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MECREDY,R.C. Rochester Gas & Electric Corp.
RECIP.NAME RECIPIENT AFFILIATION

JOHNSON,A. Project Directorate I-3

SUBJECT: Revised EOPs,including Rev 1 to ATT,Rev 2 to ATT,Rev 8 to AP-SW.1,Rev 6 to FR-S.1 & Rev 11 to FR-H.1.W/920626 ltr.

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Revised 7/16/92

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June 26, 1992

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Mr. Allen Johnson
Project Directorate I-3
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R. E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,

Robert C. Mecredy

Robert C. Mecredy

Enclosures
c: Mr. Lee Bettenhausen, USNRC, Region 1
Resident Inspector, Ginna Station

emergency.pro

Ltr No 1748978418

A005
1/1

	Abnormal Procedures Index - issued 6/23
	EOP/AP Attachment Index - 6/23/92
ATT	EOP/AP Attachments - Rev. 1 - 5/13/92 - SW Isolation
ATT	EOP/AP Attachments - SW Loads in CNMT
ATT	EOP/AP Attachments - Rev. 2 - 5/14/92 - Flow Through MFW Pump B
AP-SW.1	Service Water Leak - Rev. 8
AP-TURB.1	Turbine Trip Without RX Trip Required, Rev. 5
FR-S.1	Response to Reactor Restart/ATWS - Rev. 6
FR-H.1	Response to Loss of Secondary Heat Sink - Rev. 11

50-244 Superseded Per Rev's to Emergency Operating Procedures 2/3
Dtd 6/26/92 #9207090120

ABNORMAL PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
AP-CCW.1	LEAKAGE INTO THE COMPONENT COOLING LOOP	7	90/02/28
AP-CCW.2	LOSS OF CCW DURING POWER OPERATION	8	89/12/19
AP-CCW.3	LOSS OF CCW - PLANT SHUTDOWN	7	89/12/19
AP-CR.1	CONTROL ROOM INACCESSIBILITY	11	91/05/10
AP-CVCS.1	CVCS LEAK	5	89/12/19
AP-CVCS.2	IMMEDIATE BORATION	7	90/02/23
AP-CW.1	LOSS OF A CIRC WATER PUMP	4	90/02/23
AP-ELEC.1	LOSS OF #12A OR 12B TRANSFORMER	8	91/06/17
AP-ELEC.2	SAFEGUARD BUSSES LOW VOLTAGE OR SYSTEM LOW FREQUENCY	5	90/02/23

ABNORMAL PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
AP-FW.1	PARTIAL OR COMPLETE LOSS OF MAIN FEEDWATER	8	91/05/10
AP-IA.1	LOSS OF INSTRUMENT AIR	6	92/03/19
AP-PRZR.1	ABNORMAL PRESSURIZER PRESSURE	5	91/11/01
AP-RCC.1	CONTINUOUS CONTROL ROD WITHDRAWAL/INSERTION	4	90/02/23
AP-RCC.2	RCC/RPI MALFUNCTION	4	90/04/09
AP-RCP.1	RCP SEAL MALFUNCTION	6	89/11/17
AP-RCS.1	REACTOR COOLANT LEAK	7	91/11/01
AP-RCS.2	LOSS OF REACTOR COOLANT FLOW	5	90/02/23
AP-RCS.3	HIGH REACTOR COOLANT ACTIVITY	5	90/11/20

ABNORMAL PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
AP-RHR.1	LOSS OF RHR	8	90/06/01
AP-RHR.2	LOSS OF RHR WHILE OPERATING AT RCS REDUCED INVENTORY CONDITIONS	4	92/03/27
AP-SW.1	SERVICE WATER LEAK	7	90/02/23
AP-TURB.1	TURBINE TRIP WITHOUT RX TRIP REQUIRED	4	90/02/23
AP-TURB.2	AUTOMATIC TURBINE RUNBACK	10	91/10/11
AP-TURB.3	TURBINE VIBRATIONS	5	90/02/23
AP-TURB.4	LOSS OF CONDENSER VACUUM	6	90/02/23

EMERGENCY PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
E-0	REACTOR TRIP OR SAFETY INJECTION	16	91/05/03
E-1	LOSS OF REACTOR OR SECONDARY COOLANT	10	91/05/03
E-2	FAULTED STEAM GENERATOR ISOLATION	4	90/04/09
E-3	STEAM GENERATOR TUBE RUPTURE	12	91/07/26

