APPENDIX A

REPORT RADIOACTIVE EFFLUENTS

FACILITY: ROCHESTER GAS & ELECTRIC GINNA STATION DOCKET: 50-244 YEAR: 1981

I. LIQUID RELEASES

	UNITS	JAN.	FEB.	MARCH	APRIL	MAY	JUNE	TOTAL
1. GROSS RADIOACTIVITY (β,γ)								
A) TOTAL RELEASED (except 2,3 & 4)	CURIES	3.52 E-3	1.62 E-3	1.32 E-3	5.67 E-3	1.53 E-3	7.39 E-3	2.11 E-2
B) AVERAGE CONCENTRATION RELEASED	µCI/ML	5.07 E-7	4.98 E-7	1.22 E-6	1.05 E-6	2.5 E-6	1.9 E-6	N/A
C) MAXIMUM CONCENTRATION RELEASED	µCI/ML	2.6 E-6	1.33 E-6	3.19 E-5	1.1 E-5	3.5 E-5	4.06 E-5	N/A
2. TRITIUM								
A) TOTAL RELEASED	CURIES	6.6	2.71	12.5	48.03	43.8	21.64	135.28
B) AVERAGE CONCENTRATION RELEASED	µCI/ML	1.22 E-7	5.13 E-8	2.1 E-7	9.26 E-7	1.3 E-6	4.96 E-7	N/A
C) PERCENT OF LIMIT	%	0.004	0.002	0.007	0.031	0.043	0.017	N/A
3. DISSOLVED NOBLE GASES								
A) TOTAL RELEASED	CURIES	ND	ND	ND	ND	ND	ND	ND
B) AVERAGE CONCENTRATION RELEASED	µCI/ML	-------	-------	-------	-------	-------	-------	N/A
4. GROSS ALPHA RADIOACTIVITY								
A) TOTAL RELEASED	CURIES	ND	ND	ND	ND	ND	ND	ND
B) AVERAGE CONCENTRATION RELEASED	µCI/ML	-------	-------	-------	-------	-------	-------	N/A
5. VOLUME OF LIQUID WASTE TO DISCHARGE CANAL	LITERS	3.88 E 6	3.77 E 6	3.46 E 6	2.75 E 6	7.37 E 5	4.1 E 6	1.87 E-7
6. VOLUME OF DILUTION WATER	LITERS	5.37 E 10	5.28 E 10	5.98 E 10	5.19 E 10	3.38 E 10	4.36 E 10	2.96 E11
7. ISOTOPES RELEASED	CURIES							
Cerium 141	"	ND	ND	ND	ND	7.1 E-6	ND	7.1 E-6
Ruthenium 103	"	ND	ND	ND	ND	ND	ND	ND
Cesium 137	"	1.24 E-3	2.21 E-4	3.95 E-5	2.91 E-3	8.6 E-4	4.47 E-3	9.74 E-3
Cesium 134	"	1.10 E-3	8.7 E-4	2.87 E-4	1.94 E-3	2.65 E-4	1.81 E-3	6.27 E-3
Cobalt 58	"	1.42 E-5	4.32 E-6	6.52 E-4	3.79 E-6	5.38 E-5	9.0 E-5	8.18 E-4
Cobalt 60	"	1.49 E-5	5.25 E-4	ND	1.13 E-4	3.4 E-4	1.01 E-3	2.00 E-3
Manganese 54	"	ND	ND	9.67 E-5	ND	ND	1.03 E-5	1.07 E-4
Zirconium 95	"	ND	ND	ND	ND	ND	ND	ND
Niobium 95	"	ND	ND	ND	ND	ND	ND	ND
Cerium 144/Praseodymium 144	"	3.6 E-4	ND	2.47 E-4	7.0 E-4	ND	ND	1.31 E-3
Tellurium 132	"	ND	ND	ND	ND	ND	ND	ND
Ruthenium 106	"	ND	ND	ND	ND	ND	ND	ND
Silver 110m	"	ND	ND	ND	ND	ND	ND	ND
Molybdenum 99	"	ND	ND	ND	ND	ND	ND	ND
Barium 140	"	ND	ND	ND	ND	ND	ND	ND
Chromium 51	"	ND	ND	ND	ND	ND	ND	ND
Iron 59	"	ND	ND	ND	ND	ND	ND	ND
Tellurium 129m	"	ND	ND	ND	ND	ND	ND	ND
Cesium 136	"	ND	ND	ND	ND	ND	ND	ND
Neptunium 239	"	ND	ND	ND	ND	ND	ND	ND
Iodine 131	"	7.81 E-4	ND	ND	ND	ND	ND	7.81 E-4
Tellurium 127m	"	ND	ND	ND	ND	ND	ND	ND
Barium/Lanthanum 140	"	ND	ND	ND	ND	ND	ND	ND
Zinc 65	"	ND	ND	ND	ND	ND	ND	ND
Strontium 90	"	ND	ND	ND	ND	ND	ND	ND
Antimony 125	"	ND	ND	ND	ND	ND	ND	ND
PRECENT OF TECHNICAL SPECIFICATION LIMIT FOR ACTIVITY RELEASED	%	0.005	<0.001	<0.001	0.005	<0.001	0.001	N/A

