

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-528

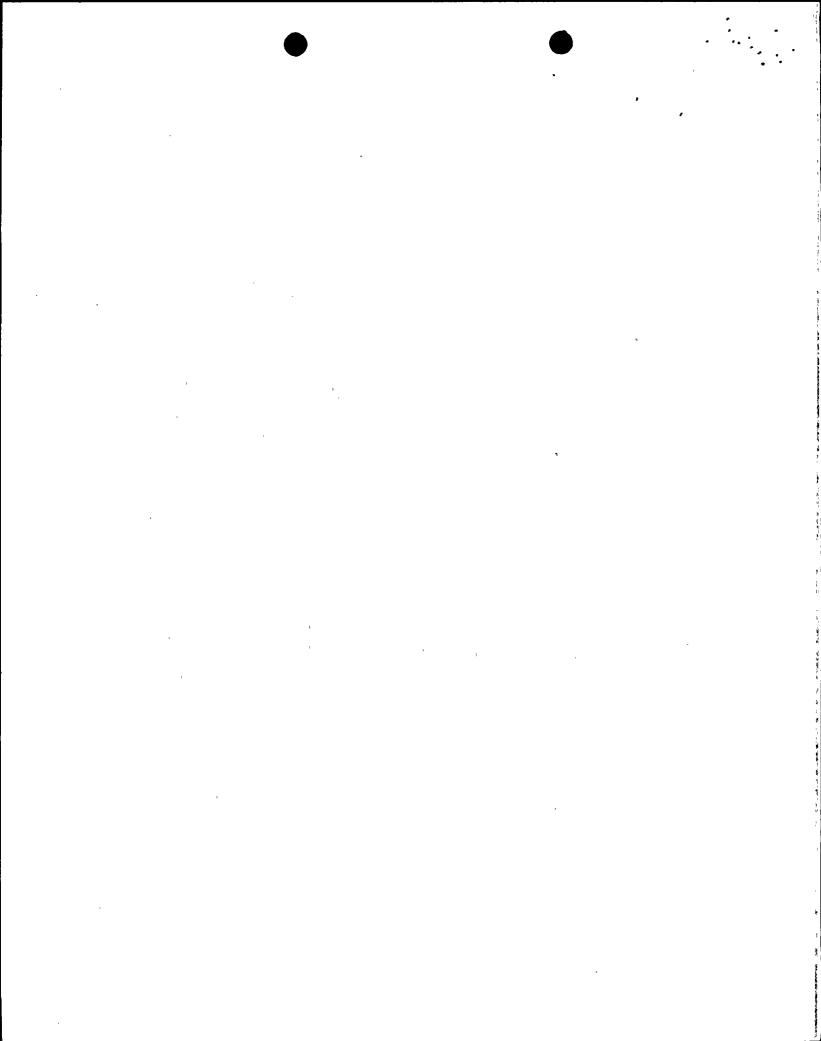
PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 51 License No. NPF-41

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment dated July 26, 1989 as superseded March 16, 1990 by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-41 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 51, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective within 45 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John T. Larkins, Acting Director Project Directorate V

