



10 CFR 50.4(b)(6)
10 CFR 50.71(e)
10 CFR 50.59(d)(2)

RS-17-135

October 24, 2017

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Clinton Power Station, Unit 1
Facility Operating License No. NPF-62
NRC Docket No. 50-461

Subject: Clinton Updated Safety Analysis Report (USAR), Revision 19

In accordance with the requirements of 10 CFR 50.71, "Maintenance of records, making of reports," paragraph (e)(4), Exelon Generation Company, (EGC), LLC submits Revision 19 to the Updated Safety Analysis Report (USAR) for Clinton Power Station.

The USAR is being submitted on Optical Storage Media (OSM) in its entirety, including documents incorporated by reference (e.g., Operational Requirements Manual (ORM)). USAR pages changed as a result of this update are delineated with "Rev. 19, October 2017" in the page footer.

Changes to the USAR and ORM have been made under the provisions of 10 CFR 50.59, "Changes, tests, and experiments." EGC has evaluated these changes in accordance with 10 CFR 50.59 and concluded that the changes do not require prior NRC approval.

Attachment A provides a brief summary of the changes incorporated into USAR Revision 19.

Attachment B provides the required summary report pursuant to 10 CFR 50.59(d)(2).

Attachment C contains a summary of regulatory commitment changes.

Attachment D summarizes the changes to the ORM.

Attachment E provides a required summary report pursuant to 10 CFR 72.48

Attachment F contains the directory path, filename, and size of each individual file.

AD53
NRR

One OSM is included in this submission. The OSM labeled, "Exelon Generation, Clinton Power Station, USAR Revision 19, October 2017," contains the following components:

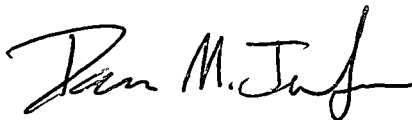
- 001 CPS USAR Rev. 19.pdf, 675 megabytes (MB)
- 002 CPS ORM.pdf, 564 kilobytes (KB)

As required by 10 CFR 50.71(e)(2)(i), I certify that to the best of my knowledge, the information contained in the enclosures and attachments to this letter accurately reflect information and analyses submitted to the NRC or prepared pursuant to NRC requirements, and changes made under the provisions of 10 CFR 50.59.

There are no new commitments made in this document. Should you have any questions concerning this letter, please contact:

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Respectfully,



Dominic M. Imburgia
Manager – Licensing Programs

Enclosure: OSM – Exelon Generation, Clinton Power Station, USAR Revision 19,
October 2017

Attachments:

- Attachment A, "CPS USAR Revision 19 Change Summary Report"
- Attachment B, "10 CFR 50.59 Summary Report"
- Attachment C, "Summary of Regulatory Commitment Changes"
- Attachment D, "Summary of Operational Requirements Manual Changes"
- Attachment E, "10 CFR 72.48 Summary Report"
- Attachment F, "OSM Directory Structure"

cc: Regional Administrator – NRC Region III
NRC Senior Resident Inspector – Clinton Power Station

Attachment A
CPS USAR Revision 19 Change Summary Report
Page 1 of 3

USAR Change 2016-018, Removal of Feedwater Performance Requirements. Remove pre-EPU numbers for Feedwater capability. IR 2656128. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2016-020, MSIV conversion to Chesterton packing without a leak-off line. USAR Chapter 5.4.5.4 is revised to state that all Main Steam Isolation Valves (MSIV) have been modified from a two set packing with a lantern ring leak-off line to a live loaded Chesterton packing without a leak-off line. IR 2718718. Modification MSF012, MSIV modification to remove packing leak off lines and the double set of packing arrangement replaced with a single set.

USAR Change 2016-022, Dry Solid Waste Packaging Equipment. USAR Section 11.4.2.5 is revised. Deleted out-dated description of Dry Active Waste (DAW) sorting methods. IR 2718148. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2016-023, Added Reference to Quality Assurance Topical Report NO-AA-10. USAR Section 13.1.2.1 is revised. The QATR added as a source document describing functions, responsibilities, and authorities of various staff positions. IR 2680308. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2016-024, Control Rod Surveillance Periodicity. USAR Section 4.6.3.1.1.5(5) is revised. The change reflects surveillance frequency change to control rod drive exercising from every 31 days to in accordance with the Surveillance Frequency Control Program as authorized by License Amendments 188 and 192. A 50.59 Applicability Review concluded the USAR change was a conforming change in accordance with the referenced License Amendments.

USAR Change 2016-025, Main Steam Bypass Valve Surveillance Periodicity. USAR Section 10.4.4.4 is revised. The change reflects surveillance frequency change to Main Steam Bypass Valve exercising from every 31 days to in accordance with the Surveillance Frequency Control Program. License Amendment 192. 50.59 Screening CL-2016-S-043 concluded the change does not require prior NRC approval.

USAR Change 2016-026, Reactor Water Clean-Up System Bypass. USAR Section 7.3.1.1.2.4 is revised. Removes discussion on RT system high differential flow trip bypass use to prevent inadvertent system isolations. IR 2685337. 50.59 Screening CL-2016-S-041 concluded the change does not require prior NRC approval.

USAR Change 2016-028, Modify Sparger Lines on Walls of Spent Fuel Pool. USAR Section 9.1.3.2 is updated. Remove interference with new SFP storage racks. EC 362811 and 50.59 CL-2006-S-051 concluded the change does not require prior NRC approval.

USAR Change 2016-029, Cask Processing. USAR Section 9.4.1 is revised. When a cask is filled with spent fuel, all pool gates are closed. AR 02742226. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2016-030, Comply with Criticality Accident Requirements of 10CFR50.68(b). USAR Section 9.1 is revised. This does include an exemption to the criticality alarm requirements of

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10CFR70.24. IR 02741851 and 50.59 CL-2016-S-042 concluded the change does not require prior NRC approval.

USAR Change 2016-031, Chapter 3 Editorial Changes. USAR Sections 3.1.2 and 3.2.3.4.3 are revised. IR 02724831. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2016-032, Revised Wording for Consistency with GE NEDC-32945P. USAR Section 6.3.3 and Figure 6.2-5 are revised. Editorial in Section 6.3.3.7.8.1 and Section title change for consistency with NEDC-32945P. IR 02724564. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2016-033, Revised Frequency of Plant Access Training and Radworker Training. USAR Sections 1.8 and 12.5.2 are revised to take exception of Regulatory Guide 8.27. Periodicity extended from annually to every two years. IR 02729441 and 50.59 Screening CL-2016-S-045 concluded the change does not require prior NRC approval.

USAR Change 2017-001, IDNR and Local Law Enforcement Changes. USAR Sections 2.1.1.2 and 2.1.2.3 are revised. Deleted reference to Illinois Department of Natural Resources (IDNR) occupying the former CPS Energy and Environmental Center Building. Clarification that the Local Sheriff may be notified by Clinton Power Station other than just the Emergency Response Organization because Site Security may also contact the local law enforcement agency. IR 03947646. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2017-002, Chapter 3 Editorial and Typographical Changes. USAR Sections 3.9.3, 3.10.1, and 7.3.2. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2017-003, Chapter 3.11 Editorial and Typographical Changes. USAR Section 3.11 is revised. IR 02680308. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2017-004, Remove Reference to RCIC as an ESF System. USAR Sections 5.4.6.2 and 15.2.9, Tables 6.2-47, 7.1-1 and 7.3-1 are revised. Removes reference to Reactor Core Isolation Cooling (RCIC) as an Engineered Safety Feature (ESF). IR 03977720 and 50.59 Screening CL-2007-S-026.

USAR Change 2017-006, Chapters 2.5 and 3.7 Editorial and Typographical Changes. USAR Sections 2.5 and 3.7 are revised. IR 3950267. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2017-007, Periodicity of VC Smoke Mode. USAR Chapter 9.4.1.4 is revised. Periodicity of Main Control Room Ventilation Smoke Mode from 18 to 24 months. IR 03989866. NRC Safety Evaluation related to Technical Specification Amendment 169, which approved the extension of Clinton Power Station Unit 1 operating cycle from 18 to 24 months.

USAR Change 2017-008, Cycle 18 Reload. USAR Chapter 6.3 and Chapter 15D.4 are revised. Changes made to support Clinton Unit 1 Reload 17. Changes include the removal of GE14 fuel

Attachment A
CPS USAR Revision 19 Change Summary Report
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from the core. EC 617418. 50.59 CL-2017-S-012 concluded the change does not require prior NRC approval.

USAR Change 2017-009, Clarify RHR Suppression Pool Cooling Function. USAR Sections 5.4.7 and 5.4.16. Clarification of statement that implies the suppression pool temperature limit is 170F. This statement comes from GE Design Specification 22A3139. The suppression pool design temperature is 185F post-accident. IR 04008240. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2017-010, FLEX Fire Suppression Error. USAR Appendix E Section 4.0, Item F.9 is revised to remove the statement concerning Main Control Room annunciation of activation of the fire protection flow switch. IR 03988361. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2017-011, Containment Fuel Storage Rack K-Infinity Value. USAR Chapter 9.1 is revised. Change made to reflect vendor analysis revision for low density fuel storage rack bundle reactivity criterion. IR 2702052. 50.59 CL-2017-S-011 concluded the change does not require prior NRC approval.

USAR Change 2017-012, Chapter 3 Figures Update. USAR Section 3.6.1 Figures 3.6-1 Sheet 62 and Sheet 63 are revised to reflect removal of piping related to expired EC 330563. IR 4026209. Non-regulatory change in accordance with NEI 98-03 Guidance.

USAR Change 2017-014, Update to Commercial Airlines. USAR Section 3.5.1.6 is revised to reflect updates to commercial airline carriers at two nearby airports. Issue Report 04019719. Non-regulatory change in accordance with NEI 98-03 Guidance.

Attachment B
10 CFR 50.59 Summary Report
Page 1 of 1

This attachment contains 50.59 evaluation summaries performed for Clinton Power Station (CPS) during this reporting period.

Activity Number / Title

CL-2014-E-033 Revision 1: Engineering Change (EC) 395976, ISFSI Thermal Impacts on Fuel Building and Equipment from Spent Fuel Loading Campaign. USAR Change 2015-010 included the evaluation of the short term pressure response (drawdown) of the secondary containment including the effects of the spent fuel cask heat load which was ultimately approved via License Amendment 210. There were no revisions to this portion of the EC.

Description of Activity

This EC (395976) is developed in support of implementing the Independent Spent Fuel Storage Installation (ISFSI) at Clinton Power Station (CPS). Revision 1 of this EC (395976) includes the following:

- Revision 1 of this EC removes the extension of the secondary containment boundary from the scope of this EC (395976) and addresses evaluation of the short term pressure response (drawdown) of the secondary containment including the effects of the spent fuel cask heat load. Inclusion of the cask heat load results in increased secondary containment drawdown time and increased dose and consequently required a License Amendment Request (LAR). The LAR has been submitted to the NRC (Letter RS-16-019). This 50.59 evaluation addresses aspects of EC 395976 not covered by the LAR.

Reason for Activity

Exelon Corporation LLC has made the decision to not pursue extension of the Secondary Containment boundary into the Fuel Building Railroad Bay. This EC assesses and addresses the impacts of the design basis heat load from a Holtec spent fuel cask on the pressure and temperature profile of the secondary containment which was included in USAR Change 2015-010.

Bases for Not Requiring Prior NRC Approval

The change removed extending the Secondary Containment into the Fuel Building Railroad Bay and the evaluation concluded that the change did not require prior NRC approval.

Attachment C
Summary of Regulatory Commitment Changes
Page 1 of 2

Existing Commitment Description

Clinton Power Station, Unit 1 will update the site SAMGs to Revision 3 of the BWROG generic severe accident guidelines (published February, 2013), by June 30, 2017.

Reference:

1. Letter RS-16-214

Revised Commitment Description

Exelon Generation Company, LLC (EGC) is revising the implementation date as documented in Enclosure 5 of Reference 1, to December 31, 2017.

Justification

This will allow sufficient time to complete the development, training, and implementation of the revised SAMGs should CPS continue power operations beyond June 1, 2017. The revised commitment maintains the intent of the original commitment.

Attachment C
Summary of Regulatory Commitment Changes
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Existing Commitment Description

EGC will conduct a one-time briefing on HU-AA-102, Technical Human Performance Practices to the design engineering population with focus on behaviors to maintain proper management involvement, questioning attitude, and challenge, by September 29, 2017.

Reference:

1. U-604368, dated September 7, 2017, Clinton Power Station Reply to a Notice of Violation; IR 05000461/2017002-02.

Revised Commitment Description

The commitment description remains the same, the date has changed from September 29, 2017 to October 19, 2017.

One-Time Action completes the intent of the original commitment.

Justification

Completes corrective step from cause for violation 05000461/2017002-02.

Attachment D
Summary of Operational Requirements Manual Changes
Page 1 of 1

No revisions to the Clinton Power Station (CPS) Operational Requirements Manual (ORM) have been made during this reporting period.

The current revision of the CPS ORM is Revision 80, as reported in Letter RS-16-196 dated October 24, 2016.

| <u>Revision Number</u> | <u>Scope of Revision</u> |
|--|--------------------------|
| No revisions during this reporting period. | N/A |

Attachment E
10 CFR 72.48 Summary Report
Page 1 of 4

This attachment contains 72.48 evaluation summaries performed for Clinton Power Station (CPS) during this reporting period.

Activity Number / Title

72.48-CL-2016-E-002: Engineering Change Order (ECO) 5018-47 Revision 1, Thermal Evaluation of HI-TRAC VW in Decontamination Pit at Clinton Power Station.

Description of Activity

The HI-TRAC was originally evaluated assuming that it would be exposed to elevated temperature once while spent fuel was being processed. In actuality the MPC sees extreme temperature repeatedly during fuel processing.

Reason for Activity

The ECO changes the temperature limits in the FSAR for various parts of the HI-TRAC VW. The original temperature limits considered for the HI-TRACVW and MPC were those identified for off-normal and accident conditions, because the expectation was that short-term operation conditions (transfer of spent fuel from pool to HI-STORM) would only occur once during the component's service life. This logic has been corrected for the HI-TRAC VW, which will be used multiple times; therefore, the short term event temperature limits in Table 2.2.3 are applicable.

Bases for Not Requiring Prior NRC Approval

The changes made by ECO 5018-47, Rev. 1, do not adversely affect structural integrity of the components, shielding properties of the transfer cask or MPC components, or criticality of the spent fuel. All the predicted component temperature limits from the thermal analysis for short-term events reported in Section 4.5 of the FSAR are below the design temperature limits in FSAR Table 2.2.3.

The proposed changes to increase the short-term temperature limits of HI-TRAC VW cask components have an adverse effect on the structural evaluations. However, these proposed changes are acceptable for the following reason: The reduction in mechanical properties of these components at the proposed higher temperatures does not affect the results significantly, as all the affected safety factors for these structural components remain above 2.0. The calculated stress values and the corresponding safety factors for these components are updated in FSAR Tables 3.4.2 and 3.4.9. Therefore, all the components satisfy the structural requirements at the proposed higher temperatures. In fact, the higher temperature limits in ECO 5018-47, Rev. 1, provide larger temperature margins during short-term operations

The changes made by ECO 5018-47, Rev. 1 have been evaluated against 10 CFR 72.48 criterion in Holtec Screening/Evaluation 72-48-1150 Rev. 1 and Holtec has determined that the proposed Activity does not require NRC approval via CoC amendment prior to implementation.

Attachment E
10 CFR 72.48 Summary Report
Page 2 of 4

Activity Number / Title

72.48-CL-2016-E-004: ECO 5018-50, ISFSI Pad Concrete Compressive Strength.

Description of Activity

Clinton will use the Holtec HI-STORM FW dry cask storage system (hereafter referred to as HI-STORM FW) and is in the process of preparing the 72.212 report for Clinton's initial campaign, which adopts revision 2 of the HISTORM FW FSAR (hereafter referred to as FSAR). FSAR, Rev. 2, Table 2.2.9 specifies the maximum thickness, subgrade modulus of elasticity, and compressive strength of the ISFSI pad concrete to establish the upper bound ISFSI pad stiffness for use in the tip-over analysis (per FSAR reference the Non-Mechanistic Tip-over Event of the Loaded HI-STORM FW Storage Cask is evaluated in HI-2094353).

Holtec, the vendor supplying Clinton's Dry Cask Storage (DCS) system, has revised FSAR Table 2.2.9, the concrete pad upper limit compressive strength, by Engineering Change Order (ECO) 5018-50, Revision 0. This ECO revises the concrete pad upper limit compressive strength from 6,000 psi to 7,000 psi, and revises the discussion of non-mechanistic tip-over in section 3.4.4.1.4 to include results for a concrete pad compressive strength of 7,000 psi.

Based on the results of the tip-over analysis, Figures 3.4.16, 3.4.17, 3.4.18, 3.4.19, and 3.4.20 are revised.

This 72.48 review is being performed to document the impact of the changes by ECO 5018-50 to the licensing bases for Clinton's DCS system.

Reason for Activity

Test results of the concrete placed on the ISFSI storage pad on September 10, 2015 indicated average 28 day results as high as 6,130 psi, exceeding the Clinton Power Station Construction Specifications (HI-2135703) limit of 5,500 psi and the FSAR Table 2.2.9 limit of 6,000 psi (Reference: IR 2555015 & Holtec FCR 2226-052).

ECO 5018-050, increases the FSAR (Table 2.2.9) upper limit for concrete compressive strength from 6,000 psi to 7,000 psi. Based on this change, the CPS construction specification upper limit was increased from 5,500 psi to 6,500 psi, providing basis for acceptance of the concrete..

Bases for Not Requiring Prior NRC Approval

The changes made by ECO 5018-50, Rev. 0, meet the acceptance criteria defined in Section 2.2.3b of the FSAR for Non-Mechanistic Tip-Over. All safety conclusions remain unchanged.

The changes made by ECO 5018-50, Rev. 0 have been evaluated against 10 CFR 72.48 criterion in Holtec Screening/Evaluation 72-48-1148 Rev. 0 and it has been determined that the proposed Activity does not require NRC approval via CoC amendment prior to implementation.

Attachment E
10 CFR 72.48 Summary Report
Page 3 of 4

Activity Number / Title

72.48-CL-2016-E-005: ECO 5018-45, HI-STORM FW Overpack Concrete Temperature Limits.

Description of Activity

Clinton will use the Holtec HI-STORM FW dry cask storage system (hereafter referred to as HI-STORM FW) and is in the process of preparing the 72.212 report for Clinton's initial campaign, which adopts revision 2 of the HISTORM FW FSAR (hereafter referred to as FSAR). The HI-STORM FW overpack concrete design temperature limits are identified in FSAR Table 2.2.3. The temperature limits have been increased for a fire event and changed from a section average limit to a local limit by Engineering Change Order (ECO) 5018-45, Revision 0, which is to be incorporated into revision 4 of the FSAR. The details of the change are that the short term limit for concrete temperature within the overpack from a 350°F section average to a 650°F local limit for off-normal conditions and a local temperature limit of 1100°F is adopted for concrete during a fire accident.

Reason for Activity

The increased overpack concrete temperature limits for off-normal and accident conditions can be adopted by CPS because the technical requirements from FSAR revision 2 are the same as those for FSAR revision 4, and the technical justification for the change in ECO 5018-045 ensure structural integrity of the HI-STORM FW components. (Note: The fire hazards analyses for CPS events does not predict that any HI-STORM FW components will be exposed to temperature above the increased temperature limits.)

Bases for Not Requiring Prior NRC Approval

The changes made by ECO 5018-45 to temperature limits applicable to HI-STORM FW components have no adverse effect on operation of the components or compliance with design requirements in FSAR revision 2, which are the same requirements for FSAR, revision 4. Also, the changes in temperatures limits have no impact to the effectiveness of overpack's design functions, which includes shielding for the stored spent fuel.

The changes made by ECO 5018-45, Rev. 0 have been evaluated against 10 CFR 72.48 criterion in Holtec Screening/Evaluation 72-48-1124 Rev. 0 and it has been determined that the proposed Activity does not require NRC approval via CoC amendment prior to implementation.

Attachment E
10 CFR 72.48 Summary Report
Page 4 of 4

Activity Number / Title

72.48-CL-2016-E-010: Engineering Change (EC) 404679, ISFSI 2016 Dry Cask Storage Campaign.

Description of Activity

EC 404679 documents which HI-STORM overpacks and Multi-Purpose Canisters (MPCs) will be loaded and that the required documentation, analyses and 72.48 reviews have been completed (if required) to bound the casks that are proposed to be loaded during the Clinton 2016 Spent Fuel Loading Campaign (SFLC).

EC 404679 also assigns Equipment Identification Numbers (EINs) to the cask components for configuration control. The EIN equipment data information includes pertinent component information related to the loading of spent fuel into the MPC and placement of the HI-STORM on the ISFSI cask storage pad.

Individual changes have been made to the MPCs and HI-STORMs by the cask vendor (Holtec). These changes have been accepted by the cask vendor under their 72.48 review process where appropriate.

Reason for Activity

The proposed change adds the option to cover the carbon steel HI-STORM FW baseplate with stainless steel within the inlet vents. The primary design function of the inlet vents is to permit the flow of cool air into the overpack to ensure adequate heat removal from the MPC. The use of a stainless steel baseplate or coating for the inlet vents has no impact on the method of ensuring adequate heat removal capability. Therefore, the proposed activity does not have any adverse impact on the methods of performing or controlling the design functions of the HI-STORM FW.

Bases for Not Requiring Prior NRC Approval

The MPCs and HI-STORMS to be loaded during the 2016 Clinton SFLC require an Exelon 72.48 Review. An evaluation is required as several of the Engineering Change Orders (ECOs) and Supplier Manufacturing Deviation Reports (SMDRs) are regarded as adverse, however, the adverse impact is minimal and it is determined that NRC prior approval is not required.

Attachment F
OSM Directory Structure
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| Directory Path | File Name | Size |
|-----------------------|---|-------------|
| 001 CPS USAR Rev 19 | 000 USAR Cover and LOEP.pdf | 432 KB |
| 001 CPS USAR Rev 19 | 001 CH 01 Intro and General Desc.pdf | 734 KB |
| 001 CPS USAR Rev 19 | 002 CH 02 Site Characteristics.pdf | 1608 KB |
| 001 CPS USAR Rev 19 | 003 CH 02 Figures Part 1 of 2.pdf | 86922 KB |
| 001 CPS USAR Rev 19 | 004 CH 02 Figures Part 2 of 2.pdf | 33952 KB |
| 001 CPS USAR Rev 19 | 005 CH 03 Design of Struct Comp Equip.pdf | 2649 KB |
| 001 CPS USAR Rev 19 | 006 CH 03 Figures Part 1 of 5.pdf | 41602 KB |
| 001 CPS USAR Rev 19 | 007 CH 03 Figures Part 2 of 5.pdf | 77531 KB |
| 001 CPS USAR Rev 19 | 008 CH 03 Figures Part 3 of 5.pdf | 81571 KB |
| 001 CPS USAR Rev 19 | 009 CH 03 Figures Part 4 of 5.pdf | 69153 KB |
| 001 CPS USAR Rev 19 | 010 CH 03 Figures Part 5 of 5.pdf | 84368 KB |
| 001 CPS USAR Rev 19 | 011 CH 04 Reactor.pdf | 2838 KB |
| 001 CPS USAR Rev 19 | 012 CH 05 RCS and Connect Systems.pdf | 2964 KB |
| 001 CPS USAR Rev 19 | 013 CH 06 Engineered Safety Features.pdf | 20800 KB |
| 001 CPS USAR Rev 19 | 014 CH 07 Instrument and Control Sys.pdf | 6167 KB |
| 001 CPS USAR Rev 19 | 015 CH 08 Electric Power.pdf | 1204 KB |
| 001 CPS USAR Rev 19 | 016 CH 09 Auxiliary Systems.pdf | 5056 KB |
| 001 CPS USAR Rev 19 | 017 CH 10 Steam and Power Conv.pdf | 754 KB |
| 001 CPS USAR Rev 19 | 018 CH 11 Radioactive Waste Mgmt.pdf | 933 KB |
| 001 CPS USAR Rev 19 | 019 CH 12 Radiation Protection.pdf | 2485 KB |
| 001 CPS USAR Rev 19 | 020 CH 13 Conduct of Operations.pdf | 617 KB |
| 001 CPS USAR Rev 19 | 021 CH 14 Initial Test Program.pdf | 955 KB |
| 001 CPS USAR Rev 19 | 022 CH 15 Accident Analysis.pdf | 13988 KB |
| 001 CPS USAR Rev 19 | 023 CH 16 Technical Specifications.pdf | 42 KB |
| 001 CPS USAR Rev 19 | 024 CH 17 Quality Assurance.pdf | 41 KB |
| 001 CPS USAR Rev 19 | 025 APP A Glossary.pdf | 94 KB |
| 001 CPS USAR Rev 19 | 026 APP B Const Matl Stds and QC.pdf | 150 KB |
| 001 CPS USAR Rev 19 | 027 APP C Computer Programs.pdf | 7092 KB |
| 001 CPS USAR Rev 19 | 028 APP D TMI Requirements.pdf | 1950 KB |
| 001 CPS USAR Rev 19 | 029 APP E Fire Protection Report.pdf | 99859 KB |
| 001 CPS USAR Rev 19 | 030 APP F Safe Shutdown.pdf | 43387 KB |
| 002 CPS ORM | 001 CPS ORM.pdf | 564 KB |