APPENDIX A

REPORT RADIOACTIVE EFFLUENTS

FACILITY: ROCHESTER GAS & ELECTRIC GINNA STATION DOCKET: 50-244 YEAR: 1981

II. AIRBORNE RELEASES

	UNITS	JAN.	FEB.	MARCH	APRIL	MAY	JUNE	TOTAL
1. TOTAL NOBLE GASES	CURIES	40.56	43.4	67.64	105.8	68.23	21.70	347.33
2. TOTAL HALOGENS	CURIES	6.62 E-5	1.15 E-5	1.54 E-5	4.5 E-4	4.23 E-3	1.09 E-4	4.88 E-3
3. TOTAL PARTICULATE GROSS RADIOACTIVITY (β, γ)	CURIES	1.35 E-7	2.37 E-7	3.99 E-7	2.6 E-7	1.23 E-6	1.34 E-6	3.6 E-6
4. TOTAL TRITIUM	CURIES	3.00	1.81	5.13	2.85	11.5	2.82	27.11
5. TOTAL PARTICULATE GROSS ALPHA RADIOACTIVITY	CURIES	ND	ND	ND	ND	ND	ND	ND
6. MAXIMUM NOBLE GAS RELEASE RATE	μCI/SEC	230.	106.	165.	1610.	1580.	28.3	N/A
7. PERCENT OF APPLICABLE LIMIT FOR:								
A) NOBLE GASSES	%	0.027	0.032	0.051	0.055	0.043	0.015	N/A
B) HALOGENS	%	0.035	0.028	0.037	0.39	0.68	0.076	N/A
C) PARTICULATES	%	<0.0001	<0.0001	<0.0001	<0.0001	0.0002	0.0009	N/A
8. ISOTOPE RELEASED:	CURIES							
PARTICULATES	"							
Cesium 137	"	ND	ND	1.21 E-7	7.83 E-7	2.44 E-7	9.33 E-7	1.15 E-6
Cobalt 60	"	ND	ND	1.29 E-7	6.3 E-7	4.47 E-7	1.14 E-6	2.35 E-6
Barium/Lanthanum 140	"	ND	ND	ND	ND	ND	6.55 E-7	6.55 E-7
Cobalt 58	"	ND	ND	ND	3.26 E-7	ND	2.71 E-7	5.97 E-7
Cesium 134	"	ND	ND	ND	1.86 E-7	7.5 E-8	3.45 E-8	2.96 E-7
Ruthenium 103	"	ND	ND	ND	ND	1.14 E-7	ND	1.14 E-7
Antimony 125	"	ND	ND	1.72 E-7	ND	ND	ND	1.72 E-7
Niobium 95	"	ND	ND	ND	5.03 E-8	ND	ND	5.03 E-8
Chromium 51	"	ND	ND	ND	ND	ND	1.2 E-6	1.2 E-6
Iodine 131	"	ND	ND	ND	ND	ND	3.66 E-7	3.66 E-7
HALOGENS	CURIES							
Iodine 131	"	2.69 E-5	1.95 E-5	2.88 E-5	2.92 E-4	5.3 E-4	5.7 E-5	9.54 E-4
Iodine 132	"	ND	ND	ND	1.34 E-4	3.7 E-3	ND	3.83 E-3
Iodine 133	"	3.93 E-5	9.56 E-5	1.25 E-4	1.58 E-4	ND	5.17 E-5	4.7 E-4
Iodine 135	"	ND	ND	ND	ND	ND	ND	ND
GASES	CURIES							
Xenon 135m	"	0.108	0.099	0.073	0.027	ND	0.028	0.335
Argon 41	"	0.024	0.024	0.263	0.30	ND	0.009	0.62
Xenon 133m	"	0.013	0.004	ND	0.29	ND	0.002	0.309
Xenon 131m	"	0.027	3.21	ND	2.80	0.14	0.11	6.287
Xenon 133	"	38.6	36.2	60.1	90.5	67.45	17.95	310.8
Krypton 85	"	0.434	0.507	ND	7.20	0.64	2.49	11.271
Xenon 135	"	1.29	2.31	5.9	4.69	ND	1.10	15.29
Krypton 85m	"	0.013	0.011	0.142	0.005	ND	0.003	0.175
Krypton 87	"	0.015	0.014	0.005	0.012	ND	0.003	0.049
Krypton 88	"	0.026	0.014	0.009	0.004	ND	0.004	0.057
Xenon 138	"	0.010	0.013	1.161	0.002	ND	0.003	1.189
OTHERS AS APPROPRIATE (SPECIFY)	CURIES							
	"							
	"							
	"							