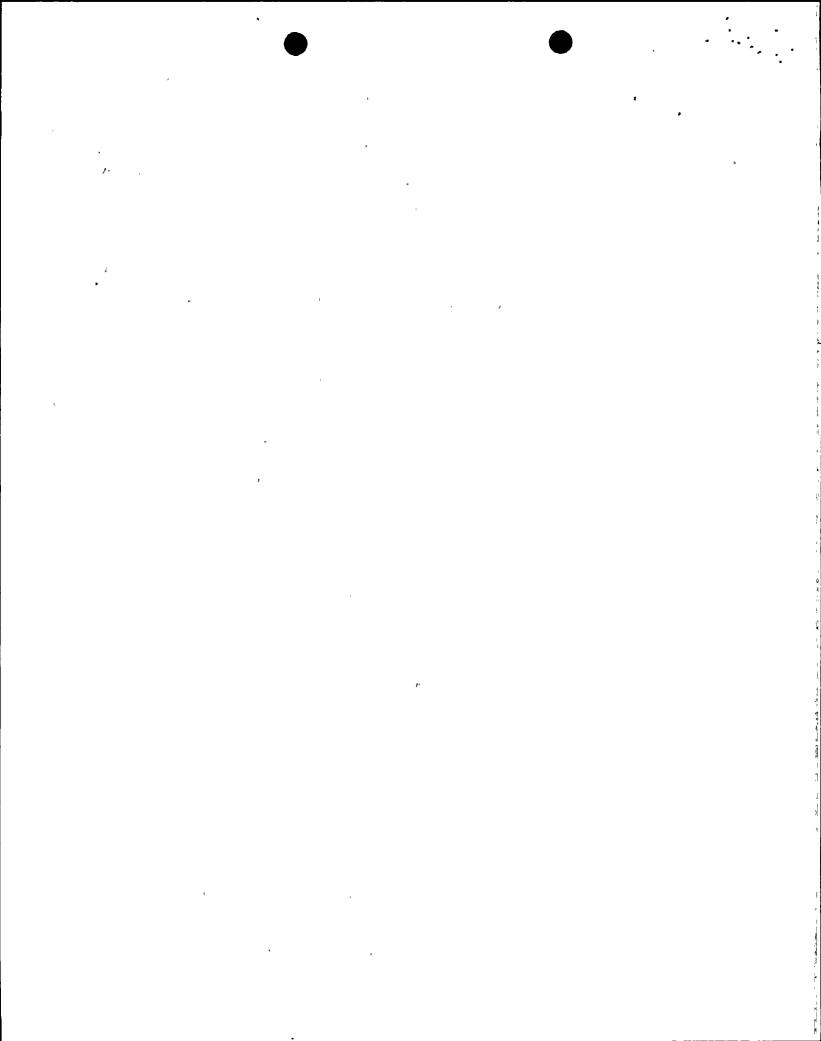
rroject Directorate V Division of Reacton Decises

Division of Reactor Projects III, IV, V and Special Projects

Office of Nuclear Reactor Regulation

Enclosure: Changes to the Technical Specifications

Date of Issuance: July 2, 1990



ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 51 TO FACILITY OPERATING LICENSE NO. NPF-41

DOCKET NO. STN 50-528

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages	<u>InsertPages</u>
6-7	6-7
6-8	6-8
6-9	6-9

6.3 UNIT STAFF QUALIFICATIONS (Continued)

Senior Reactor Operator (SRO) license requirement for the Operations Manager shall be met if either the Operations Manager or the Operations Supervisor holds a valid SRO license. The holder of the SRO license shall direct the licensed activities of the licensed operators.

6.4 TRAINING

6.4.1 A training program for the unit staff shall be maintained under the direction of the Director, Site Services or his designee and shall meet or exceed the requirements and recommendations of Section 5.0 of ANS 3.1-1978 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1 The Plant Review Board shall function to advise the Plant Director on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PRB shall be composed of nine members from the Palo Verde management staff. These positions will be designated by the Plant Director in Administrative Procedures.

The Plant Director shall designate the Chairman and Vice-Chairmen in writing. The Chairman and Vice-Chairmen may be from outside the members provided that they meet ANSI Standard 3.1, 1978.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman, Vice-Chairmen, or his designated alternate.

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QUORUM

6.5.1.5 The quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman, Vice-Chairmen, or his designated alternate and five members including alternates.

RESPONSIBILITIES

- 6.5.1.6 The PRB shall be responsible for:
 - a. Review of all proposed changes to Appendix "A" Technical Specifications.
 - b. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Nuclear Safety Group (NSG).
 - c. Review of REPORTABLE EVENTS.
 - d. Review of unit operations to detect potential nuclear safety hazards.
 - e. Performance of special reviews, investigations or analyses and reports thereon as requested by the Vice President-Nuclear Production or PRB Chairman.
 - f. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last PRB meeting.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6b. above constitutes an unreviewed safety question.
- b. Provide written notification within 24 hours to the Vice President-Nuclear Production, Plant Director and NSG of disagreement between the PRB and the Plant Director, however, the Plant Director shall have responsibility for resolution of such disagreements.

