT.	mR/hr	0.5	1.1	1.6	2.6	5.3
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.21E-05	1.07E-04	1.62E-04	3.77E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5703	13786	23653	35950	83601

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	34	55	77	120
CART. GROSS TOTAL CNTS, 5 MIN	counts	669	879	1098	1447
IODINE AIRBORNE CONC.	uCi/cc	4.92E-10	9.84E-10	1.48E-09	2.46E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.4	0.8	1.2	2.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.3	0.5	0.8	1.3
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.28E-05	3.10E-05	5.33E-05	8.10E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2851	6909	11826	17975

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	16	21	25
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.2
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.21E-06	1.07E-05
SMEARABLE CONTAMINATION	dpm/100cm2	570	1379	2365

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

ROADWAY BET. CTMT & N SERVICE BLDG.

	DOSE RATE (MR/HR)	CONTAM	AIR
5'	500	ACTUAL	ACTUAL
10'	180		
30'	18		

1/100 C/L LINE ACTUAL

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 13:45

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	22	32	42	62	113
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	676	784	941	1391
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	4.58E-10	6.87E-10	1.14E-09	2.29E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.2	0.4	0.6	1.0	1.9
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.3	0.4	0.6	1.3
CART GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.21E-05	1.07E-04	1.62E-04	3.78E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5704	13788	23857	35957	83815

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	17	22	27	37
CART GROSS TOTAL CNTS, 5 MIN	counts	500	500	632	720
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	3.43E-10	5.72E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.3	0.5
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.3
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.28E-05	3.11E-05	5.33E-05	8.10E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2852	6894	11828	17979

ROADWAY BET. CTMT + N. SERVICE BLDG.

	DOSERATE (mR/HR)	CONTAM	AIR
5'	490	ACTUAL	ACTUAL
10'	180	↓	↓
30'	18		

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	13	14	15
CART GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.1
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.21E-06	1.07E-05
SMEARABLE CONTAMINATION	dpm/100cm2	570	1379	2366

1/100 C/L LINE ACTUAL (CLOSED = OPEN)

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 14:00

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	21	31	40	59	106
CART. GROSS TOTAL CNTS, 5 MIN	counts	600	655	747	911	1323
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	4.27E-10	6.41E-10	1.07E-09	2.14E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.2	0.4	0.6	0.9	1.8
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.3	0.4	0.6	1.2
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.21E-05	1.07E-04	1.62E-04	3.78E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5705	15791	23861	35964	83828

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	17	21	28	36
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	700
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	5.34E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.3	0.5
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.3
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.29E-05	3.11E-05	5.33E-05	8.10E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2853	6895	11830	17962

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	13	14	15
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.1
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.21E-06	1.07E-06
SMEARABLE CONTAMINATION	dpm/100cm2	571	1379	2386

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

ROADWAY BET. CTMT + N. SERVICE BLDG

	DOSE RATE (mR/HR)	CONTAM ACTUAL	AIR ACTUAL
5'	490	↓	↓
10'	180		
30'	18		

1/100 C/L LINE = ACTUAL (CLOSED · OPEN)

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 14:15

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	21	30	38	56	100
CART. GROSS TOTAL CNTS, 5 MIN	counts	600	654	732	880	1272
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	4.01E-10	6.01E-10	1.00E-09	2.00E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.2	0.4	0.6	0.9	1.8
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.4	0.6	1.2
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.21E-05	1.07E-04	1.62E-04	3.78E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5707	13799	23855	35971	83841

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	16	21	25	34
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	593
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	5.01E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.3	0.4
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.3
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.29E-05	3.11E-05	5.33E-05	8.10E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2863	6907	11832	17985

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	13	14	15
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.1
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.21E-06	1.07E-05
SMEARABLE CONTAMINATION	dpm/100cm2	571	1379	2366

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

ROADWAY BET. CTMT + N. SERVICE BLDG.

	DOSE RATES (MR/HR)	CONTAM	AIR
5'	480	ACTUAL ↓	ACTUAL ↓
10'	180		
30'	18		

1/100 C/L LINE = ACTUAL

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 14:30

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	20	28	36	52	93
CART. GROSS TOTAL CNTS, 5 MIN	counts	600	641	711	852	1205
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	3.66E-10	5.49E-10	9.15E-10	1.83E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.3	0.4	0.7	1.5
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.3	0.5	0.9
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.21E-05	1.07E-04	1.62E-04	3.78E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5708	13795	23868	36075	83661

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	16	20	24	32
CART. GROSS TOTAL CNTS, 5 MIN	counts	600	600	600	676
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	4.58E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.5
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.2
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.29E-05	3.11E-05	5.33E-05	8.10E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2954	6998	11834	17966

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	13	14	14
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.21E-06	1.07E-05
SMEARABLE CONTAMINATION	dpm/100cm2	571	1380	2367

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

ROADWAY BET. CTMT & N. SERVICE BLDG.

	DOSE RATES (mR/HR)	CONTAM. AIR
		ACTUAL ACTUAL
5'	475	↓ ↓
10'	180	
30'	18	

1/100 C/L LINE = ACTUAL

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 14:45

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	18	24	30	42	72
CART. GROSS TOTAL CNTS, 5 MIN	counts	600	600	650	754	1029
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	4.12E-10	6.87E-10	1.37E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.4	0.5	1.2
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.2	0.4	0.8
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.21E-05	1.07E-04	1.62E-04	3.78E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5708	13797	23670	36000	83960

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	15	18	21	27
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	532
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	3.43E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.3
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.1	0.1	0.2
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.29E-05	3.11E-05	5.33E-05	8.10E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2864	6908	11836	17990

ROADWAY BET. CTMT & N. SERVICE BLDG.

	DOSE RATES (mR/HR)	CONTAM.	AIR
5'	475	ACTUAL	ACTUAL
10'	180	↓	↓
30'	18		

1/100 C/L LINE = ACTUAL

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	13	13	14
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.21E-06	1.07E-05
SMEARABLE CONTAMINATION	dpm/100cm2	571	1360	2367

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 15:00

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	16	20	24	32	52
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	676	852
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	4.58E-10	9.15E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.3	0.5	1.1
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.2	0.4	0.7
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.22E-05	1.07E-04	1.62E-04	3.78E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5709	13799	23673	35984	83868

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	14	16	18	22
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.3
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.1	0.1	0.2
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.29E-05	3.11E-05	5.33E-05	8.10E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2855	6899	11836	17992

ROADWAY BET. CTMT & N. SERVICE BLDG.

	DOSE RATES (MR/HR)	CONTAM	AIR
5'	475	ACTUAL	ACTUAL
10'	175	↓	↓
30'	18		

1/100 C/L LINE = ACTUAL.

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	12	13	13
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.22E-06	1.07E-05
SMEARABLE CONTAMINATION	dpm/100cm2	571	1380	2367

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 15:15

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	14	16	18	22	32
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	676
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	4.58E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.3	0.5	0.9
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.3	0.6
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.22E-06	1.07E-04	1.62E-04	3.78E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5710	13800	23675	35987	83675

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	14	15	17
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.2
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.1	0.1	0.2
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.29E-06	3.11E-06	5.33E-06	8.11E-06
SMEARABLE CONTAMINATION	dpm/100cm2	2955	6900	11837	17994

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	12	12	13
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.22E-06	1.07E-05
SMEARABLE CONTAMINATION	dpm/100cm2	571	1380	2367

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

ROADWAY BET. CTMT - N. SERVICE BLDG.

	DOSE RATES (mR/HR)	CONTAM	AIR
5'	475	ACTUAL	ACTUAL
10'	175	↓	↓
30'	18		

1/100 C/L LINE = ACTUAL

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 15:30

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	14	16	18	22	32
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	676
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	4.58E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.3	0.5	0.9
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.3	0.6
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.22E-05	1.07E-04	1.62E-04	3.78E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5711	13801	23677	35991	83862

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	14	15	17
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.2
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.1	0.1	0.2
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.29E-05	3.11E-05	5.33E-05	8.11E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2855	6901	11838	17995

ROADWAY BET. CTMT & N. SERVICE BLDG.

	DOSE RATES (MR/HR)	CONTAM	AIR
5'	475	ACTUAL	ACTUAL
10'	175	↓	↓
30'	18		

1/100 C/L LINE = ACTUAL

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	12	12	13
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.22E-06	1.07E-05
SMEARABLE CONTAMINATION	dpm/100cm2	571	1380	2368

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN = 35 CFM

ST. LUCIE NUCLEAR PLANT
ONSITE PLUME DATA
02/12/92 EXERCISE

TIME: 15:45

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2	0.1
SECTOR		N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	14	16	18	22	32
CART. GROSS TOTAL CNTS, 5 MIN	counts	570	580	590	610	676
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	4.58E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.2	0.3	0.5	0.9
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.3	0.6
CART. GROSS TOTAL CNTS, 1 MIN	counts					
GROUND DEPOSITION	uCi/cm2	2.57E-05	6.22E-05	1.07E-04	1.62E-04	3.78E-04
SMEARABLE CONTAMINATION	dpm/100cm2	5711	13803	23679	35994	83999

HALF CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3	0.2
SECTOR		N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	14	15	17
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.2
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.1	0.1	0.2
CART. GROSS TOTAL CNTS, 1 MIN	counts				
GROUND DEPOSITION	uCi/cm2	1.29E-05	3.11E-05	5.33E-05	8.11E-05
SMEARABLE CONTAMINATION	dpm/100cm2	2858	6901	11840	17997

TENTH CENTERLINE

MILES FROM SOURCE		0.5	0.4	0.3
SECTOR		N	N	N
PREFILTER GROSS COUNTS	cpm	12	12	13
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts			
GROUND DEPOSITION	uCi/cm2	2.57E-06	6.22E-06	1.07E-06
SMEARABLE CONTAMINATION	dpm/100cm2	571	1380	2366

OSC AREA

ALL AREAS ACTUAL MEASUREMENTS

ROADWAY BETWEEN CTMT. & N. SERVICE BLDG.

	DOSE RATES (MR/HR)	CONTAM. AIR
5'	475	ACTUAL
10'	175	
30'	18	ACTUAL

1/100 C/L LINE = ACTUAL

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

7.5 OFF-SITE RADIOLOGICAL DATA

This Scenario Section Contains the Following Information:

Section 7.5.1	FPL OFF-SITE MONITORING DATA
Section 7.5.2	OFF-SITE PLUME MAP
Section 7.5.3	STATE OF FLORIDA OFF-SITE MONITORING DATA

7.5.1 FPL OFF-SITE MONITORING DATA

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 12:15

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	263700	210962	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	2309610	1847226	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	6.00E-06	4.80E-06	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	402.0	321.6	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	268.0	214.4	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts	461782	369446									

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	131856	105487	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	1154705	923964	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	3.00E-06	2.40E-06	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	201.0	160.8	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	134.0	107.2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts	230941	184773									

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	26361	21107	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	231341	185173	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	6.00E-07	4.80E-07	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	40.2	32.2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	26.8	21.5	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 12:30

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	135076	106064	77055	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	1182899	946419	674967	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	3.07E-06	2.46E-06	1.75E-06	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	213.0	171.1	122.5	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	142.0	113.6	81.6	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts	236580	189284									

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	67544	54038	38533	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	591700	473460	337729	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	1.54E-06	1.23E-06	8.76E-07	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	106.5	85.2	61.2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	71.0	56.8	40.8	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13518	10617	7716	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	118740	93092	67946	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	3.07E-07	2.46E-07	1.75E-07	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	21.3	17.1	12.2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	14.2	11.4	8.2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 12:45

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	54964	43973	39474	44036	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	481866	385982	345966	385904	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	1.25E-06	1.00E-06	8.97E-07	1.00E-06	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	89.9	71.9	64.8	63.4	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	59.9	47.9	43.2	42.3	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	27488	21993	19743	22024	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	241033	192998	173283	193202	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	6.25E-07	5.00E-07	4.49E-07	5.01E-07	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	44.9	36.0	32.4	31.7	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	30.0	24.0	21.6	21.2	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	5507	4408	3958	4414	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	48807	38985	35047	39040	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	1.25E-07	1.00E-07	8.97E-08	1.00E-07	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	9.0	7.2	6.5	6.4	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	6.0	4.8	4.3	4.2	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 13:00

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	12633	10109	16067	22562	27527	21107	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	110966	88889	141055	197909	241978	185173	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	2.87E-07	2.30E-07	3.65E-07	5.13E-07	6.26E-07	4.80E-07	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	21.4	17.2	27.8	33.6	43.5	32.2	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	14.3	11.4	18.2	22.4	29.0	21.5	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	6322	5060	8040	11287	13770	10560	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	55743	44695	70777	99205	120939	92838	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	1.43E-07	1.15E-07	1.83E-07	2.56E-07	3.13E-07	2.40E-07	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	10.7	8.6	13.7	16.8	21.8	16.1	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	7.2	5.7	9.1	11.2	14.5	10.7	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	1274	1022	1618	2267	2764	2122	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	11549	9330	14655	20941	24585	18967	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	2.87E-08	2.30E-08	3.65E-08	5.13E-08	6.26E-08	4.80E-08	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	2.2	1.7	2.7	3.4	4.4	3.2	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	1.4	1.2	1.8	2.3	2.9	2.2	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 13:15