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON D.C. 20555

Docket No. 50-219 & 50-244

June 12, 1981

DISTRIBUTION:
Dockets (2)
ORB #5 RF
HSmith

SEE ATTACHED LISTING

Subject: Oyster Creek Nuclear Power Plant located in Ocean County, NJ
R. E. Ginna Nuclear Power Plant located in Wayne County, NY

The following documents concerning our review of the subject facility are transmitted for your information:

- ☐ Notice of Receipt of Application.
- ☐ Draft/Final Environmental Statement, dated _____.
- ☐ Safety Evaluation, or Supplement No._____,dated _____.
- ☐ Notice of Hearing on Application for Construction Permit.
- ☐ Notice of Consideration of Issuance of Facility Operating License.
- ☐ Application and Safety Analysis Report, Vol. _____.
- ☐ Amendment No. _____to Application/SAR, dated _____.
- ☐ Construction Permit No. CPPR-_____,dated_____.
- ☐ Facility Operating License No. DPR-_____, NPF-_____, dated_____.
- ☐ Amendment No. _____ to CPPR-_____ or DRR-_____,dated _____.
- ☒ Other: 1) Monthly operating report for 5/81 dated 5/18/81
- ☐ 2) Annual report of changes to station facilities & procedures dtd. 4/29/81

Hazel Smith /rj

Office of Nuclear Reactor Regulation
Division of Licensing
Operating Reactors Branch #5

Enclosures:
As stated

cc:

JUN 16 1981

OFFICE ►	DL:ORB #5/LA					
SURNAME ►	HSmith:rj					
DATE ►	6/12/81					

NRC—21 (6-76)

Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Director
National Oceanographic Data Center
Environmental Data Service
National Oceanic & Atomospheric Administration
U. S. Department of Commerce
Washington, D. C. 20234

Dr. William B. Stroube, Jr.
FDA Research Chemist
National Bureau of Standards
Reactor Building 235, Rm. B108
Washington, D. C. 20234

Director, Criteria and Standards
Division
Office of Radiation Programs
(ANR-460)
U. S. Environmental Protection
Agency
Washington, D. C. 20460

EPA Region

U. S. Environmental Protection
Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

OFFICE OF THE
SECRETARY

April 21, 1981

Director
Office of the Federal Register
National Archives and Records Service
Washington, D. C. 20403

Dear Sir:

Enclosed for publication in the Federal Register are an original and two certified copies of a document entitled:

Rochester Gas and Electric Corporation
R. E. Ginna Nuclear Power Plant, 50-244
Order for Modification of License

This material is to be charged to requisition number G-132, JN 340-371.