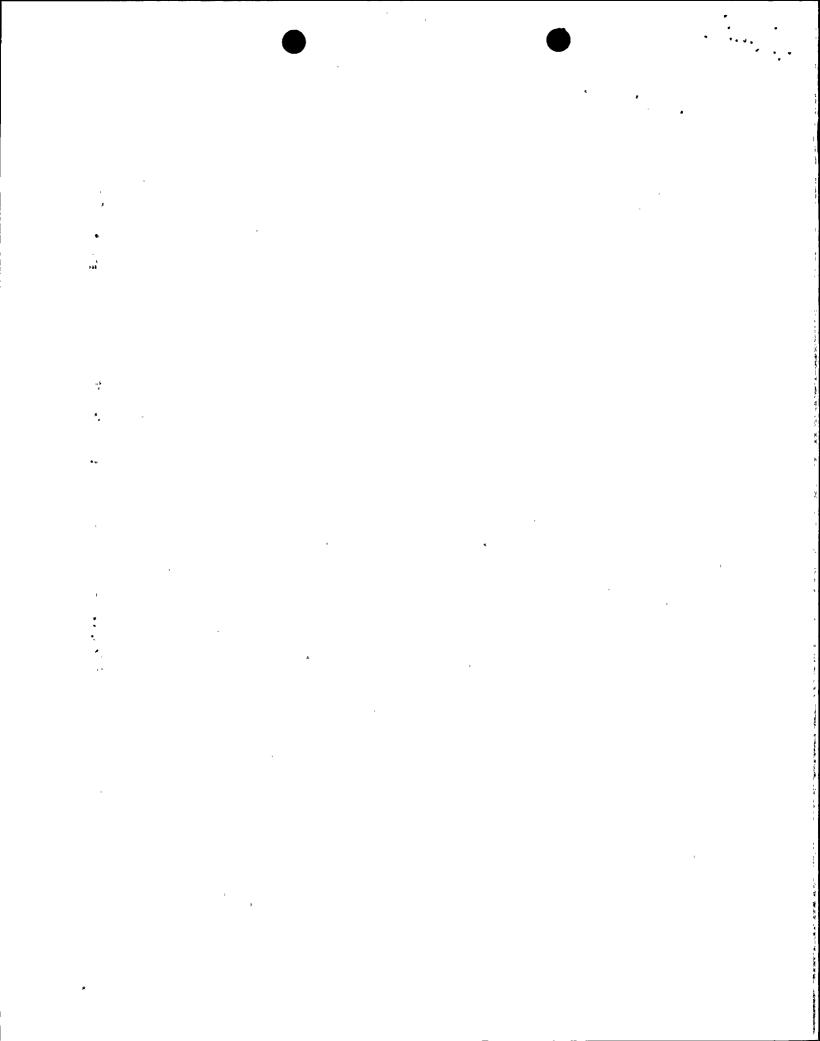
RECORDS

6.5.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Executive Vice President-Nuclear, Vice President-Nuclear Production, Plant Director and NSG.

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6.5.2 TECHNICAL REVIEW AND CONTROL ACTIVITIES

- 6.5.2.1 The Director, Standards and Technical Support shall assure that each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, is prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto.
- 6.5.2.2 Phase I IV tests described in the FSAR that are performed by the plant operations staff shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager as previously designated by the Vice President-Nuclear Production. Test results shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager.
- 6.5.2.3 Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems and components shall be approved prior to implementation by the Plant Director; or by the Plant Manager; or by the Director, Standards and Technical Support as previously designated by the Vice President-Nuclear Production.
- 6.5.2.4 Individuals responsible for reviews performed in accordance with 6.5.2.1, 6.5.2.2, and 6.5.2.3 shall be members of the supervisory staff, previously designated by the Director Standards and Technical Support to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel.
- 6.5.2.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Plant Director.
- 6.5.2.6 The station security program and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.
- 6.5.2.7 The station emergency plan and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.
- 6.5.2.8 The Director, Standards and Technical Support shall assure the performance of a review by a qualified individual/organization of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering the evaluation, recommendations and disposition of the corrective action to prevent recurrence.