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	1878	1505	3699	9187	14106	10817	15604	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	16906	13568	32761	80817	123861	95092	136997	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	4.24E-08	3.39E-08	8.38E-08	2.09E-07	3.20E-07	2.46E-07	3.55E-07	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	3.3	2.6	6.6	14.2	23.1	17.1	24.5	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	2.2	1.8	4.4	9.5	15.4	11.4	16.3	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	945	758	1856	4599	7059	5415	7808	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	8666	7034	16641	40659	62190	47796	68749	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	2.12E-08	1.70E-08	4.19E-08	1.04E-07	1.60E-07	1.23E-07	1.77E-07	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	1.7	1.3	3.3	7.1	11.6	8.6	12.2	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	1.1	0.9	2.2	4.7	7.7	5.7	8.2	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	199	161	381	929	1421	1093	1571	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	2134	1807	8728	8532	12836	9959	14150	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	4.24E-09	3.39E-09	8.38E-09	2.09E-08	3.20E-08	2.46E-08	3.55E-08	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.3	0.3	0.7	1.4	2.3	1.7	2.5	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.2	0.2	0.4	1.0	1.5	1.1	1.6	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 13:30

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	55	47	557	2119	5746	4408	7998	12852	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	679	603	5273	18946	50696	38965	70416	112910	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	9.84E-10	7.87E-10	1.24E-08	4.79E-08	1.30E-07	1.00E-07	1.82E-07	2.92E-07	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.5	0.5	1.0	3.4	9.7	7.2	13.0	19.5	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.5	0.4	0.7	2.3	6.5	4.8	8.7	13.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	34	29	285	1066	2879	2210	4005	6432	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	680	652	2906	9723	25699	19743	35458	56705	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	4.92E-10	3.94E-10	6.20E-09	2.40E-08	6.52E-08	5.00E-08	9.08E-08	1.46E-07	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.4	0.3	0.5	1.7	4.9	3.6	6.5	9.7	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.3	0.2	0.3	1.1	3.3	2.4	4.3	6.5	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	16	15	67	223	585	452	811	1296	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	977	2345	5520	4349	7482	11741	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	1.24E-09	4.79E-09	1.30E-08	1.00E-08	1.82E-08	2.92E-08	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.3	1.0	0.7	1.3	2.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.2	0.7	0.5	0.9	1.3	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 13:45

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	22	20	25	324	1329	1022	3261	6589	10101	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	3227	12029	9339	29948	58078	88822	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	7.08E-09	2.99E-08	2.30E-08	7.39E-08	1.50E-07	2.29E-07	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.2	0.2	0.2	0.5	2.3	1.7	5.5	10.3	16.9	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.2	0.4	1.6	1.2	3.7	6.9	10.9	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	17	16	18	168	670	517	1637	3301	5056	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	1864	6265	4919	14729	29289	44661	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	3.54E-09	1.50E-08	1.15E-08	3.69E-08	7.48E-08	1.15E-07	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.3	1.2	0.9	2.7	5.2	8.2	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.2	0.8	0.6	1.8	3.5	5.4	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	13	13	43	144	113	337	670	1021	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	773	1653	1384	3345	6258	9332	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	7.08E-10	2.99E-09	2.30E-09	7.39E-09	1.50E-08	2.29E-08	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.1	0.2	0.2	0.6	1.0	1.6	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.2	0.1	0.4	0.7	1.1	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 14:00

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	21	20	15	19	207	161	758	2688	5180	8817	7441
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	2205	1807	7033	23926	45740	77581	65537
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	4.43E-09	3.39E-09	1.70E-08	6.08E-08	1.18E-07	2.00E-07	1.69E-07
OPEN WINDOW @ HEAD HT.	mR/hr	0.2	0.2	0.1	0.1	0.4	0.3	1.3	4.4	8.7	14.1	11.8
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.1	0.2	0.2	0.9	2.9	5.8	9.4	7.9
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	17	16	13	16	109	87	385	1350	2596	4414	3727
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	1352	1153	3767	12213	23120	39040	33018
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	2.21E-09	1.70E-09	8.48E-09	3.04E-08	5.88E-08	1.00E-07	8.45E-08
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.1	0.2	0.1	0.7	2.2	4.3	7.0	5.9
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.1	0.1	0.1	0.4	1.5	2.9	4.7	3.9
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	13	12	13	31	27	87	280	529	892	755
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	670	631	1153	2543	5024	8208	7004
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	4.43E-10	3.39E-10	1.70E-09	6.08E-09	1.18E-08	2.00E-08	1.69E-08
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.4	0.9	1.4	1.2
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.3	0.6	0.9	0.8
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 14:15

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	21	19	15	14	17	15	122	627	2114	4522	3817
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	1466	5880	18906	39982	33613
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	2.51E-09	1.40E-08	4.78E-08	1.03E-07	8.65E-08
OPEN WINDOW @ HEAD HT.	mR/hr	0.2	0.2	0.1	0.0	0.1	0.1	0.2	1.0	3.7	7.5	6.3
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.1	0.1	0.1	0.7	2.4	5.0	4.2
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	16	16	13	13	14	14	67	319	1063	2267	1915
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	993	3190	9703	20241	17156
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	1.25E-09	6.99E-09	2.39E-08	5.13E-08	4.33E-08
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.1	0.0	0.1	0.5	1.8	3.7	3.1
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.1	0.4	1.2	2.5	2.1
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	13	12	12	12	12	23	73	222	463	393
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	1038	2341	4448	3831
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	1.40E-09	4.78E-09	1.03E-08	8.65E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.4	0.8	0.6
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.3	0.5	0.4
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 14:30

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	20	18	15	14	13	13	15	103	495	1847	1560
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	1295	4727	16583	14053
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	2.07E-09	1.10E-08	4.17E-08	3.52E-08
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.0	0.0	0.0	0.1	0.2	0.9	3.1	2.6
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.1	0.6	2.1	1.8
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	16	15	13	13	13	12	13	57	253	929	786
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	806	2614	8532	7277
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	1.03E-09	5.49E-09	2.09E-08	1.76E-08
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.1	0.4	1.6	1.3
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.3	1.1	0.9
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	13	12	12	12	12	12	21	60	195	167
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	923	2106	1855
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	1.10E-09	4.17E-09	3.52E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.3	0.3
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.2	0.2
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 14:45

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	18	17	14	13	13	13	13	14	83	433	368
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	1125	4189	3613
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	1.62E-09	9.58E-09	8.09E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.1	0.8	0.6
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.5	0.4
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	15	14	13	13	12	12	12	13	48	223	190
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	812	2345	2056
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	8.12E-10	4.79E-09	4.04E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.4	0.3
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.3	0.2
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	12	12	12	12	12	12	12	19	54	48
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	669	611
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	9.58E-10	8.09E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.1
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.1
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 15:00

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	16	15	14	13	13	13	13	12	14	74	65
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	1045	960
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	1.42E-09	1.20E-09
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.1
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.1
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	14	14	13	13	12	12	12	12	13	43	38
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	773	730
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	7.08E-10	5.98E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.1	0.1
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	12	12	12	12	12	12	12	12	12	18	17
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 15:15

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	14	14	13	13	13	13	13	12	12	13	13
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	13	13	13	12	12	12	12	12	13	13
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	12	12	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 15:30

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	14	14	13	13	13	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3 30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	13	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	12	12	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3 30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 15:45

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	14	14	13	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW ● HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW ● HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	13	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW ● HEAD HT.	mR/hr	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW ● HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	12	12	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW ● HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW ● HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE ● 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

ST. LUCIE NUCLEAR PLANT
02/12/92 EXERCISE PLUME DATA

LUDLUM 2218 BKG	100	cpm
RO-2 BKG	0.01	mR/hr
LUDLUM 2218 EFF.	5	%
FRISKER EFF.	10	%
AIR SAMPLE VOL.	35	cubic feet

TIME: 16:00

CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	14	14	13	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.1	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

HALF CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	13	13	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

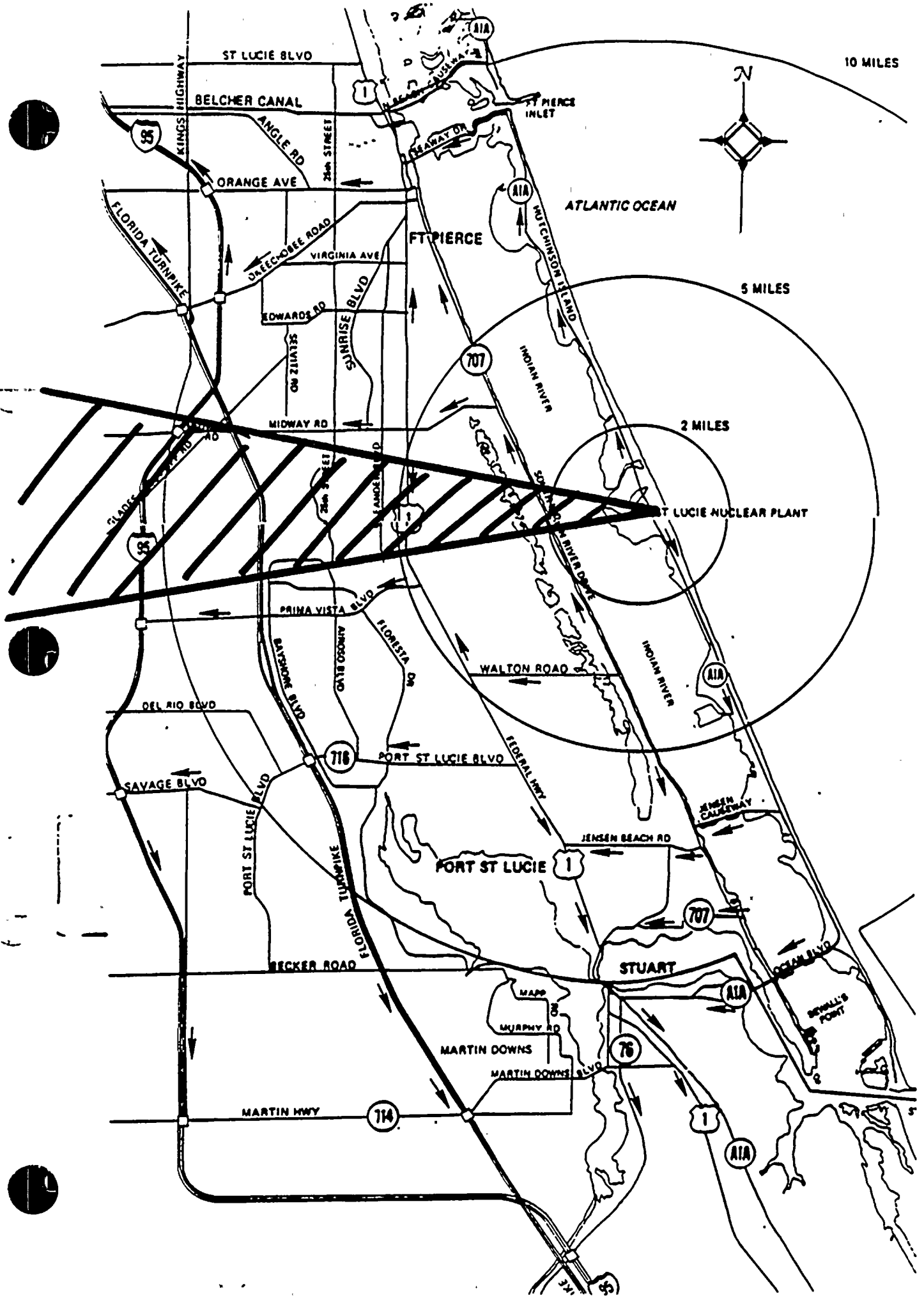
TENTH CENTERLINE

MILES FROM SOURCE		0.5	1	2	3	4	5	6	7	8	9	10
SECTOR		N	N	N	N	N	N	N	N	N	N	N
PREFILTER GROSS COUNTS	cpm	12	12	12	12	12	12	12	12	12	12	12
CART. GROSS TOTAL CNTS, 5 MIN	counts	500	500	500	500	500	500	500	500	500	500	500
IODINE AIRBORNE CONC.	uCi/cc	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10	<3.30E-10
OPEN WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CLOSED WINDOW @ HEAD HT.	mR/hr	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CART. GROSS TOTAL CNTS, 1 MIN	counts											

NOTES:

CART. GROSS TOTAL COUNTS IS TOTAL COUNTS + BACKGROUND ON IODINE CARTRIDGE @ 5 MIN. WITH 35 CUBIC FOOT SAMPLE.
BACKGROUND COUNT ON LUDLUM 2218 = 500 GROSS COUNTS, 5 MIN. COUNT, 100 CPM.
AIR SAMPLE FLOW = 7 CFM FOR 5 MIN. = 35 CFM

7.5.2 OFF-SITE PLUME MAP


ST LUCIE BLVD
BELCHER CANAL
KINGS HIGHWAY
ANGLE RD
25th STREET
ORANGE AVE
FLORIDA TURNPIKE
OKEECHOBEE ROAD
VIRGINIA AVE
EDWARDS RD
SELTZIG RD
SUNRISE BLVD
MIDWAY RD
GLADES CUT OFF RD
PRIMA VISTA BLVD
BAYSHORE BLVD
ANDROS BLVD
FLORESTA DR
OEL RIO BLVD
PORT ST LUCIE BLVD
SAVAGE BLVD
PORT ST LUCIE BLVD
BECKER ROAD
MARTIN HWY
FLORIDA TURNPIKE
FT PIERCE INLET
SEAWAY DR
FT PIERCE
HUTCHINSON ISLAND
ATLANTIC OCEAN
INDIAN RIVER
SOUTH RIVER DRIVE
ST LUCIE NUCLEAR PLANT
WALTON ROAD
INDIAN RIVER
FEDERAL HWY
JENSEN CAUSEWAY
JENSEN BEACH RD
FORT ST LUCIE
STUART
OCEAN BLVD
MAPP
MURPHY RD
MARTIN DOWNS
MARTIN DOWNS BLVD
PECKALL'S POINT
10 MILES
5 MILES
2 MILES
N
A1A
1
707
716
714
76
95

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

8.0 MISCELLANEOUS

This Scenario Section Contains the Following Information:

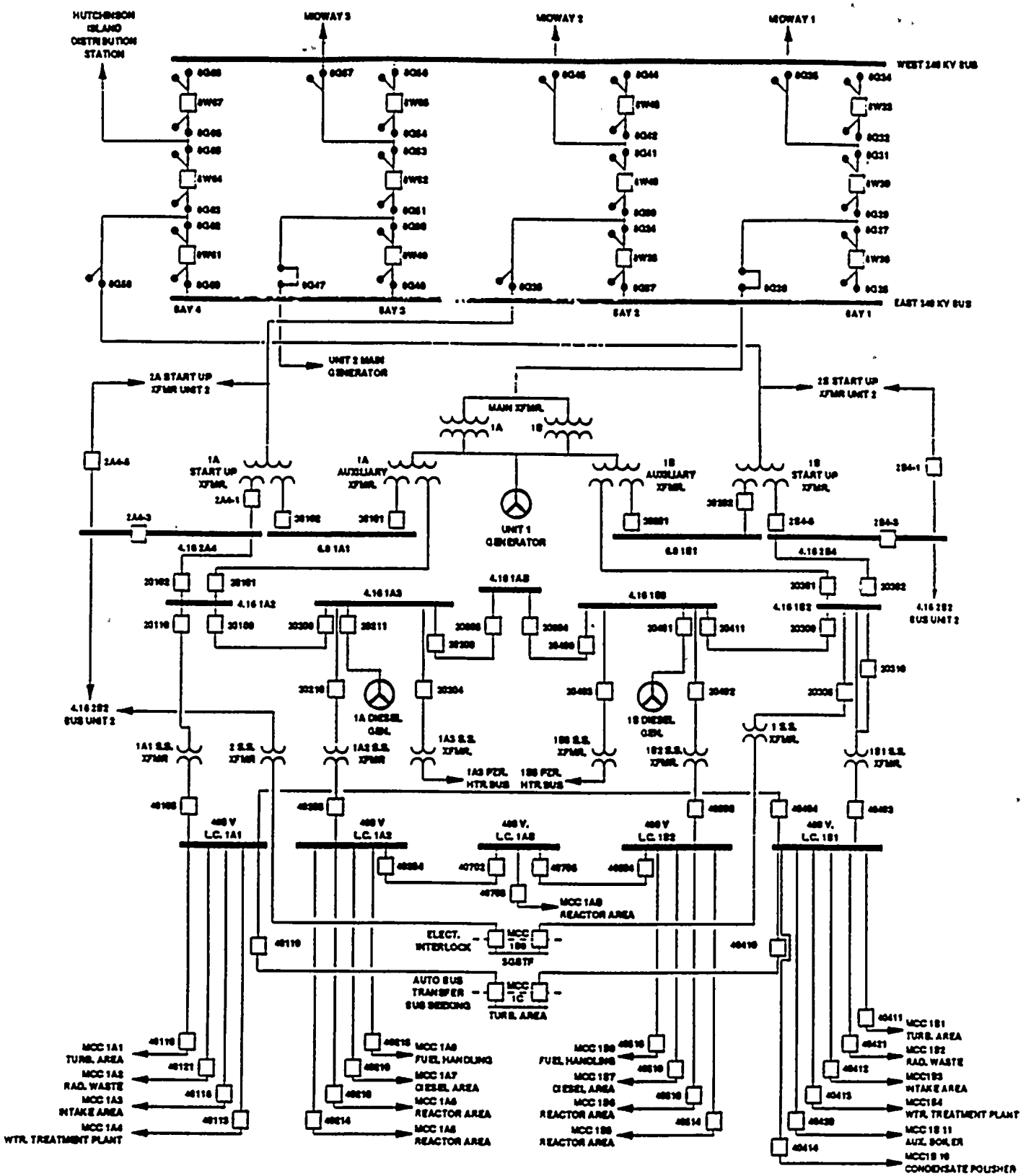
Section 8.1	EVALUATION MATERIALS - Forms and record sheets to be utilized by the FPL Evaluators in recording and evaluating the performance of the participants.
Section 8.2	REFERENCES - Pertinent scenario data references.
Section 8.3	ACRONYMS AND ABBREVIATIONS - An alphabetical listing of many acronyms and abbreviations utilized in the scenario text and at the St. Lucie Nuclear Plant.
Section 8.4	SITE LAYOUT DATA - Reference data on the St. Lucie Nuclear Plant location, key facilities, access routes, etc.

8.1 EVALUATION MATERIALS

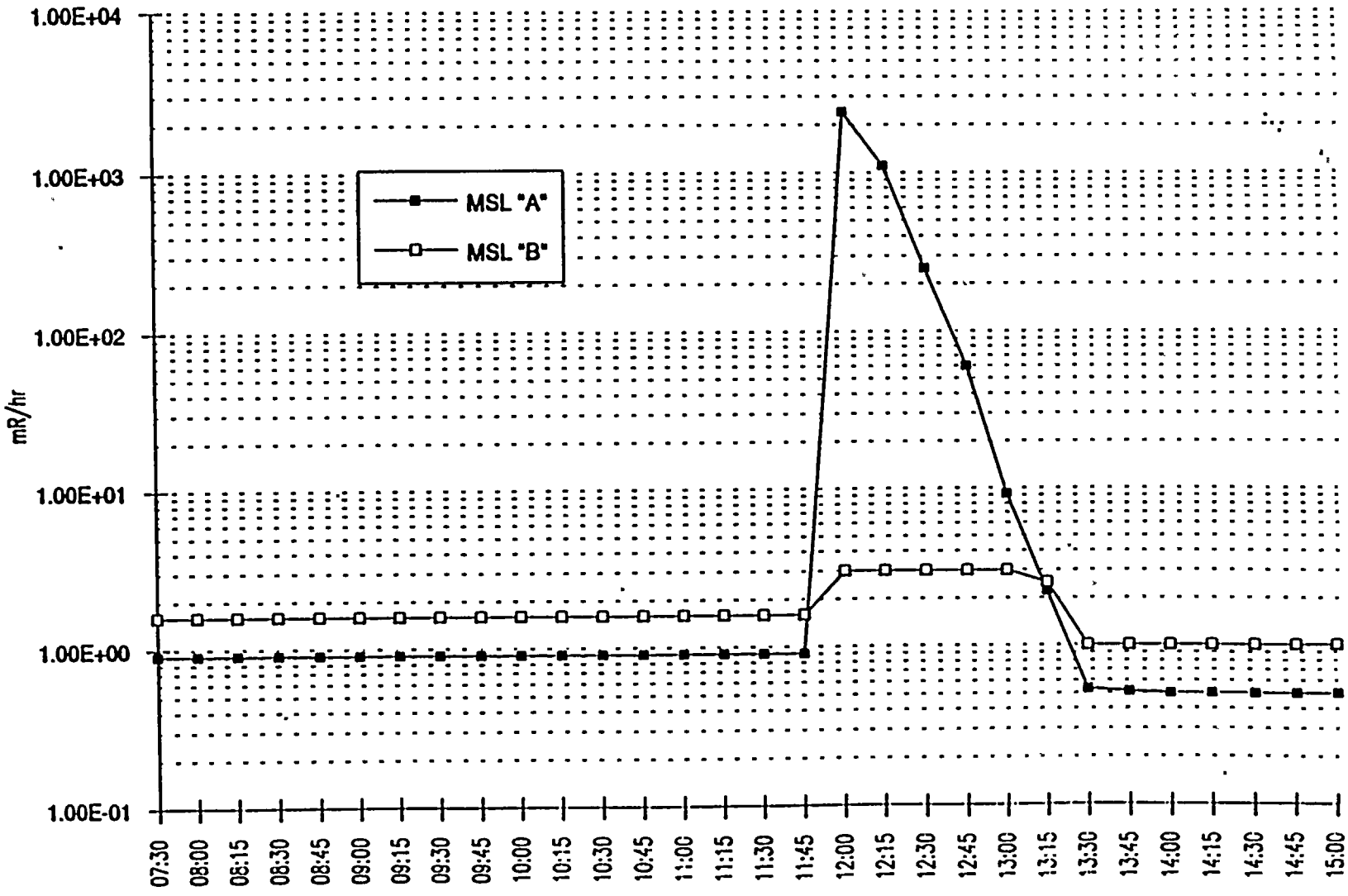
This Data to be Provided Later

8.2 REFERENCES

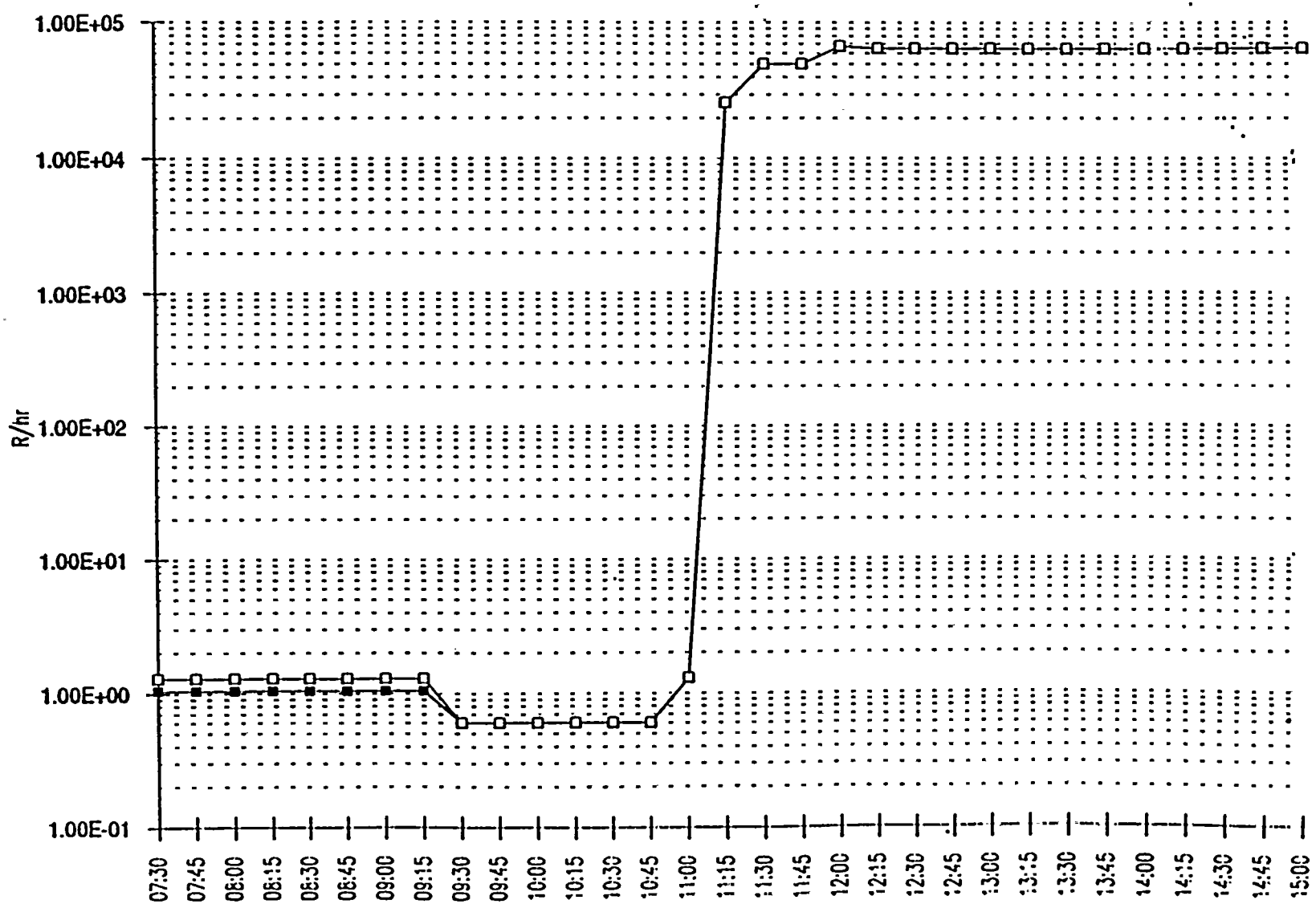
MAIN POWER DISTRIBUTION SYSTEM UNIT 1
FIGURE 1



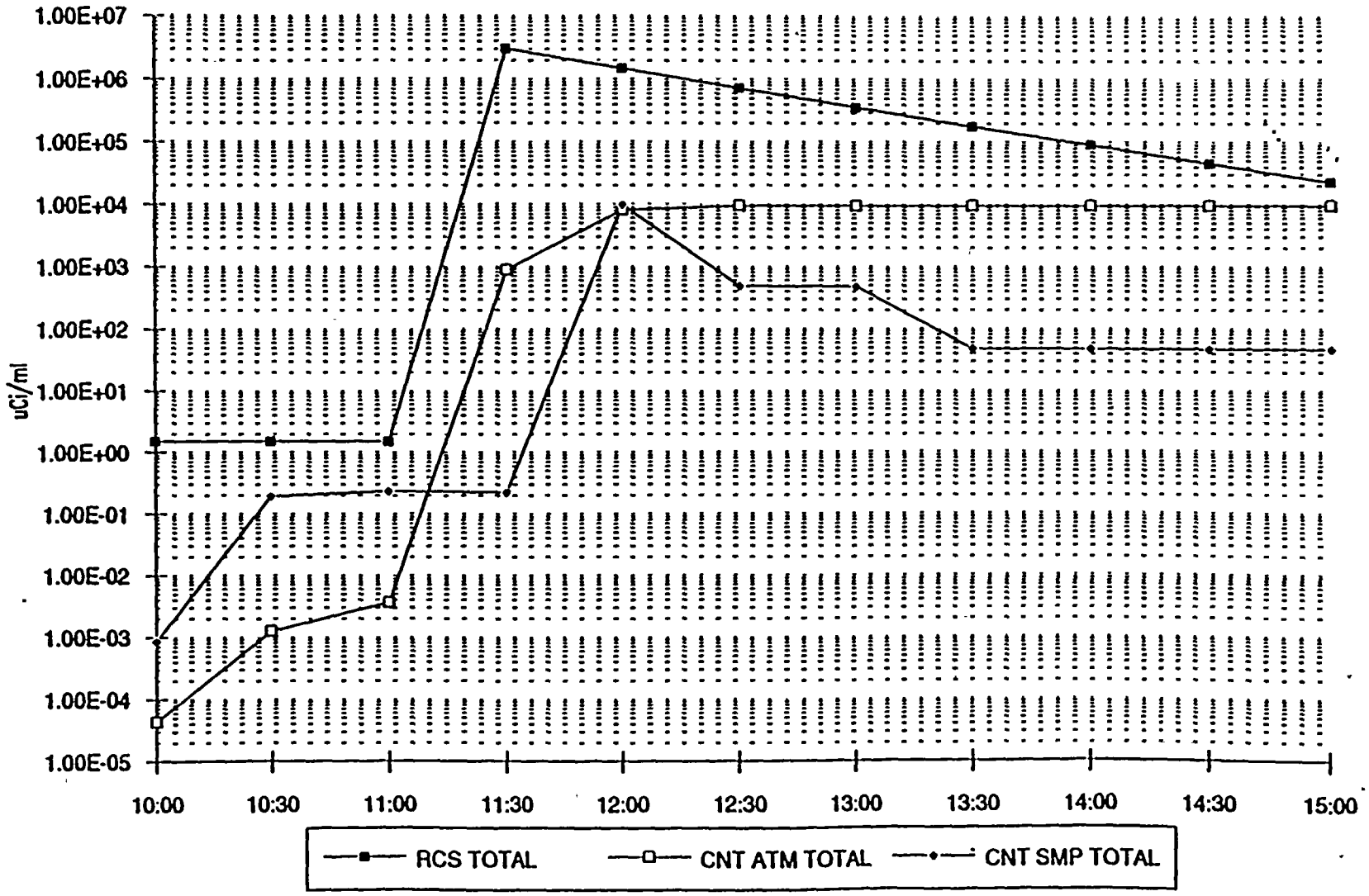
1992 Exercise Main Steam Line Monitors



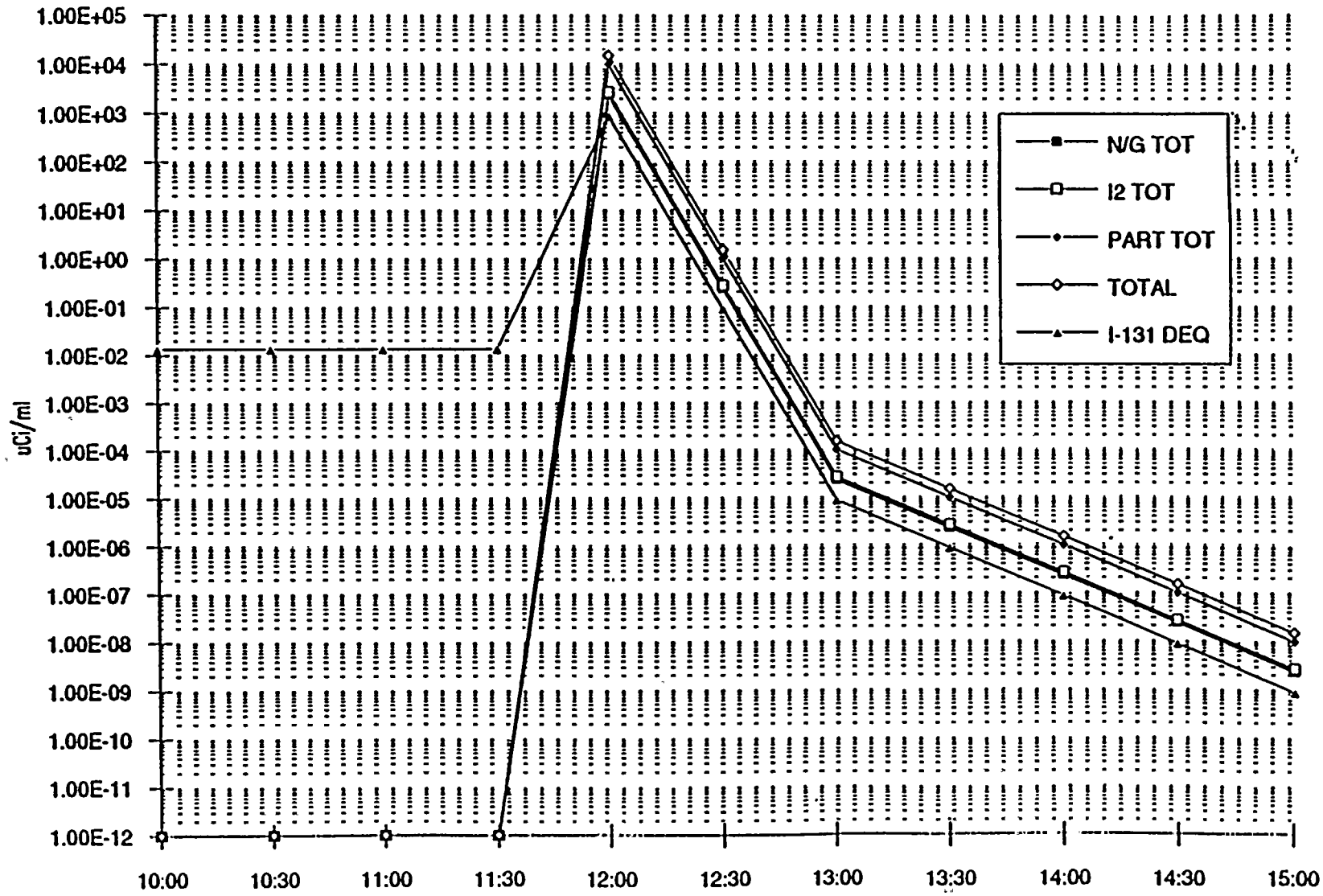
1992 Exercise CHRRMS



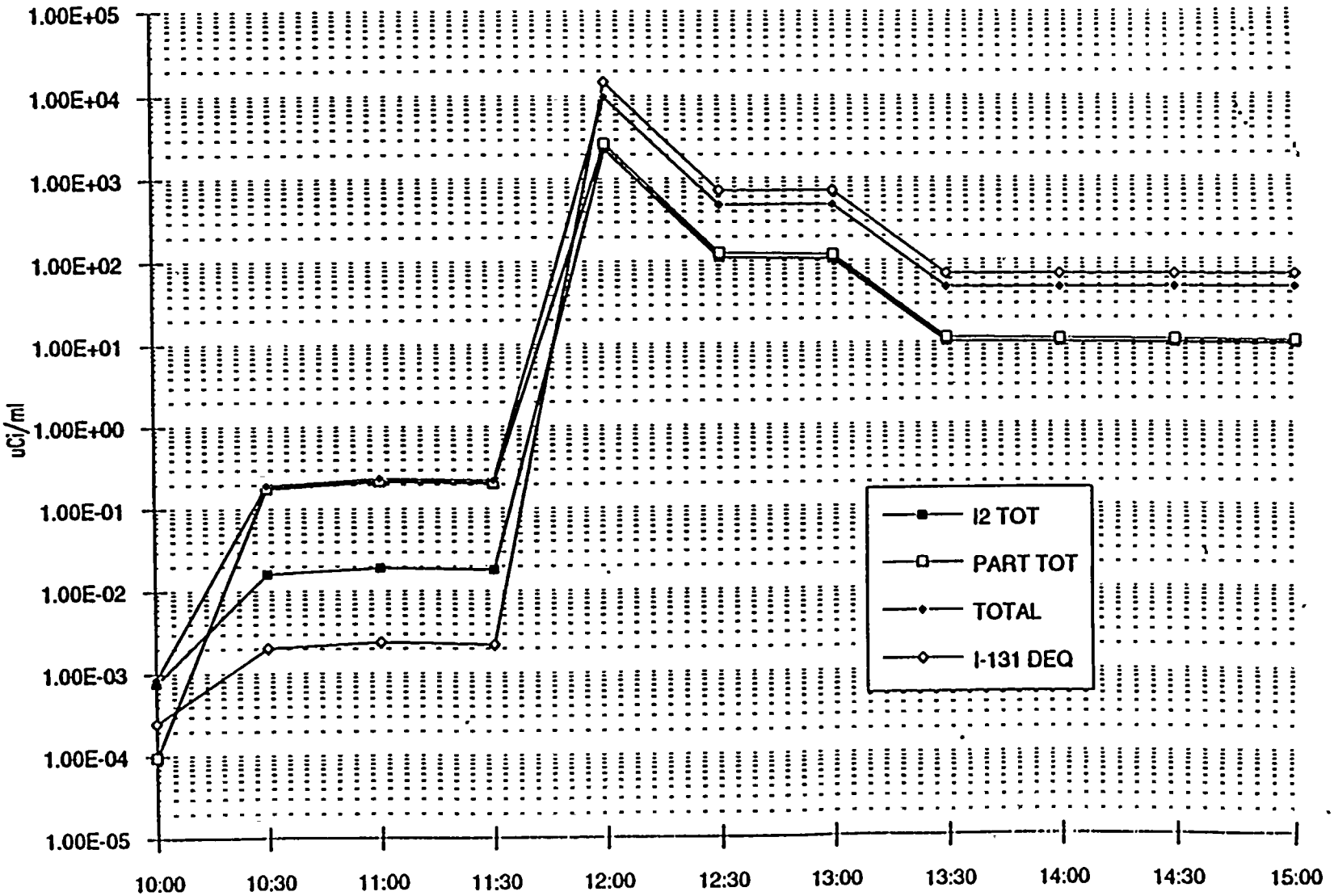
1992 Exercise Containment Chemistry Summary



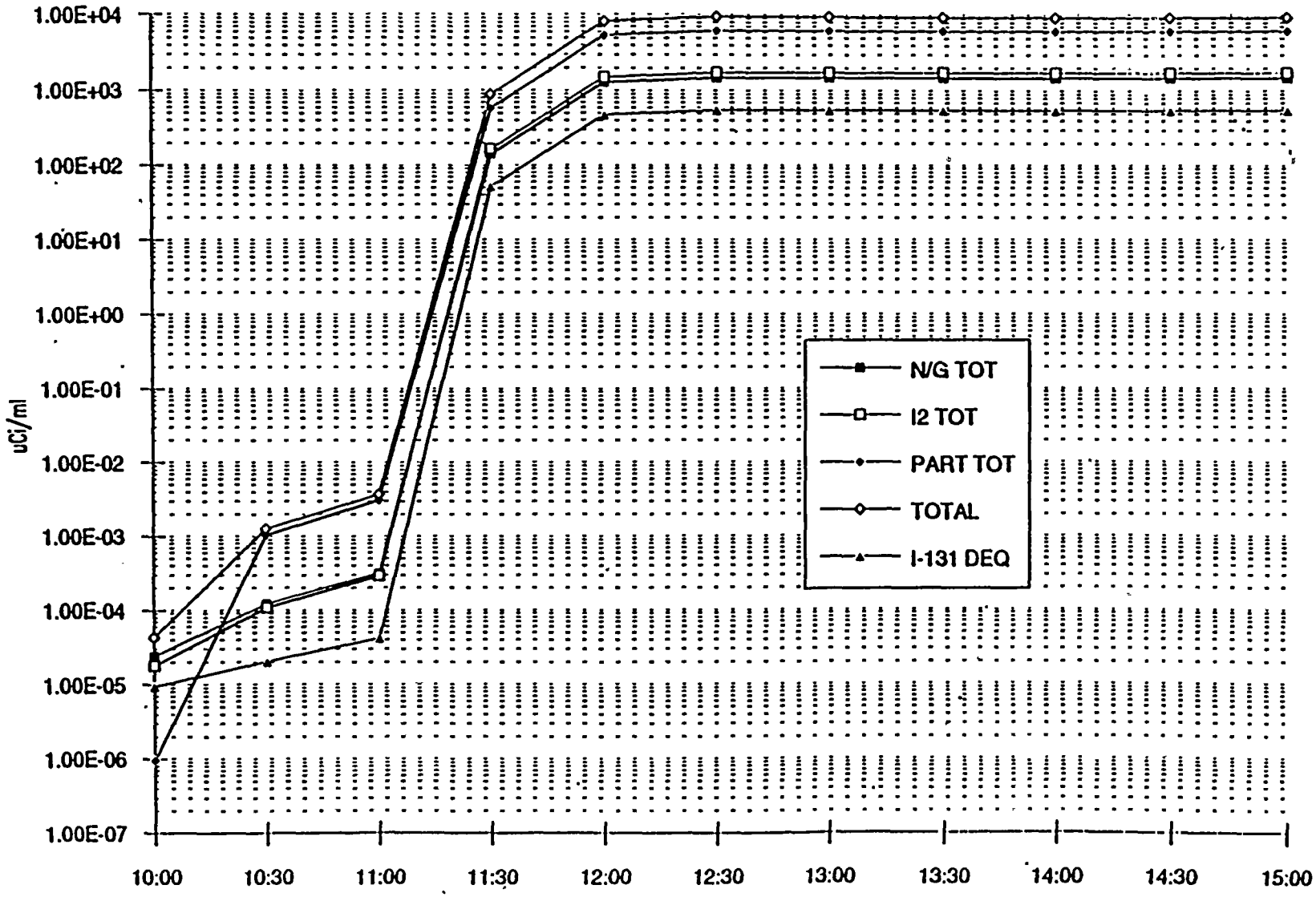
1992 Exercise S/G A Concentration



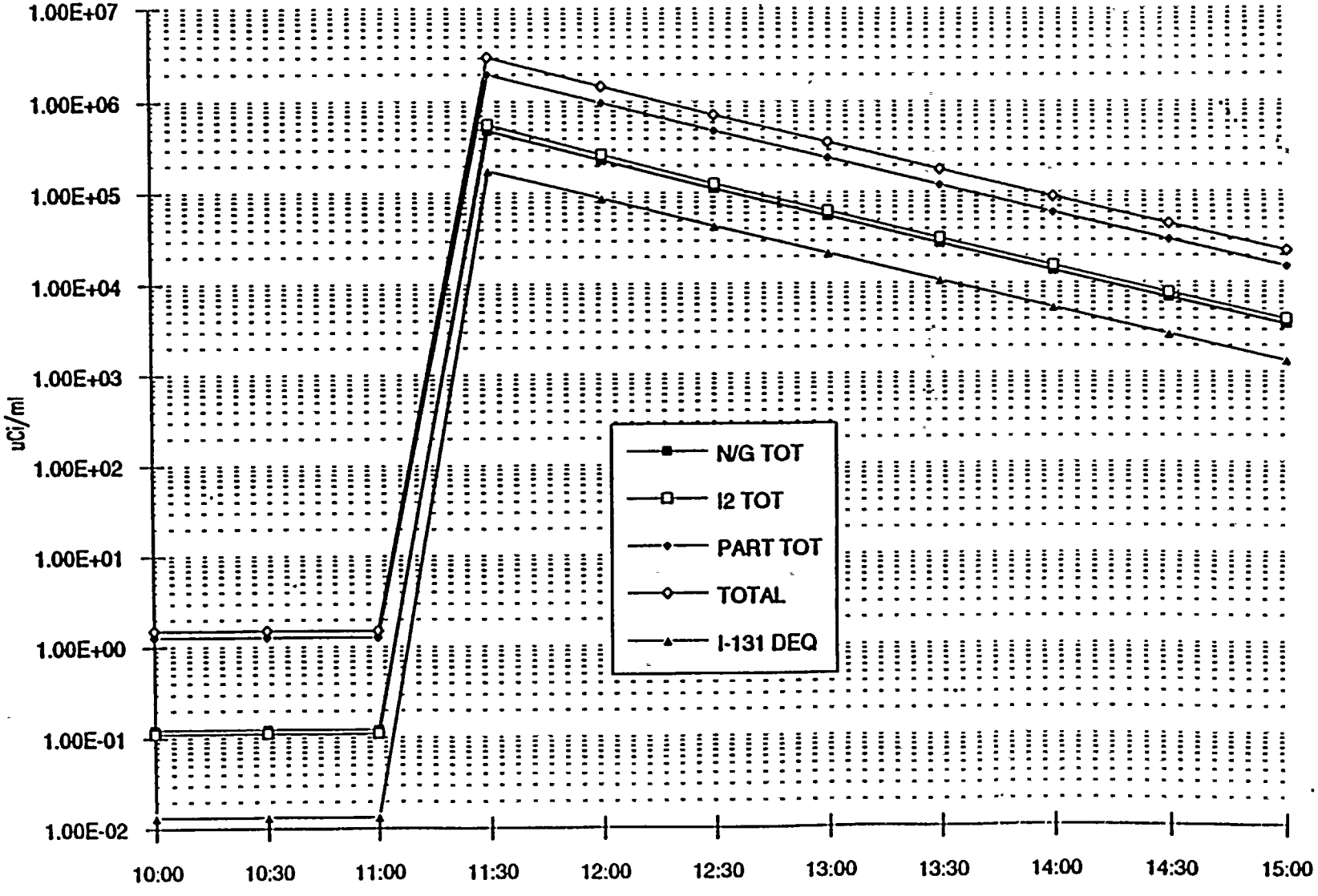
1992 Exercise Containment Sumps



1992 Exercise Containment Atmosphere



1992 Exercise RCS Activity Summary



FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

8.3 ACRONYMS AND ABBREVIATIONS

AC	Alternating Current
ADV	Atmospheric Dump Valve
AFAS	Auxiliary Feedwater Actuation Signal
AFP	Auxiliary Feedwater Pump
AFW	Auxiliary Feedwater
ANPO	Auxiliary Nuclear Plant Operator
ANPS	Assistant Nuclear Plant Superintendent
ARC	American Red Cross
ARM	Area Radiation Monitor
ATWS	Anticipated Transient Without SCRAM
AUX	Auxiliary
BAMT	Boric Acid Makeup Tank
BATT	Battery
BHB	Backup Heater Breaker
BOP	Balance of Plant
C-E	Combustion Engineering
CAS	Central Alarm Station
CAT	Chemical Addition Tank
CCAS	Containment Cooling Actuation Signal
CCW	Component Cooling Water
CEA	Control Element Assembly
CEDM	Control Element Drive Mechanism
CET	Core Exit Thermocouple
CFMS	Critical Function Monitoring System
CHRRM	Containment High-Range Radiation Monitor
CIAS	Containment Isolation Actuation Signal
CNTMT	Containment
COND	Condensate (Condenser)

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE NUCLEAR PLANT
1992 EMERGENCY PREPAREDNESS
EVALUATED EXERCISE
FEBRUARY 12, 1992

8.3 ACRONYMS AND ABBREVIATIONS

CONT	Controller
CPIAS	Containment Purge Isolation Actuation Signal
CPM	Counts Per Minute
CPS	Counts Per Second
CR	Control Room
CRAC	Control Room Auxiliary Console
CS	Containment Spray
CSAS	Containment Spray Actuation Signal
CSS	Containment Spray System
CST	Condensate Storage Tank
CTL	Chemistry Team Leader
CTPC	Containment Temperature and Pressure Control
CVCS	Chemical Volume Control System
DC	Direct Current
DCS	Duty Call Supervisor
DE	Dose Equivalent (eg: DE-I_{131})
DECON	Decontaminate (Decontamination)
DEM	Florida Division of Emergency Management
DER	Florida Department of Environmental Regulation
DHRS	Florida Department of Health & Rehabilitative Services
DNB	Departure From Nucleate Boiling
DPM	Disintegrations Per Minute
DPS	Disintegrations Per Second
EAL	Emergency Action Level
EBS	Emergency Broadcast System
EC	Emergency Coordinator
ECCS	Emergency Core Cooling System
ECO	Emergency Control Officer

8.3 ACRONYMS AND ABBREVIATIONS (Continued)

EDG	Emergency Diesel Generator
EDT	Equipment Drain Tank
EIM	Emergency Information Manager
EMT	Emergency Medical Technician
ENC	Emergency News Center
EOC	Emergency Operating Center
EOF	Emergency Operations Facility
EOP	Emergency Operating Procedure
EPA	U.S. Environmental Protection Agency
EPIP	Emergency Plan Implementing Procedure
EPZ	Emergency Planning Zone
ERDADS	Emergency Response Data Acquisition and Display System
ERF	Emergency Response Facility
ESC	Engineered Safeguards Cabinet
ESF	Engineered Safety Feature
ESM	Emergency Security Manager
F-A	First Aid
FEMA	Federal Emergency Management Agency.
FEOC	Field Emergency Operations Center (DEM)
FCV	Flow Control Valve
FPL	Florida Power and Light Company
FRERP	Federal Radiological Emergency Response Plan
FRMAP	Federal Radiological Monitoring and Assessment Plan
FSAR	Final Safety Analysis Report
FT	Flash Tank
GAC	Governor's Advisory Committee
GAM	Governor's Affairs Manager
GAR	Governor's Authorized Representative
GDT	Gas Delay Tank
GE	General Emergency
GPM	Gallons Per Minute

8.3 ACRONYMS AND ABBREVIATIONS (Continued)

HPSI	High Pressure Safety Injection
HPT	Health Physics Technician
HR	Heat Removal
HRD	Hot Ring-Down (Phone System)
HSCP	Hot Shutdown Control Panel
HT	Holdup Tank
HVAC	Heating, Ventilation and Air Conditioning
HX	Heat Exchanger
IC	Inventory Control
I&C	Instrumentation and Calibration
ICW	Intake Cooling Water
IRS	Iodine Removal System
JENC	Joint Emergency News Center
KI	Potassium Iodide
LCO	Limiting Condition for Operation
LGR	Local Government Radio
LOCA	Loss of Coolant Accident
LOF	Loss of Feedwater
LOOP	Loss of Off-Site Power
LPRT	Low Pressurizer Pressure Relief Valve Trip
LPSI	Low Pressure Safety Injection
LPMS	Loose Parts Monitoring System
MCA	Multi-Channel Analyzer
MCC	Motor Control Center
MDA	Minimum Detectable Activity
MERL	Mobile Emergency Radiological Laboratory (DEM)
MOV	Motor Operated Valve
MPC	Maximum Permissible Concentration
MPH	Miles Per Hour
MSIS	Main Steam Isolation Signal

8.3 ACRONYMS AND ABBREVIATIONS (Continued)

MSIV	Main Steam Isolation Valve
MSLB	Main Steam Line Break
MSSV	Main Steam Safety Valve
NAWAS	National Warning System
NEDO	Nuclear Energy Duty Officer
NC	Natural Circulation
NPO	Nuclear Plant Operator
NPS	Nuclear Plant Supervisor
NRC	U.S. Nuclear Regulatory Commission
NSSS	Nuclear Steam Supply System
NUE	Notification of Unusual Event
NWE	Nuclear Watch Engineer
OSC	Operational Support Center
PACB	Plant Auxiliary Control Board
PAG	Protective Action Guideline
PAR	Protective Action Recommendation
PASS	Post-Accident Sampling System
PCs	Protective Clothing
PIO	Public Information Officer
PLCS	Pressurizer Level Control System
PORV	Power Operated Relief Valve
PPM	Parts Per Million
PRM	Process Radiation Monitor
PROC	Procedure
PZR	Pressurizer
QT	Quench Tank
RAB	Reactor Auxiliary Building
RAS	Recirculation Actuation Signal
RAT	Resin Addition Tank
RC	Reactivity Control

8.3 ACRONYMS AND ABBREVIATIONS (Continued)

RCA	Radiologically Controlled Area
RCC	Reception and Care Center
RCP	Reactor Coolant Pump
RCS	Reactor Coolant System
RDT	Reactor Drain Tank
RECS	Radiological Emergency Communication System
REP	Radiation Exposure Permit
RM	Recovery Manager
RO	Reactor Operator
RPM	Radiation Protection Manager
RPS	Reactor Protective System
RRS	Reactor Regulatory System
RSO	Radiation Safety Officer
RT	Reactor Trip
RTGB	Reactor Turbine Generator Board
RTL	Radiation Team Leader
RV	Reactor Vessel
RVLMS	Reactor Vessel Level Monitoring System
RWT	Refueling Water Tank
Rx	Reactor
SAE	Site Area Emergency
SAS	Secondary Alarm Station
SB	Station Blackout
SCFM	Standard Cubic Feet Per Minute
SCS	Shutdown Cooling System
SE	Solenoid Valve
SEC	Security
SED	Sequence of Events Diagram
SEOC	State Emergency Operating Center
SFSC	Safety Function Status Check
S/G	Steam Generator
SIAS	Safety Injection Actuation Signal
SIS	Safety Injection System
SIT	Safety Injection Tank

8.3 ACRONYMS AND ABBREVIATIONS (Continued)

SMM	Subcooled Margin Monitor
SMRAP	Southern Mutual Radiological Assistance Plan
SRO	Senior Reactor Operator
SRT	Spent Resin Tank
STA	Shift Technical Advisor
SWGR	Switchgear
SWP	State Warning Point
T_{ave}	Average Coolant Temperature
TBV	Turbine Bypass Valve
T_c	Cold Leg Temperature
TCV	Temperature Control Valve
TCW	Turbine Cooling Water
T_h	Hot Leg Temperature
TO	Turbine Operator
T_{ref}	Turbine Control System Reference Temperature
TSC	Technical Support Center
VCT	Volume Control Tank
XFMR	Transformer

8.4 SITE LAYOUT DATA

1.5.1 Control Room (Figure 3)

For any emergency response, the Control Room of the affected unit serves the initial point of control. The Nuclear Plant Supervisor stations himself in the affected unit's Control Room when he assumes the role of Emergency Coordinator. The Emergency Coordinator can leave the Control Room if necessary, after a proper turnover to a qualified alternate Emergency Coordinator, to make a personal assessment regarding plant safety.

The Control Rooms are designed to remain tenable under accident conditions described in the Final Safety Analysis Report. All plant related operations are directed from the Control Room. Nuclear plant instrumentation, including area and process radiation monitoring system instrumentation, is provided in the Control Room to give early warning of a potential emergency and to provide for continuing indication of an emergency situation. The Control Rooms contain the controls and instrumentation necessary for operation of the reactor under normal and emergency conditions.

A supply of radiological emergency equipment is maintained in the Control Room. Each Control Room contains the necessary communications equipment for notifying on-site personnel and offsite authorities in the event of an accident. This includes the State's Hot Ringdown telephone, National Warning System (NAWAS), Local Government Radio System, Emergency Notification System (ENS) to the NRC Operations Center (in Bethesda, MD), commercial telephones, Florida Power and Light Company radio system, public address system, PAX telephone system, portable radio sets (walkie-talkies), and a radio paging system. These systems are used as defined by procedure to accomplish the necessary notifications and communications.

1.5.2 Technical Support Center (Figure 4)

The Company maintains an on-site Technical Support Center to provide the Control Room and the Emergency Operations Facility with in-depth diagnostic and engineering assistance without adding to congestion within the Control Room. This assistance can help determine the operational decisions that would be appropriate to better control and mitigate the consequences of the emergency. The Technical Support Center is located in a block of rooms adjacent to the Unit 1 Control Room.

Activation of the Technical Support Center will be initiated by the Emergency Coordinator in the event of an Alert, Site Area Emergency or General Emergency. Arrangements have been made to staff the Technical Support Center in timely manner.

The Technical Support Center contains pertinent records and drawings. It has an emergency communications network similar to the Control Room, with the exception of NAWAS.

1.5.3 Operational Support Center

The Company maintains an on-site Operational Support Center to serve as an assembly point for auxiliary operators, health physics technicians, maintenance personnel, and other plant shift personnel available to support the emergency response. Required staff will be assigned to appropriate activities by the Emergency Coordinator or his designee.

The Operational Support Center is located on the second floor of the Service Building in the large conference room. Equipment that can be used by personnel dispatched from the facility is stored in the same building.

Activation of the Operational Support Center will be initiated by the Emergency Coordinator. It will be in operation for an Alert, Site Area Emergency or General Emergency and will be staffed in timely manner.

1.5.4 Emergency Operations Facility (Figure 6)

The Company maintains an Emergency Operations Facility from which evaluation and coordination of all FPL activities related to an emergency can be carried out and from which FPL can provide information to Federal, State, and local authorities.

The Emergency Operations Facility is located at the intersection of State Road 712 (Midway Road) and I-95 approximately 10.5 miles west of the St. Lucie Plant. The Emergency Operations Facility has sufficient space to accommodate the FPL response organization and designated representatives of Federal, State, and local government.

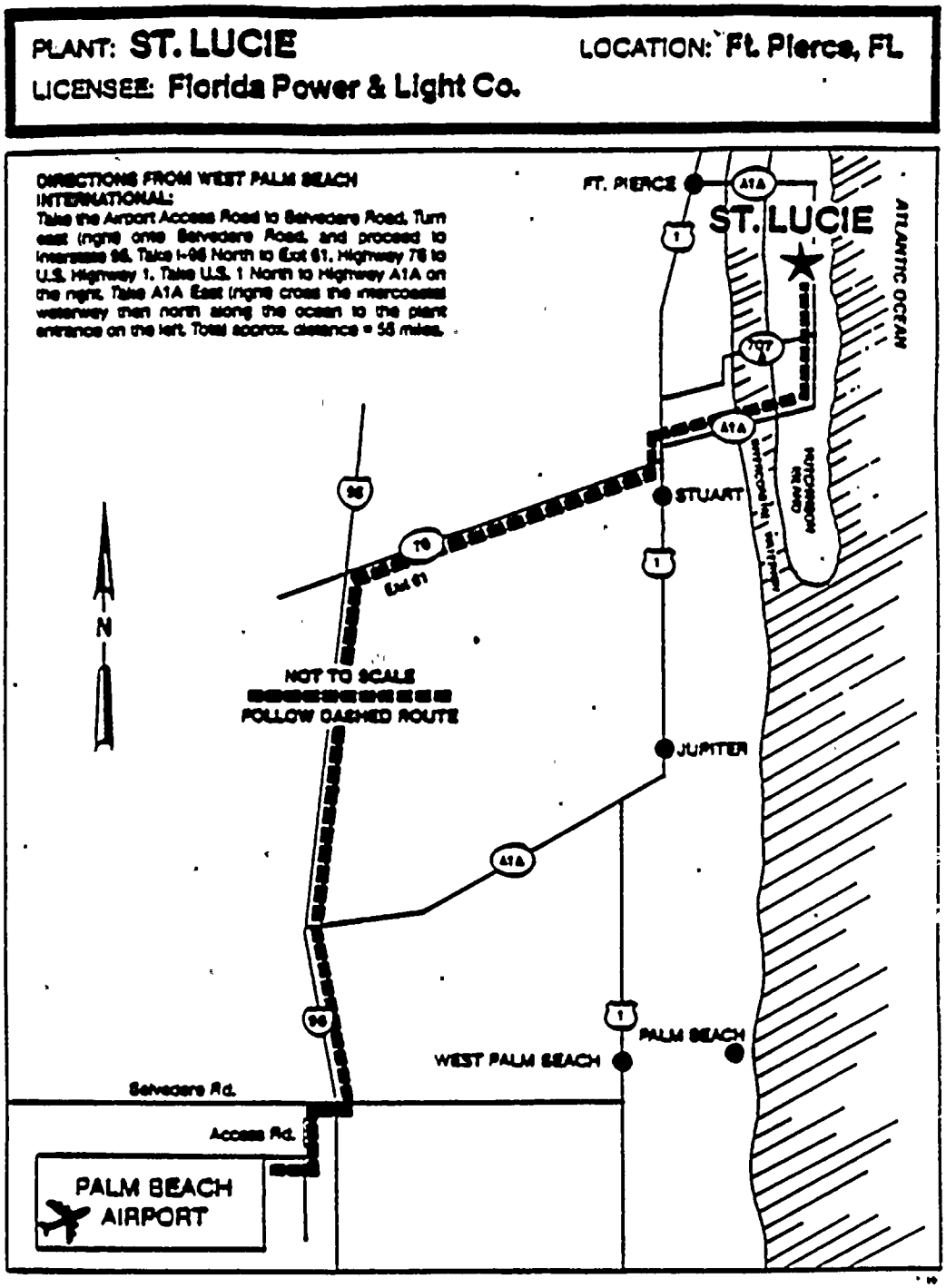
The Emergency Operations Facility has an emergency communications network including but not limited to Local Government Radio, commercial telephone lines, State Hot Ringdown phone, NRC ENS, and FPL Company Radio. Essential precalculated emergency data, UFSAR, T/S, Procedures, and selected drawings are available. Activation of the Emergency Operations Facility is the responsibility of the Emergency Control Officer and is required for a Site Area Emergency or General Emergency. Arrangements have been made for staffing in a timely manner.

1.5.5 Emergency News Center (Figure 6)

An Emergency News Center is provided to allow the news media access to information from the Emergency Operations Facility. The Emergency Information Manager will designate an individual to supervise the Emergency News Center. The Center is co-located with the Emergency Operations Facility (Midway Road and I-95).

1.4 Travel Information

Figure 1: Travel Directions to Plant St. Lucie



1.5 Site Map and Emergency Response Facilities

Figure 2: St. Lucie Plant Site Layout/On-site Emergency Response Facilities

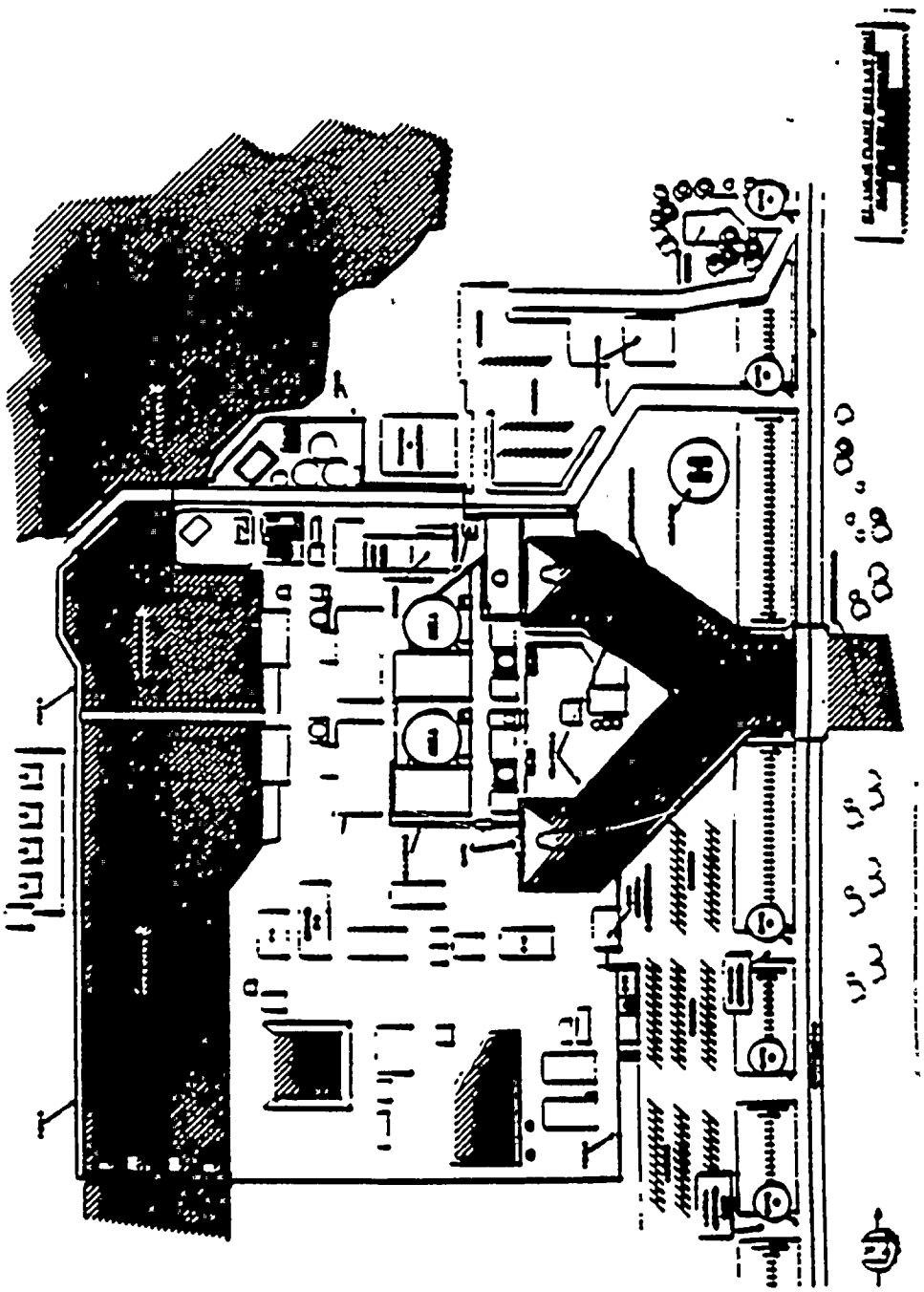
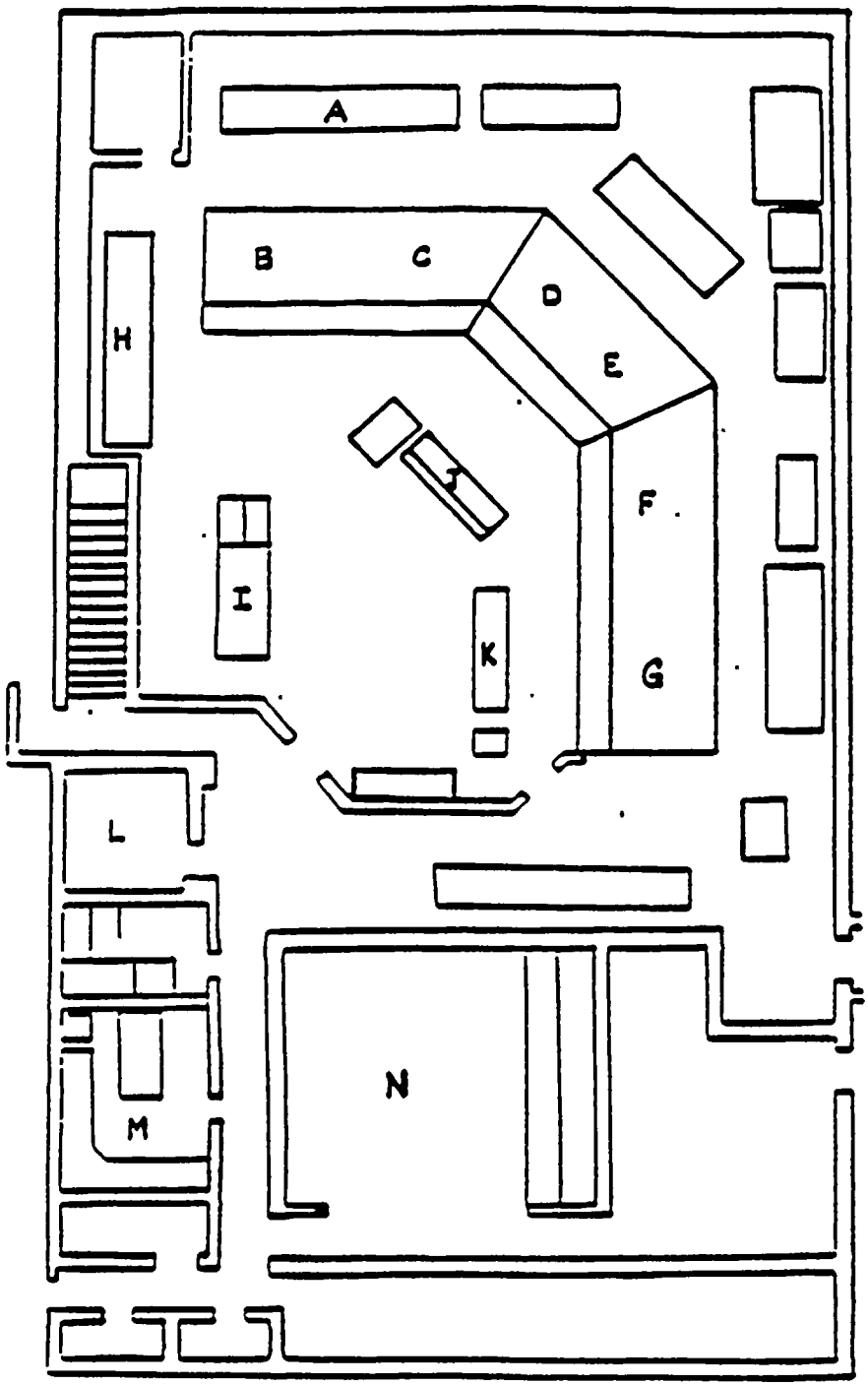


Figure 3: Unit 1 Control Room 62.0'EL.



A - Engineered Safeguard Logic Cabinet
B - Engineered Safeguard
C - Chemical Volume Contro
D - Reactivity
E - Reactor Coolant System
F - Condensate Cooling
G - Turbine Generator
H - Rad Monitoring Panels
I - Reactor Protective Syst
J - Communications Equipmen
K - ERDADS
L - Office
M - Kitchen
N - Technical Support Center

Figure 4 Technical Support Center

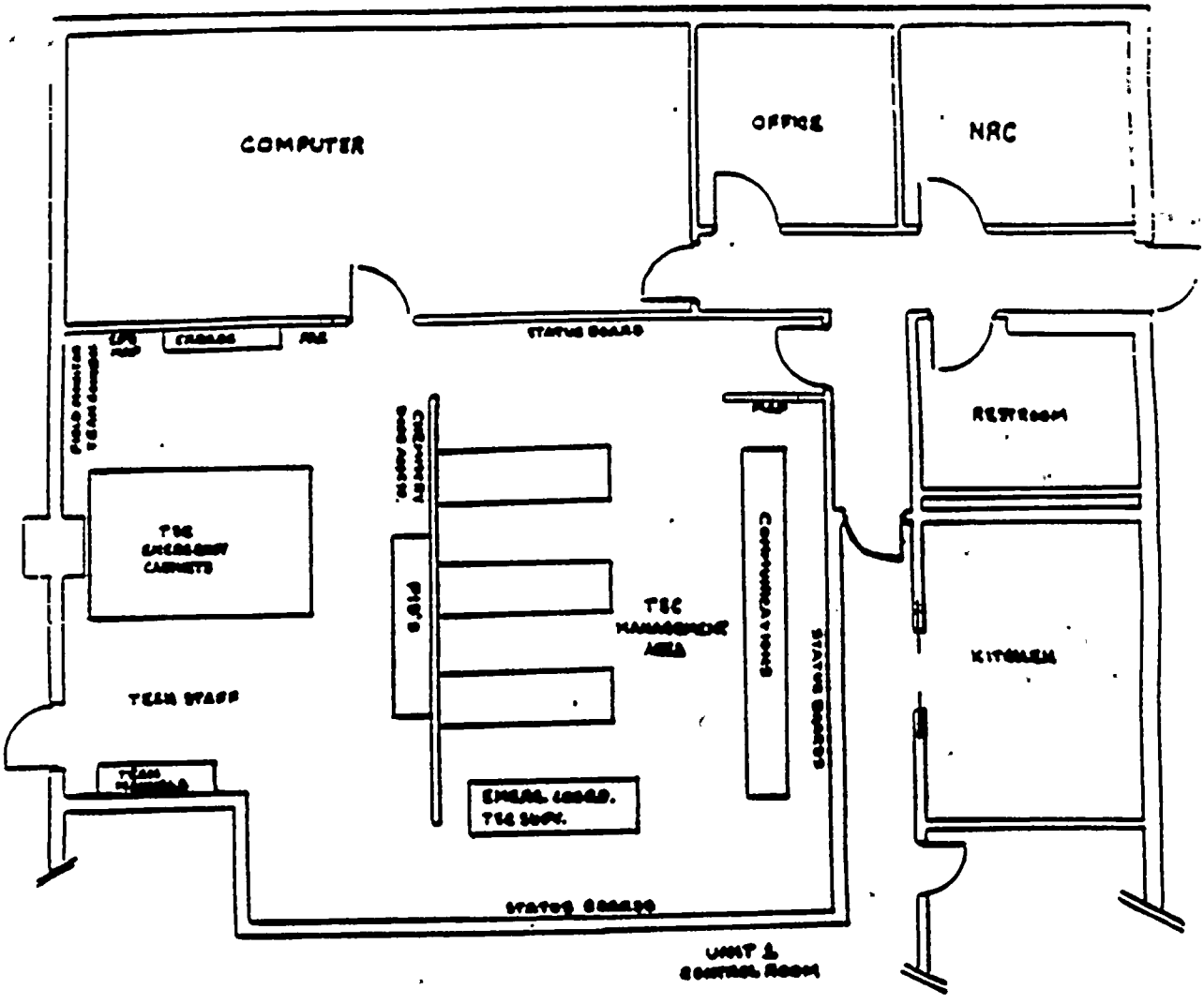


Figure 5: Travel Directions to the Emergency Operations Facility and News Center

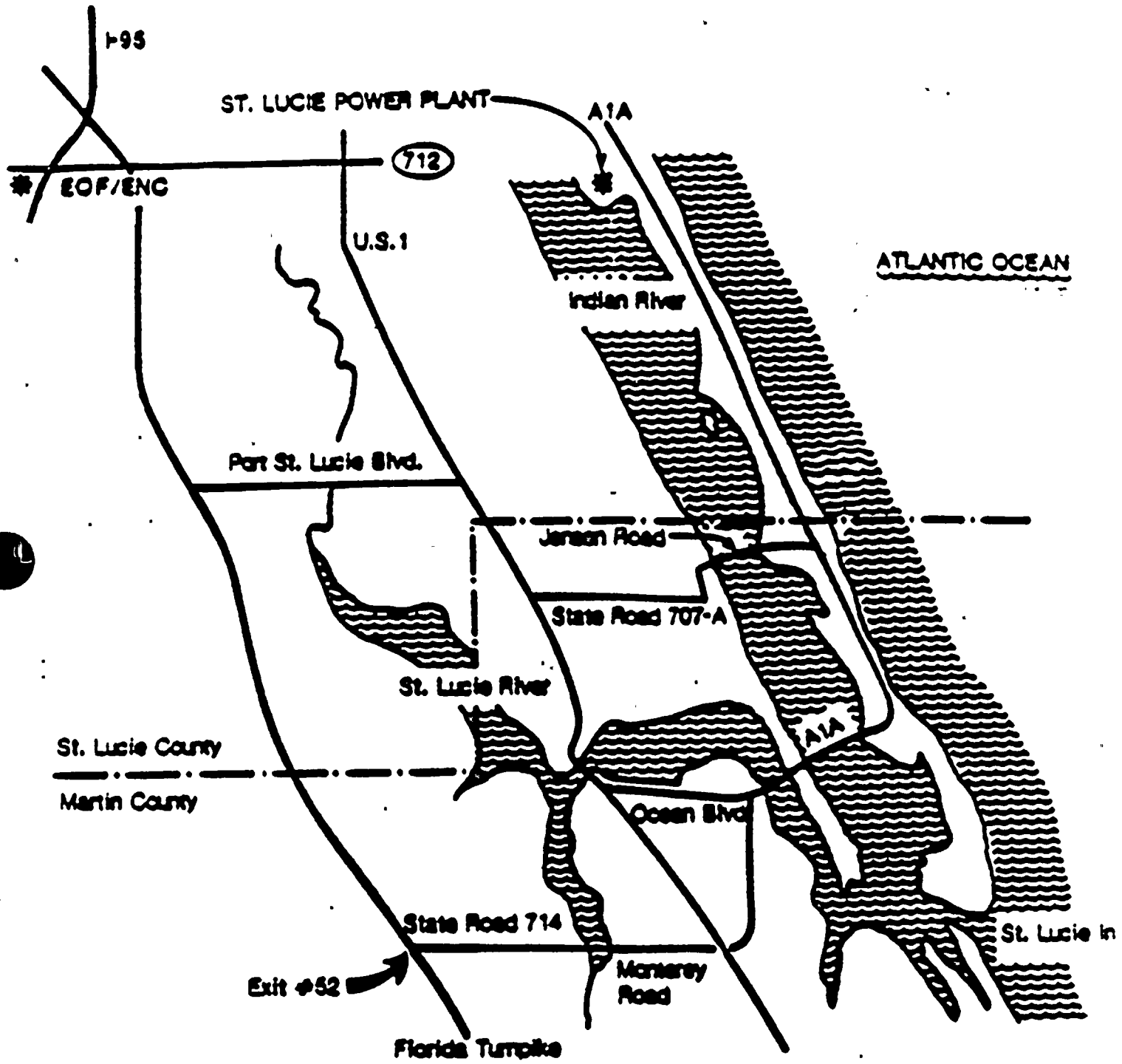
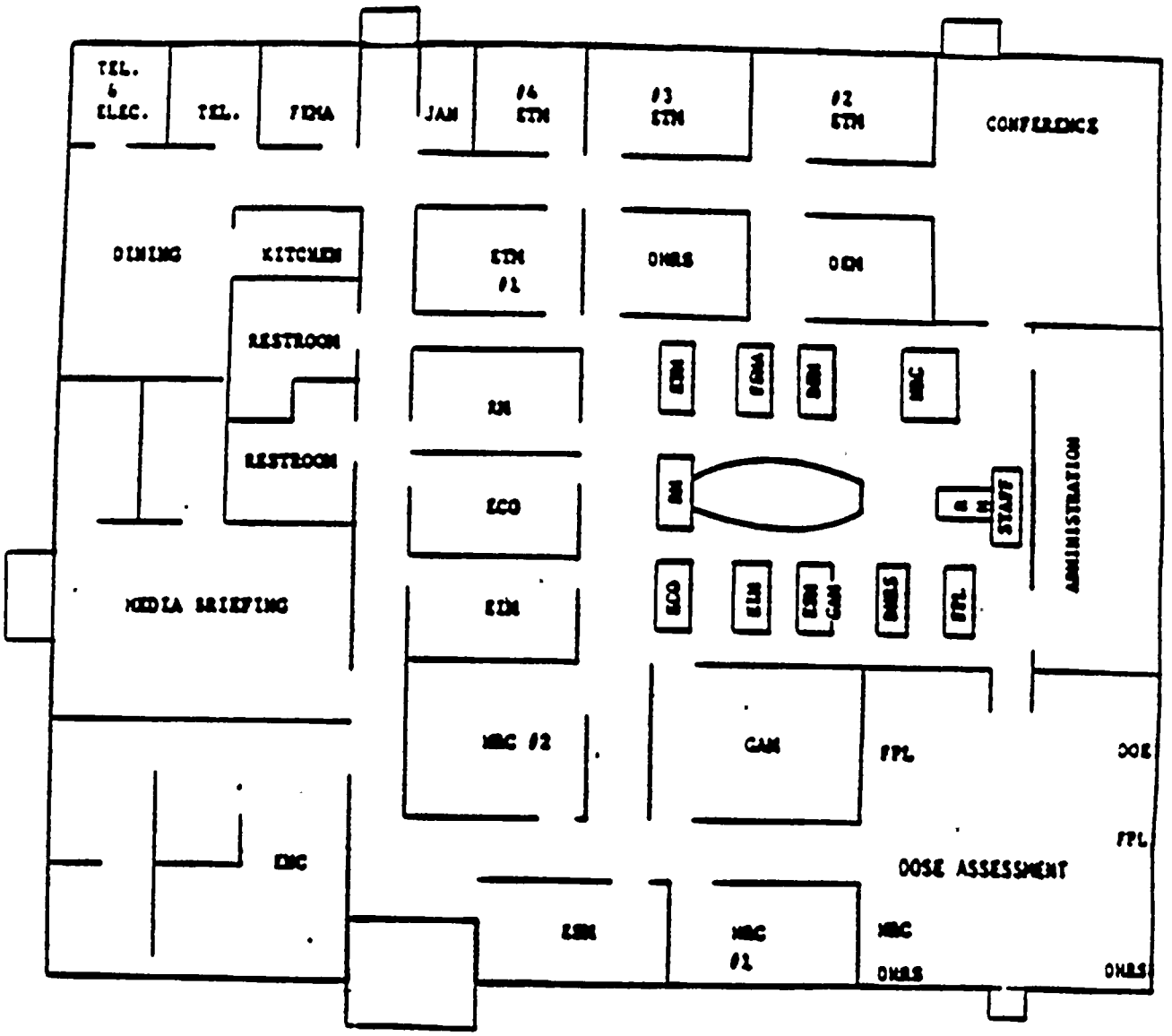
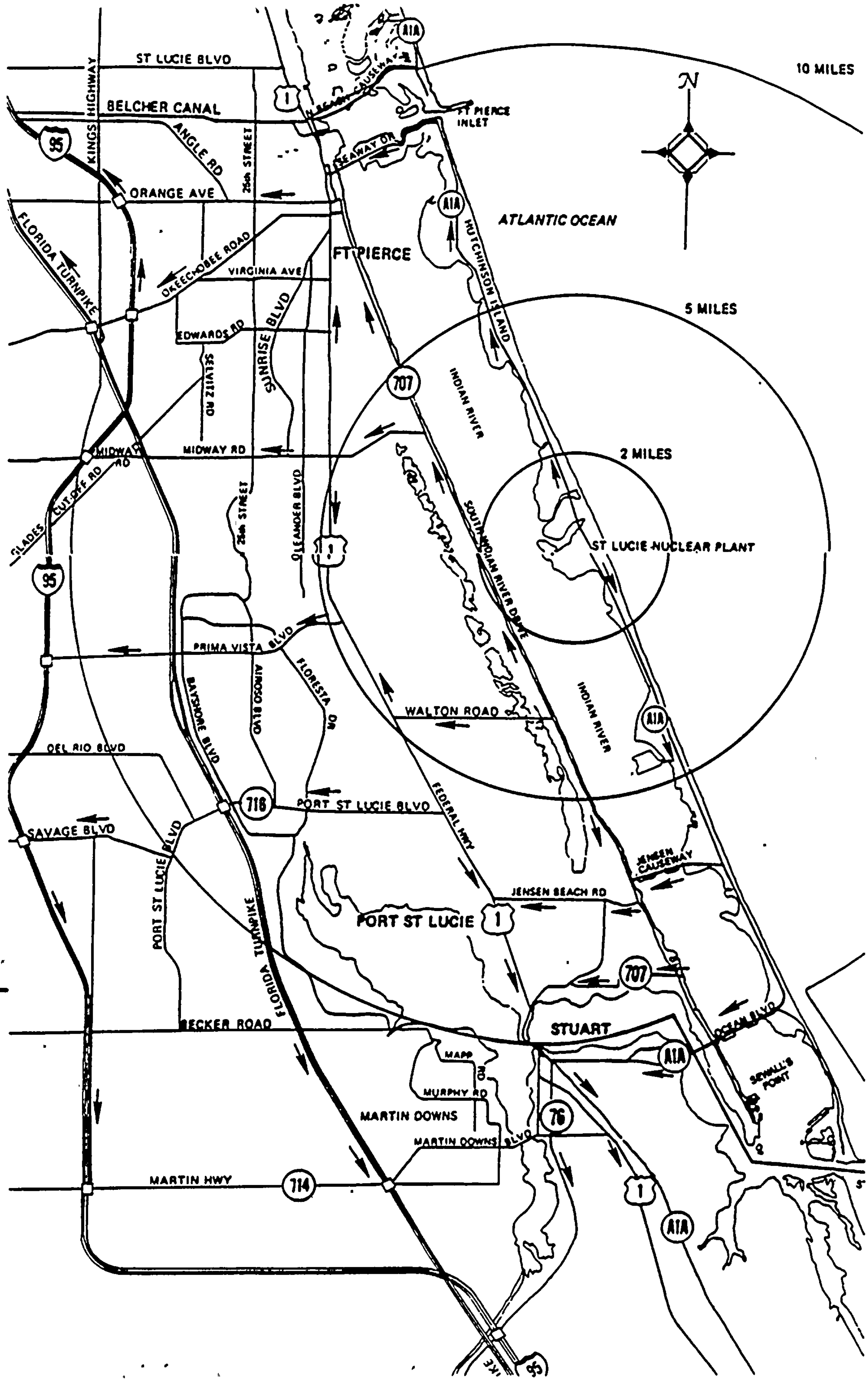
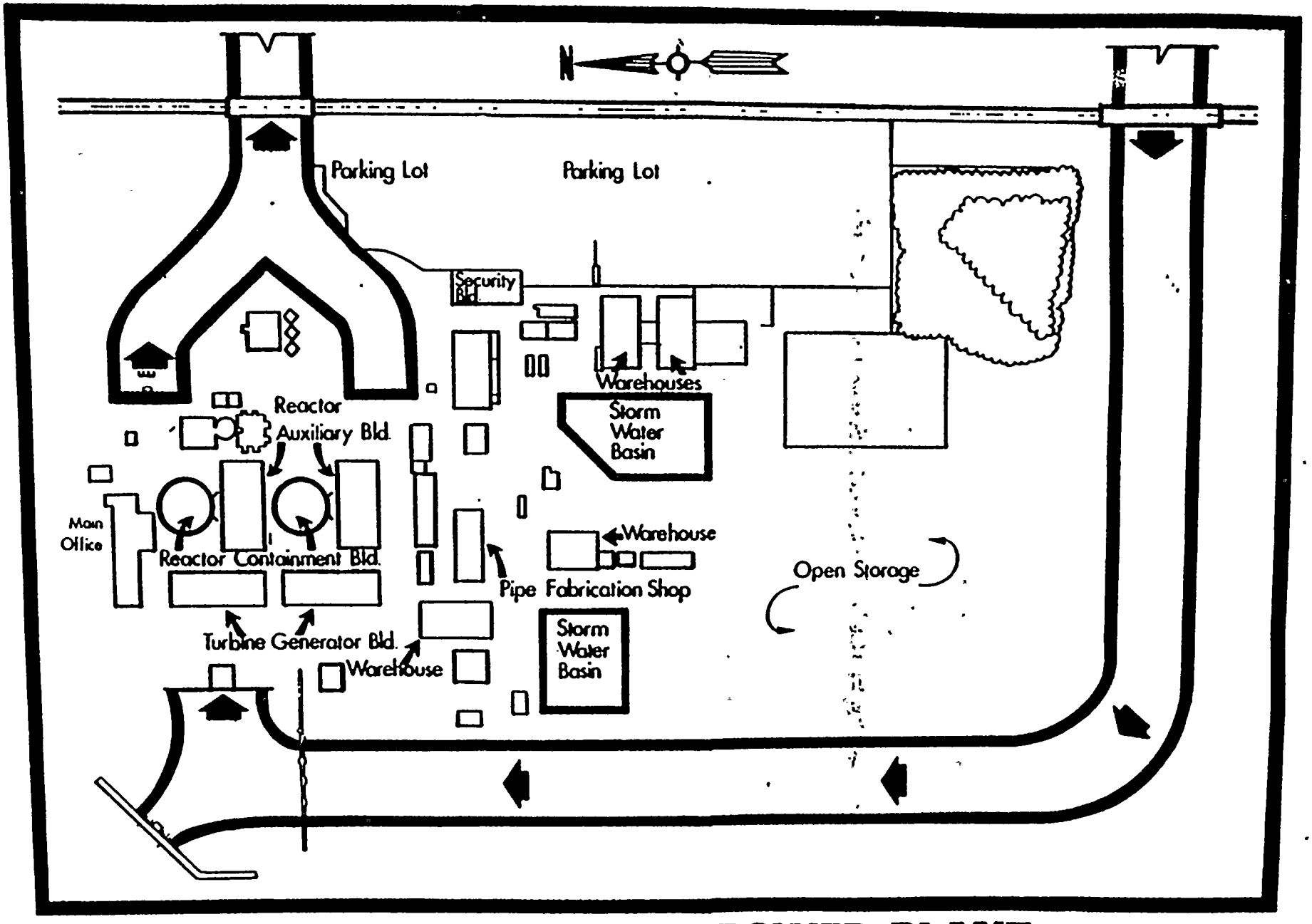


Figure 5: Emergency Operations Facility and News Center




10 MILES
5 MILES
2 MILES
ATLANTIC OCEAN
ST LUCIE NUCLEAR PLANT
FT PIERCE
PORT ST LUCIE
STUART
MARTIN DOWNS
INDIAN RIVER
INDIAN RIVER
HUTCHINSON ISLAND
ST LUCIE BLVD
BELCHER CANAL
ANGLE RD
KINGS HIGHWAY
25th STREET
ORANGE AVE
FT PIERCE INLET
SEAWAY DR
OKEECHOBEE ROAD
VIRGINIA AVE
FLORIDA TURNPIKE
EDWARDS RD
SUNRISE BLVD
SELVITZ RD
MIDWAY RD
MIDWAY RD
GLADES CUT-OFF RD
OLEANDER BLVD
SOUTH INDIAN RIVER DRIVE
PRIMA VISTA BLVD
AIROSO BLVD
FLORESTA DR
BAYSHORE BLVD
WALTON ROAD
DEL RIO BLVD
PORT ST LUCIE BLVD
FEDERAL HWY
SAVAGE BLVD
PORT ST LUCIE BLVD
FLORIDA TURNPIKE
JENSEN CAUSEWAY
JENSEN BEACH RD
BECKER ROAD
OCEAN BLVD
SEWALL'S POINT
MAPP RD
MURPHY RD
MARTIN DOWNS BLVD
MARTIN HWY
95
1
A1A
707
716
76
714



ST. LUCIE NUCLEAR POWER PLANT