Sincerely,

Samuel J. Chilk
Secretary of the Commission

Enclosures:
Original and 2 certified copies

bcc: Record Services Branch
Office of Public Affairs
Executive Legal Director
Office of Congressional Affairs
Office of the General Counsel

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of	)	
	)	
ROCHESTER GAS & ELECTRIC CORP.	)	
	)	Docket No. 50-244
R. E. Ginna Nuclear Power Plant	)	
	)	
	)	

ORDER FOR MODIFICATION OF LICENSE

I

The Rochester Gas and Electric Corporation (the licensee) holds Provisional Operating License No. DPR-18, which authorizes the licensee to operate the R. E. Ginna Nuclear Power Plant (the facility) at power levels not in excess of 1520 megawatts (thermal) rated power. The facility, which is located at the licensee's site in Wayne County, New York is a pressurized water reactor (PWR) used for the commercial generation of electricity.

II

The Reactor Safety Study (RSS), WASH-1400, identified in a PWR an inter-system loss of coolant accident (LOCA) which is a significant contributor to risk of core melt accidents (Event V). The design examined in the RSS contained in-series check valves isolating the high pressure Primary Coolant System (PCS) from the Low Pressure Injection System (LPIS) piping. The scenario which leads to the Event V accident is initiated by the failure of these check valves to function as a pressure isolation barrier. This causes an overpressurization and rupture of the LPIS low pressure piping which results in a LOCA that bypasses containment.

In order to better define the Event V concern, all light water reactor licensees were requested by letter dated February 23, 1980, to provide the following in accordance with 10 CFR 50.54(f):

1. Describe the valve configurations and indicate if an Event V isolation valve configuration exists within the Class I boundary of the high pressure piping connecting PCS piping to low pressure system piping; e.g., (1) two check valves in series, or (2) two check valves in series with a motor operated valve (MOV);
2. If either of the above Event V configurations exist, indicate whether continuous surveillance or periodic tests are being performed on such valves to ensure integrity. Also indicate whether valves have been known, or found, to lack integrity; and
3. If either of the above Event V configurations exist, indicate whether plant procedures should be revised or if plant modifications should be made to increase reliability.

In addition to the above, licensees were asked to perform individual check valve leak testing prior to plant startup after the next scheduled outage.

By letter dated March 14, 1980, the licensee responded to our February letter. Based upon the review of this response as well as the review of previously docketed information for the facility, I have concluded in consonance with the attached Safety Evaluation (Attachment 1) that one or more valve configuration(s) of concern exist at the facility. The attached Technical Evaluation Report (TER) (Attachment 2) provides, in Section 4.0, a tabulation of the subject valves.

The staff's concern has been exacerbated due not only to the large number of plants which have an Event V configuration(s) but also because of recent unsatisfactory operating experience. Specifically, two plants have leak tested check valves with unsatisfactory results. At Davis-Besse, a pressure isolation check valve in the LPIS failed and the ensuing investigation found that valve internals had become disassembled. At the Sequoyah Nuclear Plant, two Residual Heat Removal (RHR) injection check valves and one RHR recirculation check valve failed because valves jammed open against valve over-travel limiters.

It is, therefore, apparent that when pressure isolation is provided by two in-series check valves and when failure of one valve in the pair can go undetected for a substantial length of time, verification of valve integrity is required. Since these valves are important to safety, they should be tested periodically to ensure low probability of gross failure. As a result, I have determined that periodic examination of check valves must be undertaken by the licensee as provided in Section III below to verify that each valve is seated properly and functioning as a pressure isolation device. Such testing will reduce the overall risk of an inter-system LOCA. The testing mandated by this Order may be accomplished by direct volumetric leakage measurement or by other equivalent means capable of demonstrating that leakage limits are not exceeded in accordance with Section 2.2 of the attached TER.

In view of the operating experiences described above and the potential consequences of check valve failure, I have determined that prompt action is necessary to increase the level of assurance that multiple pressure isolation barriers are in place and will remain intact. Therefore, the public health, safety and interest require that this modification of Provisional Operating License No. DPR-18 be immediately effective.