UNITED STATES NUCLEAR REGULATORY COMMISSION, WASHINGTON, D. C. 20555

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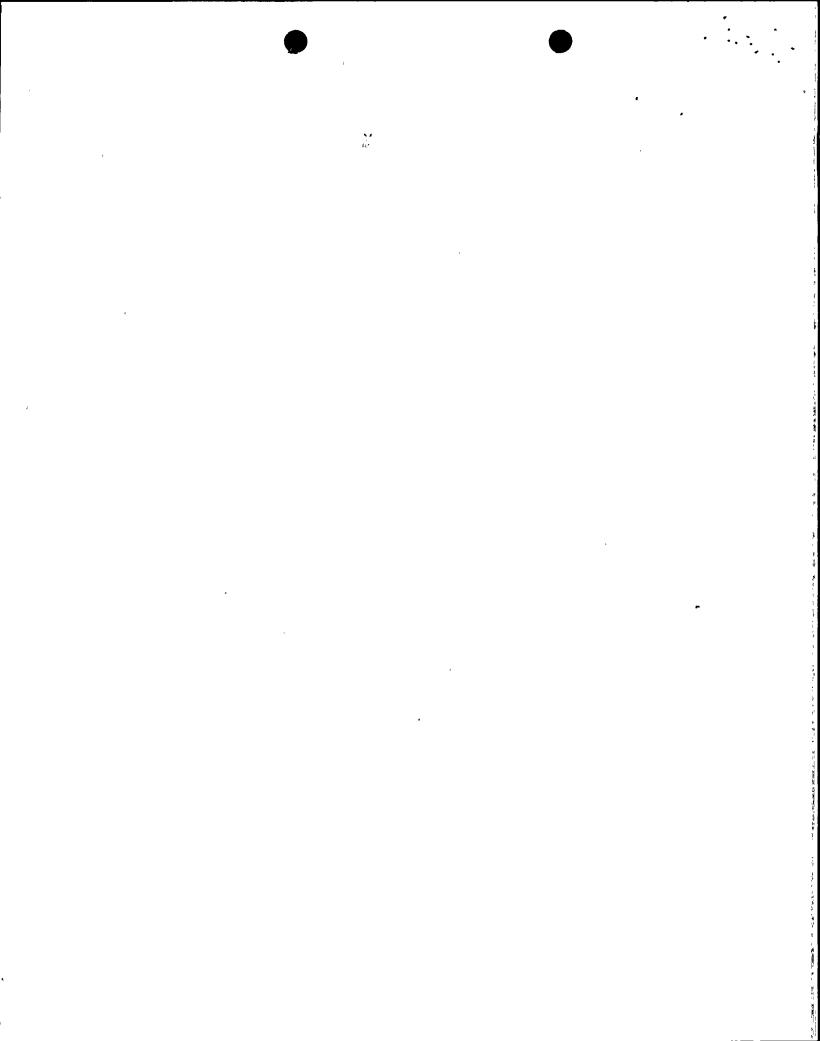
DOCKET NO. STN 50-529

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37 License No. NPF-51

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment dated July 26, 1989 as superseded March 16, 1990 by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Part I;
 - B. The facility will operate in conformity with the application, the provisions of Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations:
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-51 is hereby amended to read as follows:



(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No.37, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective within 45 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John T. Larkins, Acting Director Project Directorate V