EMERGENCY CONTINGENCY ACTIONS PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
ECA-0.0	LOSS OF ALL AC POWER	12	91/08/05
ECA-0.1	LOSS OF ALL AC POWER RECOVERY WITHOUT SI REQUIRED	5	91/05/03
ECA-0.2	LOSS OF ALL AC POWER RECOVERY WITH SI REQUIRED	6	90/04/09
ECA-1.1	LOSS OF EMERGENCY COOLANT RECIRCULATION	6	91/05/03
ECA-1.2	LOCA OUTSIDE CONTAINMENT	2	90/04/09
ECA-2.1	UNCONTROLLED DEPRESSURIZATION OF ALL STEAM GENERATORS	6	91/05/03
ECA-3.1	SGTR WITH LOSS OF REACTOR COOLANT-SUBCOOLED RECOVERY DESIRED	7	91/05/03
ECA-3.2	SGTR WITH LOSS OF REACTOR COOLANT-SATURATED RECOVERY DESIRED	9	91/05/03

EMERGENCY CONTINGENCY ACTIONS PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
ECA-3.3	SGTR WITHOUT PRESSURIZER PRESSURE CONTROL	6	91/05/03

EQUIPMENT SUB-PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
ES-0.0	REDIAGNOSIS	7	90/04/09
ES-0.1	REACTOR TRIP RESPONSE	7	90/12/19
ES-0.2	NATURAL CIRCULATION COOLDOWN	2	91/05/03
ES-0.3	NATURAL CIRCULATION COOLDOWN WITH STEAM VOID IN VESSEL (WITH RVLIS)	1	90/04/30
ES-1.1	SI TERMINATION	7	91/05/03
ES-1.2	POST LOCA COOLDOWN AND DEPRESSURIZATION	8	91/06/24
ES-1.3	TRANSFER TO COLD LEG RECIRCULATION	13	91/10/28
ES-3.1	POST-SGTR COOLDOWN USING BACKFILL	4	91/05/03
ES-3.2	POST-SGTR COOLDOWN USING BLOWDOWN	5	91/05/03

EQUIPMENT SUB-PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
ES-3.3	POST-SGTR COOLDOWN USING STEAM DUMP	5	91/05/03

CRITICAL SAFETY FUNCTION STATUS TREES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
F-0.1	SUBCRITICALITY CSFST	1	89/07/21
F-0.2	CORE COOLING CSFST	3	89/07/21
F-0.3	HEAT SINK CSFST	2	90/01/12
F-0.4	INTEGRITY CSFST	1	89/07/21
F-0.5	CONTAINMENT CSFST	2	90/01/12
F-0.6	INVENTORY CSFST	3	90/01/12

FUNCTIONAL RESTORATION GUIDELILNES PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
FR-C.1	RESPONSE TO INADEQUATE CORE COOLING	6	90/04/09
FR-C.2	RESPONSE TO DEGRADED CORE COOLING	5	90/04/09
FR-C.3	RESPONSE TO SATURATED CORE COOLING	4	90/04/09
FR-H.1	RESPONSE TO LOSS OF SECONDARY HEAT SINK	10	91/06/14
FR-H.2	RESPONSE TO STEAM GENERATOR OVERPRESSURE	2	90/04/09
FR-H.3	RESPONSE TO STEAM GENERATOR HIGH LEVEL	2	90/04/09
FR-H.4	RESPONSE TO LOSS OF NORMAL STEAM RELEASE CAPABILITIES	1	90/04/09
FR-H.5	RESPONSE TO STEAM GENERATOR LOW LEVEL	2	90/04/09
FR-I.1	RESPONSE TO HIGH PRESSURIZER		

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
	LEVEL	3	91/05/03
FR-I.2	RESPONSE TO LOW PRESSURIZER LEVEL	4	91/05/03
FR-I.3	RESPONSE TO VOIDS IN REACTOR VESSEL	5	91/05/03
FR-P.1	RESPONSE TO IMMINENT PRESSURIZED THERMAL SHOCK CONDITION	6	91/05/03
FR-P.2	RESPONSE TO ANTICIPATED PRESSURIZED THERMAL SHOCK CONDITION	2	90/04/09
FR-S.1	RESPONSE TO NUCLEAR POWER GENERATION/ATWS	4	90/04/09
FR-S.2	RESPONSE TO LOSS OF CORE SHUTDOWN	2	90/04/09
FR-Z.1	RESPONSE TO HIGH CONTAINMENT PRESSURE	2	90/04/09
FR-Z.2	RESPONSE TO CONTAINMENT FLOODING	2	90/04/09

FUNCTIONAL RESTORATION GUIDELILNES PROCEDURES INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
FR-Z.3	RESPONSE TO HIGH CONTAINMENT RADIATION LEVEL	2	90/04/09

EOP/AP ATTACHMENT INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
ATT	AUX BLDG SW	0	90/04/09
ATT	CI/CVI	1	90/04/09
ATT	CNMT RECIRC FANS	1	90/04/30
ATT	COND TO S/G	1	90/04/09
ATT	DC LOADS	2	91/05/10
ATT	D/G STOP	1	90/04/09
ATT	EXCESS L/D	1	90/04/30
ATT	FAULTED S/G	2	90/09/17
ATT	GEN DEGAS	3	91/05/10
ATT	LETDOWN	3	91/01/17

EOP/AP ATTACHMENT INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
ATT	N2 PORVS	1	90/04/09
ATT	NC	1	90/04/30
ATT	NONVITAL	1	90/04/09
ATT	RCP START	1	90/04/09
ATT	RCS ISOLATION	0	90/04/09
ATT	RHR COOL	1	90/04/09
ATT	RHR NPSH	0	91/05/10
ATT	RHR SYSTEM	1	90/04/30
ATT	RUPTURED S/G	3	91/09/05
ATT	SAFW	2	91/10/11

EOP/AP ATTACHMENT INDEX

ISSUED: 03/28/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
ATT	SD-1	2	90/06/27
ATT	SD-2	2	90/06/27
ATT	SEAL COOLING	1	90/04/09
ATT	SFP-RWST	1	90/04/30
ATT	SI FLUSH	2	91/09/05
ATT	SI/UV	1	90/04/09
ATT	VENT TIME	1	90/04/30
ATT	NORMAL RHR COOLING	0	92/03/27

EMERGENCY CONTINGENCY ACTIONS PROCEDURES INDEX

ISSUED: 01/08/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
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ECA-1.2	LOCA OUTSIDE CONTAINMENT	2	90/04/09
ECA-2.1	UNCONTROLLED DEPRESSURIZATION OF ALL STEAM GENERATORS	6	91/05/03
ECA-3.1	SGTR WITH LOSS OF REACTOR COOLANT-SUBCOOLED RECOVERY DESIRED	7	91/05/03
ECA-3.2	SGTR WITH LOSS OF REACTOR COOLANT-SATURATED RECOVERY DESIRED	9	91/05/03

EMERGENCY CONTINGENCY ACTIONS PROCEDURES INDEX

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CRITICAL SAFETY FUNCTION STATUS TREES INDEX

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F-0.4	INTEGRITY CSFST	1	89/07/21
F-0.5	CONTAINMENT CSFST	2	90/01/12
F-0.6	INVENTORY CSFST	3	90/01/12

FUNCTIONAL RESTORATION GUIDELINES PROCEDURES INDEX

ISSUED: 01/08/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
FR-C.1	RESPONSE TO INADEQUATE CORE COOLING	6	90/04/09
FR-C.2	RESPONSE TO DEGRADED CORE COOLING	5	90/04/09
FR-C.3	RESPONSE TO SATURATED CORE COOLING	4	90/04/09
FR-H.1	RESPONSE TO LOSS OF SECONDARY HEAT SINK	10	91/06/14
FR-H.2	RESPONSE TO STEAM GENERATOR OVERPRESSURE	2	90/04/09
FR-H.3	RESPONSE TO STEAM GENERATOR HIGH LEVEL	2	90/04/09
FR-H.4	RESPONSE TO LOSS OF NORMAL STEAM RELEASE CAPABILITIES	1	90/04/09
FR-H.5	RESPONSE TO STEAM GENERATOR LOW LEVEL	2	90/04/09
FR-I.1	RESPONSE TO HIGH PRESSURIZER		

FUNCTIONAL RESTORATION GUIDELINES PROCEDURES INDEX

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PROCEDURE	TITLE	REV	EFFECTIVE DATE
	LEVEL	3	91/05/03
FR-I.2	RESPONSE TO LOW PRESSURIZER LEVEL	4	91/05/03
FR-I.3	RESPONSE TO VOIDS IN REACTOR VESSEL	5	91/05/03
FR-P.1	RESPONSE TO IMMINENT PRESSURIZED THERMAL SHOCK CONDITION	6	91/05/03
FR-P.2	RESPONSE TO ANTICIPATED PRESSURIZED THERMAL SHOCK CONDITION	2	90/04/09
FR-S.1	RESPONSE TO NUCLEAR POWER GENERATION/ATWS	4	90/04/09
FR-S.2	RESPONSE TO LOSS OF CORE SHUTDOWN	2	90/04/09
FR-Z.1	RESPONSE TO HIGH CONTAINMENT PRESSURE	2	90/04/09
FR-Z.2	RESPONSE TO CONTAINMENT FLOODING	2	90/04/09

FUNCTIONAL RESTORATION GUIDELINES PROCEDURES INDEX

ISSUED: 01/08/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
FR-Z.3	RESPONSE TO HIGH CONTAINMENT RADIATION LEVEL	2	90/04/09

EOP/AP ATTACHMENT INDEX

ISSUED: 01/08/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
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EOP/AP ATTACHMENT INDEX

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ATT	NC	1	90/04/30
ATT	NONVITAL	1	90/04/09
ATT	RCP START	1	90/04/09
ATT	RCS ISOLATION	0	90/04/09
ATT	RHR COOL	1	90/04/09
ATT	RHR NPSH	0	91/05/10
ATT	RHR SYSTEM	1	90/04/30
ATT	RUPTURED S/G	3	91/09/05
ATT	SAFW	2	91/10/11

EOP/AP ATTACHMENT INDEX

ISSUED: 01/08/92

PROCEDURE	TITLE	REV	EFFECTIVE DATE
ATT	SD-1	2	90/06/27
ATT	SD-2	2	90/06/27
ATT	SEAL COOLING	1	90/04/09
ATT	SFP-RWST	1	90/04/30
ATT	SI FLUSH	2	91/09/05
ATT	SI/UV	1	90/04/09
ATT	VENT TIME	1	90/04/30

EOP: ATT	TITLE: EOP/AP ATTACHMENTS	REV: 1 PAGE 1 of 2

ATTACHMENT SD-1 Supt. Joseph Widay Date 4-4-90

Perform the following local actions to complete normal secondary system shutdown:

- Close Reheater 4th pass temperature control valves:
  - V-2432 (SW corner 1A MSR)
  - V-2433 (SW corner 1B MSR)
  - V-2434 (SW corner 2A MSR)
  - V-2435 (SW corner 2B MSR)

- Close reheater steam chain valves:
  - V-3550
  - V-3551
  - V-3552
  - V-3553

- Open Reheater steamline vents (SW corner of condenser, middle floor)
  - V-8500
  - V-8501
  - V-8502
  - V-8504
  - V-8505

- Close flange heating isolation valves:
  - MOV-3601A
  - MOV-3602A

- Open the following valves to align for condensate feed system cooldown RECIRC:
  - V-3982B (at #5 heater outlet header)
  - V-3983B (at #5 heater outlet header)
  - V-4363 (at #5 heater outlet header)
  - V-4365 (by MFW regulating valves)
  - V-4361 (southwest corner of condenser, middle floor)
  - V-3976A A MFP discharge valve bypass valve
  - V-3977 B MFP discharge valve bypass valve

- Secure all 5 secondary chemical addition pumps on TURB BLDG middle floor by #5 heaters.

- Secure all 3 ammonia pumps, TUB BLDG basement by MCC A.

- Isolate SW from the following coolers:
  - Main Feed Pump Oil Coolers:
    - V-4703
    - V-4704
  - Exciter Air Cooler:
    - V-4679 (chain valve next to condensate transfer pump)
  - Bus Duct Air Cooler
    - V-4674
    - V-4674C (mini bypass around V-4674)

ATTACHMENT SD-1 (Cont'd)

- Throttle SW as necessary from following coolers:
  - Generator Seal Oil Unit Coolers (H2 side and air side):
    - V-4676A (mini bypass disch valve inside seal oil enclosure Bldg. NW corner)
    - V-4677A (mini bypass disch valve inside seal oil enclosure Bldg. NW corner)
  - Main Lube Oil Coolers (SW corner of Turb Oil Reservoir)
    - V-4691
    - V-4692

- Notify Control Room when turbine shaft stops. Control Room personnel will determine if adequate power available to start turning gear.

- Transfer house heating steam to house heating boiler if necessary (refer to T-35H, NUCLEAR HOUSE HEATING STEAM TO BOILER STEAM SUPPLY CHANGE).

- Perform T-14G, STEAM GENERATOR BLOWDOWN HEAT RECOVERY SYSTEM SHUTDOWN.

- Restore MAKEUP to CSTs as directed by Control Room.

EOP: ATT	TITLE: EOP/AP ATTACHMENTS	REV: 1 PAGE 1 of 1

ATTACHMENT LETDOWN Supt. Joseph A. Widay Date 1-4-90

A) The following conditions must be met to place either normal or excess letdown in service:
   - o IA to CNMT - ESTABLISHED
   - o CCW - IN SERVICE
   - o PRZR level - GREATER THAN 13%.

B) Establish Normal Letdown:

   1. Establish charging line flow to REGEN Hx - GREATER THAN 20 gpm.

   2. Place the following switches to CLOSE:
      o Letdown orifice valves (AOV-200A, AOV-200B, and AOV-202)
      o AOV-427, loop B cold leg to REGEN Hx

   3. Place letdown controllers TCV-130 and PCV-135 in MANUAL at 25% open.

   4. Verify AOV-371, letdown isolation valve - OPEN.

   5. Open loop B cold leg to REGEN Hx, AOV-427.

   6. Open one 40 gpm letdown orifice valve (AOV-200A or AOV-200B).

   7. Place TCV-130 in AUTO at 105°F.

   8. Place PCV-135 in AUTO at 250 psig.

   9. Adjust charging pump speed and HCV-142 as necessary.

C) IF normal letdown can NOT be established, THEN establish excess letdown:

   1. Place AOV-312 to NORMAL.

   2. Ensure CCW from excess letdown Hx, (AOV-745) - OPEN

   3. Open excess letdown isolation valve AOV-310.

   4. Slowly open HCV-123 to maintain excess letdown temperature less than 195°F and pressure less than 100 psig.

   5. Adjust charging pump speed as necessary.

ATTACHMENT RUPTURED S/G (CONT)

****
CAUTION

PART B OF THIS ATTACHMENT SHOULD ONLY BE PERFORMED IF RUPTURED S/G MSIV CANNOT BE CLOSED.
****

PART B. Dispatch AO to locally perform the following when a ruptured S/G MSIV cannot be closed, if areas are accessible:

1) Close Air Ejector/Gland steam root valve, V-3540 (Main steam header TURB BLDG).
2) Close flange heating isolation valves, MOV-3601A and MOV-3602A.
3) Verify MSIV bypass valves shut.
4) Notify Control Room that main flowpaths are isolated, THEN complete isolation by closing the following valves:
   - o Support heating steam valves, V-3669 and V-3668, (INT BLDG steam header area)
   - o Steam to sampling system valves, V-3413A and V-3412A, (INT BLDG steam header area)
   - o Upstream traps isolation valves, V-3520 and V-3521, (TURB BLDG east of MFW regulating valves)
   - o MFW regulating valve and bypass valve manual isolation valves for both S/Gs:
     S/G A, V-3985 and V-3989
     S/G B, V-3984 and V-3988
   - o Steam to trap header isolation valves
     - o V-8513 (Main steam header TURB BLDG)
     - o V-8529 (south side EH skid)
     - o V-8510 (south side EH skid)
   - o Steam dump header isolation and bypass valves (Main steam header TURB BLDG on platform overhead)
     - o V-3532 and V-3658
     - o V-3533 and V-3659
   - o Reheat steam chain valves and reheat steamline warmup valves (warmup vlvs located east end of 1A and 2A MSRs TURB BLDG middle floor)
     - o 1A MSR, V-3551 and V-3645
     - o 1B MSR, V-3550 and V-3646
     - o 2A MSR, V-3553 and V-3647
     - o 2B MSR, V-3552 and V-3648