III

Accordingly, pursuant to Section 161i of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED THAT EFFECTIVE IMMEDIATELY, Provisional Operating License No. DPR-18 is modified by the addition of the following requirements:

1. Implement Technical Specifications (Attachment 3) which require periodic surveillance over the life of the plant and which specify limiting conditions for operation for PCS pressure isolation valves.
2. If check valves have not been (a) individually tested within 12 months preceding the date of the Order, and (b) found to comply with the leakage rate criteria set forth in the Technical Specifications described in Attachment 3, the MOV in each line shall be closed within 30 days of the effective date of this Order and quarterly Inservice Inspection (ISI). MOV cycling ceased until the check valve tests have been satisfactorily accomplished. (Prior to closing the MOV, procedures shall be implemented and operators trained to assure

that the MOV remains closed. Once closed, the MOV shall be tagged closed to further preclude inadvertent valve opening).

3. The MOV shall not be closed as indicated in paragraph 2 above unless a supporting safety evaluation has been prepared. If the MOV is in an emergency core cooling system (ECCS), the safety evaluation shall include a determination as to whether the requirements of 10 CFR 50.46 and Appendix K to 10 CFR Part 50 will continue to be satisfied with the MOV closed. If the MOV is not in an ECCS, the safety evaluation shall include a determination as to whether operation with the MOV closed presents an unreviewed safety question as defined in 10 CFR 50.59(a)(2). If the requirements of 10 CFR 50.46 and Appendix K have not been satisfied, or if an unreviewed safety question exists as defined in 10 CFR 50.59, then the facility shall be shut down within 30 days of the date of this Order and remain shutdown until check valves are satisfactorily tested in accordance with the Technical Specifications set forth in Attachment 3.
4. The records of the check valve tests required by this Order shall be made available for inspection by the NRC's Office of Inspection and Enforcement.

IV

The licensee or any other person who has an interest affected by this Order may request a hearing on this Order within 25 days of its publication in the Federal Register. A request for hearing shall be submitted to the Secretary, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy of the request shall also be sent to the Executive Legal Director at the same address, and to Harry H. Voigt, Esquire, LeBoeuf, Lamb, Leiby & MacRae, 1333 New Hampshire Avenue, N. W. Suite 1100, Washington, D. C. 20036, attorney for the licensee. If a hearing is requested by a person other than the licensee, that person shall describe, in accordance with 10 CFR 2.714(a)(2), the manner in which his or her interest is affected by this Order. ANY REQUEST FOR A HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is requested by the licensee or other person who has an interest affected by this Order, the Commission will issue an order designating the time and place of any such hearing. If a hearing is held, the issues to be considered at such a hearing shall be:

(a) Whether the licensee should be required to individually leak test check valves in accordance with the Technical Specifications set forth in Attachment 3 to this Order.

(b) Whether the actions required by Paragraphs 2 and 3 of Section III of this Order must be taken if check valves have not been tested within 12 months preceding the date of this Order.

Operation of the facility on terms consistent with this Order is not stayed by the pendency of any proceedings on this Order. In the event that a need for further action becomes apparent, either in the course of proceedings on this Order or any other time, the Director will take appropriate action.

FOR THE NUCLEAR REGULATORY COMMISSION

Darrell G. Eisenhut, Director
Division of Licensing

Effective Date: April 20, 1981
Bethesda, Maryland

Attachments:
1. Safety Evaluation Report
2. Technical Evaluation Report
3. Technical Specifications

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

OFFICE OF THE
SECRETARY

April 8, 1981

Director
Office of the Federal Register
National Archives and Records Service
Washington, D. C. 20403

Dear Sir:

Enclosed for publication in the Federal Register are an original and two certified copies of a document entitled:

ROCHESTER GAS AND ELECTRIC CORPORATION

Docket No. 50-244

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL OPERATING LICENSE

Publication of the above document at the earliest possible date would be appreciated.

This material is to be charged to requisition number G-132, JN 340-371.

Sincerely,

Samuel J. Chilk
Secretary of the Commission

Enclosures:
Original and 2 certified copies

bcc: Record Services Branch
Office of Public Affairs
Executive Legal Director
Office of Congressional Affairs
Office of the General Counsel

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-244

ROCHESTER GAS AND ELECTRIC CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 39 to Provisional Operating License No. DPR-18, issued to Rochester Gas and Electric Corporation (the licensee), which revised the Technical Specifications for operation of the R. E. Ginna Nuclear Power Plant (the facility) located in Wayne County, New York. The amendment is to become effective June 30, 1981.