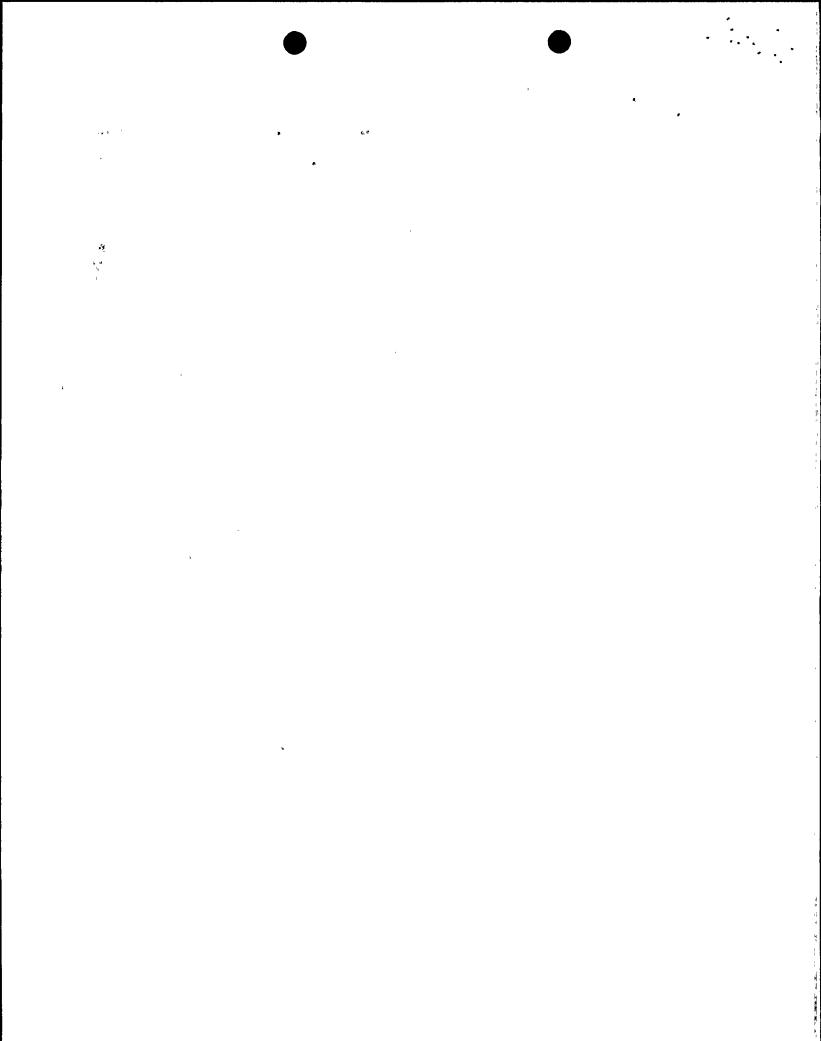
Division of Reactor Projects III,

IV, V and Special Projects

Office of Nuclear Reactor Regulation

Enclosure: Changes to the Technical Specifications

Date of Issuance: July 2, 1990



ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. NPF-51

DOCKET NO. STN 50-529

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages	<u>Insert Pages</u>
6-7	6-7
6-8	6-8
6-9	6-9

6.3 UNIT STAFF QUALIFICATIONS (Continued)

Senior Reactor Operator (SRO) license requirement for the Operations Manager shall be met if either the Operations Manager or the Operations Supervisor holds a valid SRO license. The holder of the SRO license shall direct the licensed activities of the licensed operators.

6.4 TRAINING

6.4.1 A training program for the unit staff shall be maintained under the direction of the Director, Site Services or his designee and shall meet or exceed the requirements and recommendations of Section 5.0 of ANS 3.1-1978 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1 The Plant Review Board shall function to advise the Plant Director on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PRB shall be composed of nine members from the Palo Verde management staff. These positions will be designated by the Plant Director in Administrative Procedures.

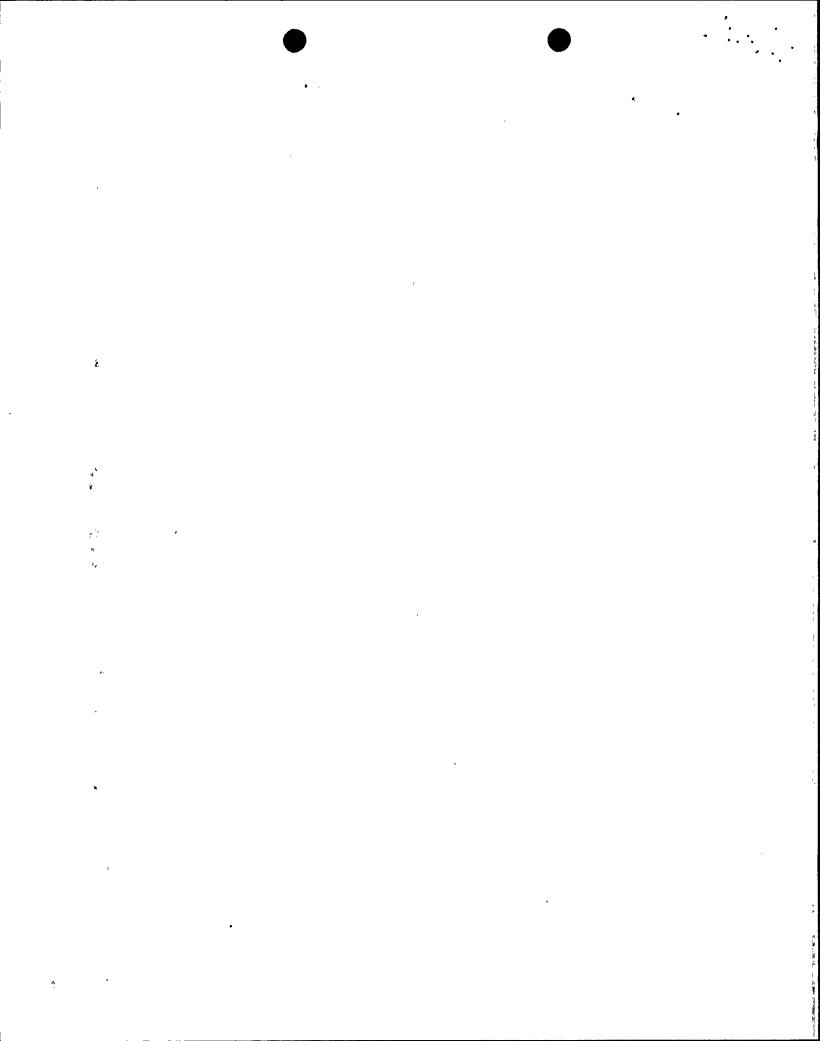
The Plant Director shall designate the Chairman and Vice-Chairmen in writing. The Chairman and Vice-Chairmen may be from outside the members provided that they meet ANSI Standard 3.1, 1978.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman, Vice-Chairmen, or his designated alternate.



QUORUM

6.5.1.5 The quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman, Vice-Chairmen, or his designated alternate and five members including alternates.

RESPONSIBILITIES

- 6.5.1.6 The PRB shall be responsible for:
 - a. Review of all proposed changes to Appendix "A" Technical Specifications.
 - b. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Nuclear Safety Group (NSG).
 - Review of REPORTABLE EVENTS.
 - d. Review of unit operations to detect potential nuclear safety hazards.
 - e. Performance of special reviews, investigations or analyses and reports thereon as requested by the Vice President-Nuclear Production or PRB Chairman.
 - f. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last PRB meeting.

AUTHORITY

6.5.1.7 The PRB shall:

- a. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6b. above constitutes an unreviewed safety question.
- b. Provide written notification within 24 hours to the Vice President-Nuclear Production, Plant Director and NSG of disagreement between the PRB and the Plant Director; however, the Plant Director shall have responsibility for resolution of such disagreements.

RECORDS

6.5.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Executive Vice President-Nuclear, Vice President-Nuclear Production, Plant Director, and NSG.

6.5.2 TECHNICAL REVIEW AND CONTROL ACTIVITIES

- 6.5.2.1 The Director, Standards and Technical Support shall assure that each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, is prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto.
- 6.5.2.2 Phase I IV tests described in the FSAR that are performed by the plant operations staff shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager as previously designated by the Vice President-Nuclear Production. Test results shall be approved by the Director, Standards and Technical Support or the Engineering Evaluations Manager.
- 6.5.2.3 Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems and components shall be approved prior to implementation by the Plant Director; or by the Plant Manager; or by the Director, Standards and Technical Support as previously designated by the Vice President-Nuclear Production.
- 6.5.2.4 Individuals responsible for reviews performed in accordance with 6.5.2.1, 6.5.2.2, and 6.5.2.3 shall be members of the supervisory staff, previously designated by the Director Standards and Technical Support to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel.
- 6.5.2.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Plant Director.
- 6.5.2.6 The station security program and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.
- 6.5.2.7 The station emergency plan and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.
- 6.5.2.8 The Director, Standards and Technical Support shall assure the performance of a review by a qualified individual/organization of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering the evaluation, recommendations and disposition of the corrective action to prevent recurrence.



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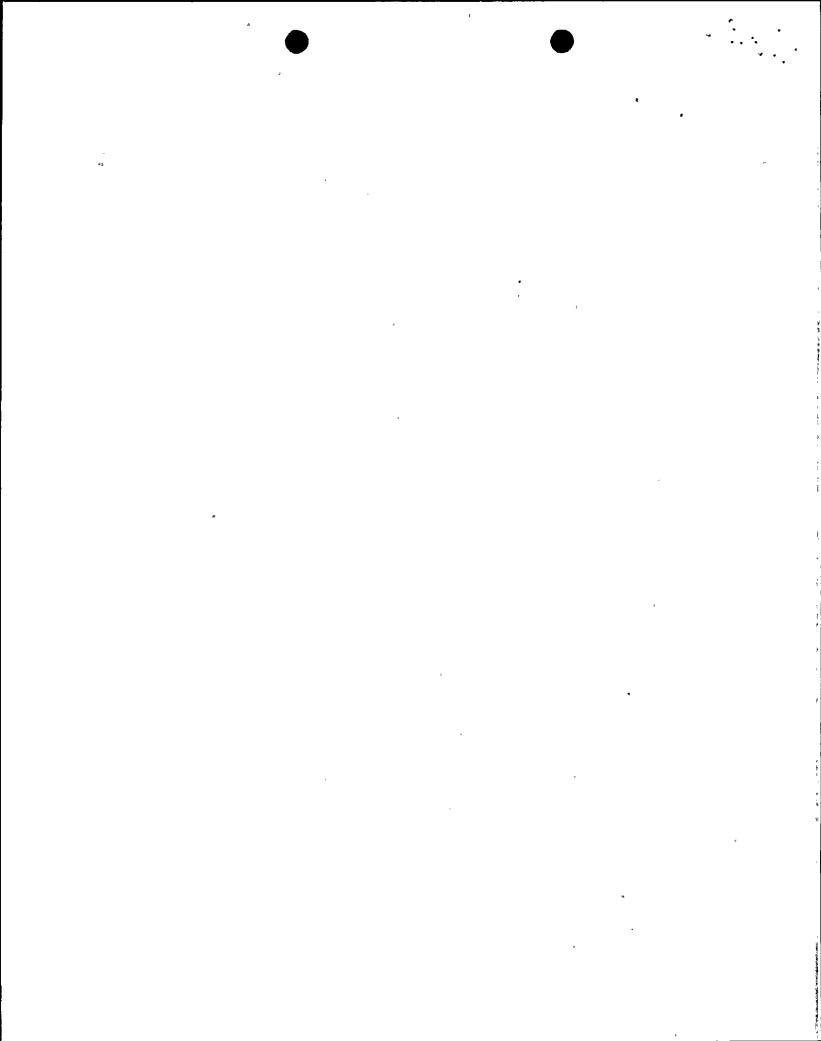
DOCKET NO. STN 50-530

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.23 License No. NPF-74

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment, dated July 26, 1989 as superseded March 16, 1990 by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-74 is hereby amended to read as follows:



(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix Å, as revised through Amendment No. 23, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective within 45 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles M. Traimell
John T. Larkins, Acting Director
Project Directorate V

Division of Reactor Projects III, IV, V and Special Projects

Office of Nuclear Reactor Regulation

Enclosure: Changes to the Technical Specifications

Date of Issuance: July 2, 1990

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ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 23 TO FACILITY OPERATING LICENSE NO. NPF-74

DOCKET NO. STN 50-530

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages	<u>Insert Pages</u>
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6-8	6-8
6-9	6-9

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6.3 UNIT STAFF QUALIFICATIONS (Continued)

Senior Reactor Operator (SRO) license requirement for the Operations Manager shall be met if either the Operations Manager or the Operations Supervisor holds a valid SRO license. The holder of the SRO license shall direct the licensed activities of the licensed operators.

6.4 TRAINING

6.4.1 A training program for the unit staff shall be maintained under the direction of the Director, Site Services or his designee and shall meet or exceed the requirements and recommendations of Section 5.0 of ANS 3.1-1978 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW BOARD (PRB)

FUNCTION

6.5.1.1 The Plant Review Board shall function to advise the Plant Director on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PRB shall be composed of nine members from the Palo Verde management staff. These positions will be designated by the Plant Director in Administrative Procedures.

The Plant Director shall designate the Chairman and Vice-Chairmen in writing. The Chairman and Vice-Chairmen may be from outside the members provided that they meet ANSI Standard 3.1, 1978.

<u>ALTERNATES</u>

6.5.1.3 All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman, Vice-Chairmen, or his designated alternate.

QUORUM

6.5.1.5 The quorum of the PRB necessary for the performance of the PRB responsibility and authority provisions of these Technical Specifications shall consist of the Chairman, Vice-Chairmen, or his designated alternate and five members including alternates.

RESPONSIBILITIES

- 6.5.1.6 The PRB shall be responsible for:
 - a. Review of all proposed changes to Appendix "A" Technical Specifications.
 - b. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Nuclear Safety Group (NSG).
 - c. Review of REPORTABLE EVENTS.
 - d. Review of unit operations to detect potential nuclear safety hazards.
 - e. Performance of special reviews, investigations or analyses and reports thereon as requested by the Vice President-Nuclear Production or PRB chairman.
 - f. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last PRB meeting.

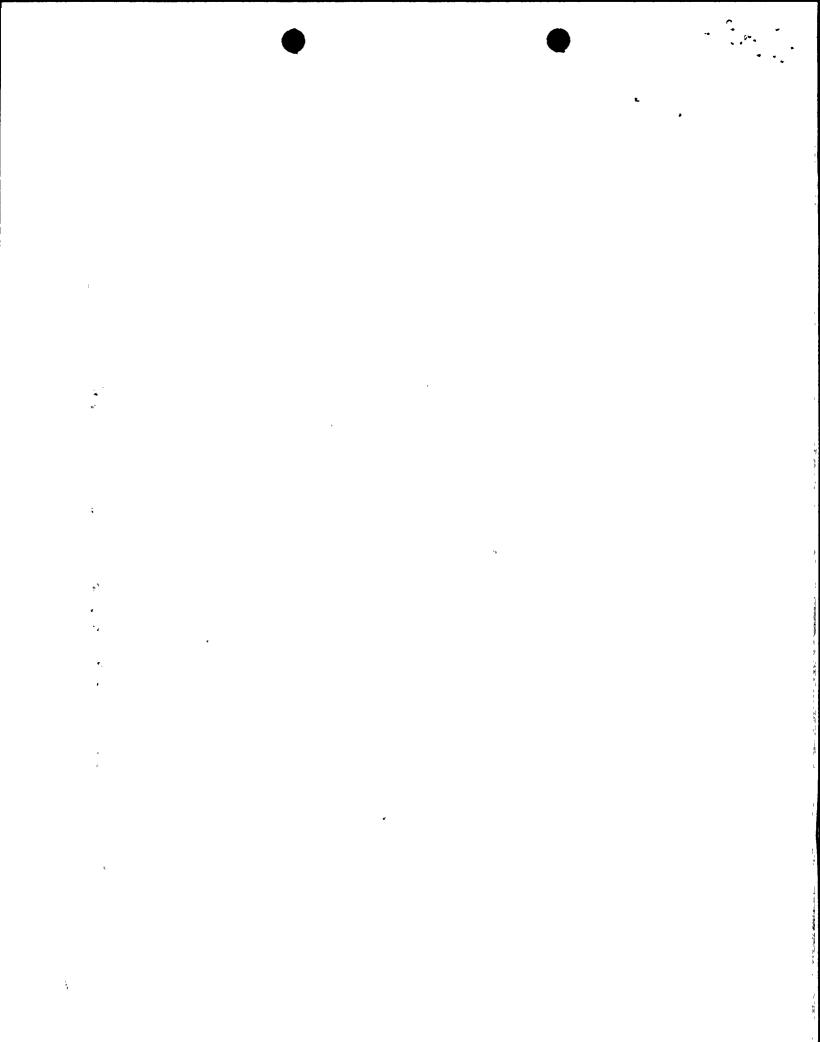
<u>AUTHORITY</u>

6.5.1.7 The PRB shall:

- a. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6b. above constitutes an unreviewed safety question.
- b. Provide written notification within 24 hours to the Vice President-Nuclear Production, Plant Director and NSG of disagreement between the PRB and the Plant Director; however, the Plant Director shall have responsibility for resolution of such disagreements.

RECORDS

6.5.1.8 The PRB shall maintain written minutes of each PRB meeting that, at a minimum, document the results of all PRB activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Executive Vice President-Nuclear, Vice President-Nuclear Production, Plant Director and NSG.



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- 6.5.2.3 Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems and components shall be approved prior to implementation by the Plant Director; or by the Plant Manager; or by the Director, Standards and Technical Support as previously designated by the Vice President-Nuclear Production.
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- 6.5.2.6 The station security program and implementing procedures shall be reviewed. Recommended changes shall be approved by the Director, Site Services or designated alternate and transmitted to the Vice President-Nuclear Production and to the NSG.
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