The amendment modifies the Technical Specifications to reflect modifications of the Fire Protection System being completed at the facility.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment notarized February 11, 1981 (submitted by letter dated February 13, 1981), and (2) Amendment No. 39 to License No. DPR-18, including the Commission's letter of transmittal. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D.C., and at the Rochester Public Library, 115 South Avenue, Rochester, New York 14627. A copy of item (2) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this first day of April, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

OFFICE OF THE
SECRETARY

April 6, 1981

Director
Office of the Federal Register
National Archives and Records Service
Washington, D. C. 20403

Dear Sir:

Enclosed for publication in the Federal Register are an original and two certified copies of a document entitled:

ROCHESTER GAS AND ELECTRIC CORPORATION

Docket No. 50-244

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL OPERATING LICENSE

Publication of the above document at the earliest possible date would be appreciated.

This material is to be charged to requisition number G-132, JN 340-371.

Sincerely,

Samuel J. Chilk
Secretary of the Commission

Enclosures:
Original and 2 certified copies

bcc: Record Services Branch
Office of Public Affairs
Executive Legal Director
Office of Congressional Affairs
Office of the General Counsel

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-244

ROCHESTER GAS AND ELECTRIC CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO
PROVISIONAL OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 38 to Provisional Operating License No. DPR-18, to Rochester Gas and Electric Corporation (the licensee), which revised the technical specifications for operation of the R.E. Ginna Plant (facility) located in Wayne County, New York. This amendment is effective as of the date of its issuance.

The amendment incorporates limiting conditions for operation and surveillance requirements regarding degraded grid voltage protection for the Class 1E power system.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 9, 1980 (transmitted by letter dated September 15, 1980), as preceded and supported by submittals dated July 21, 1977, November 21, 1977, December 16, 1977 (transmitted by letter dated December 22, 1977), July 31, 1979 (transmitted August 3, 1979 - two separate submittals), and December 19, 1979, (2) Amendment No. 38 to License No. DPR-18, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Rochester Public Library, 115 South Avenue, Rochester, New York 14627. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 26th day of March, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

OFFICE OF THE
SECRETARY

March 16, 1981

Director
Office of the Federal Register
National Archives and Records Service
Washington, D. C. 20403

Dear Sir:

Enclosed for publication in the Federal Register are an original and two certified copies of a document entitled:

ROCHESTER GAS AND ELECTRIC CORPORATION

Docket No. 50-244

NOTICE OF ISSUANCE OF AMENDMENT TO OPERATING LICENSE

Publication of the above document at the earliest possible date would be appreciated.

This material is to be charged to requisition number G-132, JN 340-371.

Sincerely,

Samuel J. Chilk
Secretary of the Commission

Enclosures:
Original and 2 certified copies

bcc: Record Services Branch
Office of Public Affairs
Executive Legal Director
Office of Congressional Affairs
Office of the General Counsel

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-244

ROCHESTER GAS AND ELECTRIC CORPORATION

NOTICE OF ISSUANCE OF AMENDMENT TO OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 37 to Provisional Operating License No. DPR-18 to Rochester Gas and Electric Corporation (the licensee) which revised the license for operation of the R. E. Ginna Nuclear Plant located in Wayne County, New York. The amendment is effective as of the date of issuance and the portion relating to Safeguards Contingency is to be fully implemented within 30 days of Commission approval in accordance with the provisions of 10 CFR 73.40(b).

The amendment adds a license condition to include the Commission-approved Safeguards Contingency Plan as part of the license, and revises the license condition relating to the Security Plan to incorporate approved changes.

The licensee's filings comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

The licensee's filings (transmitted by letters dated April 3, 1980, July 24, 1980, and September 10, 1980) are being withheld from public disclosure pursuant to 10 CFR 2.790(d). The withheld information is subject to disclosure in accordance with the provisions of 10 CFR §9.12.

For further details with respect to this action, see (1) Amendment No. to License No. DPR-18, and (2) the Commission's related letter to the licensee dated March 11 , 1981. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Rochester Public Library, 115 South Avenue, Rochester, New York 14627. A copy of items (1) and (2) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C., 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 11th day of March, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing