Vogtle PEmails

From: Hoellman, Jordan
Sent: Tuesday, October 31, 2017 12:31 PM
To: Vogtle PEmails
Subject: Draft ITP ITAAC Consolidation LAR for 11/9/17 Pre-Submittal Meeting
Attachments: Draft LAR-17-038 For NRC Presubmital on 11-9-17.pdf

Attached is the draft Initial Test Program (ITP) ITAAC Consolidation License Amendment Request (LAR) for discussion at the 11/9 pre-submittal meeting.

Hearing Identifier: Vogtle_COL_Docs_Public
Email Number: 177

Mail Envelope Properties (c4e285a274a74ae28b00ff5c6517918b)

Subject: Draft ITP ITAAC Consolidation LAR for 11/9/17 Pre-Submittal Meeting
Sent Date: 10/31/2017 12:31:12 PM
Received Date: 10/31/2017 12:31:17 PM
From: Hoellman, Jordan

Created By: Jordan.Hoellman2@nrc.gov

Recipients:
"Vogtle PEmails" <Vogtle.PEmails@nrc.gov>
Tracking Status: None

Post Office: HQPWMSMRS03.nrc.gov

Files	**Size**	**Date & Time**
MESSAGE	157	10/31/2017 12:31:17 PM
Draft LAR-17-038 For NRC Presubmital on 11-9-17.pdf		1821314

Options
Priority: Standard
Return Notification: No
Reply Requested: No
Sensitivity: Normal
Expiration Date:
Recipients Received:

Southern Nuclear Operating Company

ND-17-0xxx

Enclosure 1

Vogtle Electric Generating Plant (VEGP) Units 3 and 4

Request for License Amendment:

Testing Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) Consolidation

(LAR-17-038)

(This Enclosure consists of 26 pages, including this cover page.)

Table of Contents

Pursuant to 10 CFR 52.98(c), and in accordance with 10 CFR 50.90, Southern Nuclear Operating Company (SNC) (the “Licensee”) hereby requests an amendment to Combined License (COL) Nos. NPF-91 and NPF-92, for Vogtle Electric Generating Plant (VEGP) Units 3 and 4, respectively.

1. SUMMARY DESCRIPTION

The proposed changes would make non-technical changes to COL Appendix C (and corresponding plant-specific Tier 1) information. The changes consolidate Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC), by combining the associated Design Commitment (DC), Inspections, Tests, Analyses (ITA) and Acceptance Criteria (AC). The consolidation relocates and combines multiple ITAAC into a single ITAAC. The DC, ITA and AC remain unchanged in the combined ITAAC. The proposed changes consolidate ITAAC within the following four categories:

1. “Initial Test Program” (ITP) ITAAC, which demonstrate testing of components within a system along with associated system testing.
2. “Initial Test Program and Related Inspections” ITAAC, which verify the existence and location of components and equipment features prior to demonstrating testing of components within a system along with associated system testing.
3. “Related Inspections and Analyses” ITAAC, which verify similar attributes of related components within a system through inspections, tests outside of ITP scope, analyses, or some combination of the three.
4. "Security Testing" ITAAC, which demonstrate related aspects of the security system through inspections, tests outside of ITP scope, analyses, or some combination of the three.

The requested amendment proposes changes to COL Appendix C information, with corresponding changes to plant-specific DCD Tier 1 information, as appropriate. This enclosure requests approval of the license amendment necessary to implement the COL Appendix C changes described below. Enclosure 2 requests the exemption necessary to implement the changes to the plant-specific DCD Tier 1 information.

The purpose of the proposed changes being requested is to:

- Reduce the forthcoming surge in ITAAC closures without compromising completion of the approved design, resulting in levelization of both Licensee and Regulator resources.

- Reduce the number of ITAAC Closure Notifications (ICNs) and regulatory burden for SNC and the NRC.
- Obtain formal upfront NRC review and approval to consolidate related ITAAC.

Consolidation will not result in removal of any quality activity/design attribute or safety margin.

2. DETAILED DESCRIPTION AND TECHNICAL EVALUATION

Updated Final Safety Analysis Report (UFSAR) Tier 2 design descriptions are derived from plant design documents. 10 CFR Part 52, Appendix D, Section II.D, states that Tier 1 design information is “derived from Tier 2 information.” There are certain ITAAC which have been identified in COL Appendix C (and plant-specific Tier 1) that require completion of tests that may be completed at the same time. Even though many of these tests are completed at the same time, they are in separate ITAAC causing additional administrative work to prepare and submit individual completion packages and ICNs. Therefore, it is proposed to make changes to COL Appendix C (and plant-specific Tier 1) to consolidate two or more ITAAC. For each of the ITAAC proposed for consolidation, the associated UFSAR design information is consistent with the current plant design, so no structure, system, or component (SSC), design function, or analysis, as described in the UFSAR, is affected by the proposed changes. The proposed changes to consolidate these ITAAC do not add, remove or change any text within any of the ITAAC. These proposed changes are only administrative in that they rearrange the location of ITAAC and change some of the numbers.

For each Category described below, multiple ITAAC are proposed for consolidation to allow a single completion package and ICN for each consolidated ITAAC.

Category 1 – “Initial Test Program” ITAAC

Several ITAAC verify that components within a given system demonstrate their safety or non-safety related function by testing their listed function or capability during the component test phase. There are additional ITAAC that demonstrate the given integrated system performs in accordance with design criteria through pre-operational system testing. These ITAAC (hereafter referred to collectively as ITP ITAAC or ITAAC in Category 1) require completed test results in order to close each individual testing ITAAC. Additionally, individual component ITAAC are performed prior to the associated system pre-operational testing ITAAC, and are therefore related ITAAC. In this category of ITAAC being consolidated, one or more tests are used to complete several related ITAAC within the same system and therefore consolidation into a single ITAAC is logical. Within this category of ITP ITAAC, 149 ITAAC are proposed to be consolidated into 30 ITAAC. The following basis was used to evaluate ITP ITAAC which could be consolidated.

- Pre-operational testing ITAAC in the same system are often completed in the same pre-operational test procedure, and therefore the related ITAAC use the same ITAAC closure documentation and can be combined.
- Component testing ITAAC in the same system, where that system has pre-operational testing ITAAC, can be combined with the pre-operational testing ITAAC

because the component tests are pre-requisites for the system pre-operational testing.

- Component testing ITAAC in the same system that do not have related pre-operational testing ITAAC can be combined when the tests involve related components.

Consolidation of these ITP ITAAC reduces administrative and regulatory burden on the licensee and the regulator by combining multiple closure notifications into a single closure notification that more closely aligns to the technical processes and documentation required by the Vogtle 3&4 COLs for performing integrated testing and generating the documentation needed to complete and close these ITAAC. In consolidating these ITAAC, the closure documentation will not change, and the scope of inspections, tests and analyses required for each system will not change. All of the proposed changes are administrative. Any ITAAC number proposed to be consolidated that has been previously identified as a targeted ITAAC is consolidated into another targeted ITAAC; therefore there is no reduction in regulatory oversight. Consolidation of ITP ITAAC in COL Appendix C (and plant-specific Tier 1, as applicable) does not alter the DC, ITA, or AC for the impacted ITAAC and, therefore, does not impact the scope of the 10 CFR 52.103(g) finding to be made by the Commission, indicating that the Acceptance Criteria (AC) in COL Appendix C are met.

Enclosure 4 provides a roadmap of all the proposed consolidation changes and identifies where each ITAAC will now be hosted.

In the example below, 12 ITP ITAAC in the Chemical and Volume Control System (CVS) are proposed to be consolidated into one ITAAC, 2.3.02.08a.i, based on the following:

- ITAAC Numbers 2.3.02.08a.i, 2.3.02.08b, 2.3.02.11a.iii, 2.3.02.11a.iv are all completed in pre-operational system testing.
- ITAAC Numbers 2.3.02.09, 2.3.02.10a, 2.3.02.10b.i, 2.3.02.10b.ii, and 2.3.02.11b are component testing ITAAC related to valves in Table 2.3.2-1, and verify various capabilities of those components and their interfaces with other systems.
- ITAAC Numbers 2.3.02.12a, 2.3.02.12b, and 2.3.02.13 are component tests related to pumps and other CVS components in Table 2.3.2-3, and verify various capabilities of those components and their interfaces with other systems.

After consolidation, ITAAC 2.3.02.08a.i would appear as shown in Table 1 below. The new, consolidated ITAAC retains all the text from the 12 individual ITAAC without any changes.

Table 1. Consolidated ITP CVS ITAAC Example

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
301	2.3.02.08a.i	8.a) The CVS provides makeup water to the RCS.	i) Testing will be performed by aligning a flow path from each CVS makeup pump, actuating makeup flow to the RCS at pressure greater than or equal to 2000 psia, and measuring the flow rate in the makeup pump discharge line with each pump suction aligned to the boric acid storage tank.	i) Each CVS makeup pump provides a flow rate of greater than or equal to 100 gpm.
		8.b) The CVS provides the pressurizer auxiliary spray.	Testing will be performed by aligning a flow path from each CVS makeup pump to the pressurizer auxiliary spray and measuring the flow rate in the makeup pump discharge line with each pump suction aligned to the boric acid storage tank and with RCS pressure greater than or equal to 2000 psia.	Each CVS makeup pump provides spray flow to the pressurizer.
		9. Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.
		10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.	Stroke testing will be performed on the remotely operated valves identified in Table 2.3.2-1 using the controls in the MCR.	Controls in the MCR operate to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.
		10.b) The valves identified in Table 2.3.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	i) Testing will be performed using real or simulated signals into the PMS.	i) The valves identified in Table 2.3.2-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.
			ii) Testing will be performed to demonstrate that the remotely operated CVS isolation valves CVS-V090, V091, V136A/B close within the required response time.	ii) These valves close within the following times after receipt of an actuation signal: V090, V091 < 30 sec V136A/B < 20 sec

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		11.a) The motor-operated and check valves identified in Table 2.3.2-1 perform an active safety-related function to change position as indicated in the table.	iii) Tests of the motor-operated valves will be performed under pre-operational flow, differential pressure, and temperature conditions.	iii) Each motor-operated valve changes position as indicated in Table 2.3.2-1 under pre-operational test conditions.
			iv) Exercise testing of the check valves with active safety functions identified in Table 2.3.2-1 will be performed under pre-operational test pressure, temperature and fluid flow conditions.	iv) Each check valve changes position as indicated in Table 2.3.2-1.
		11.b) After loss of motive power, the remotely operated valves identified in Table 2.3.2-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	Upon loss of motive power, each remotely operated valve identified in Table 2.3.2-1 assumes the indicated loss of motive power position.
		12.a) Controls exist in the MCR to cause the pumps identified in Table 2.3.2-3 to perform the listed function.	Testing will be performed to actuate the pumps identified in Table 2.3.2-3 using controls in the MCR.	Controls in the MCR cause pumps identified in Table 2.3.2-3 to perform the listed function.
		12.b) The pumps identified in Table 2.3.2-3 start after receiving a signal from the PLS.	Testing will be performed to confirm starting of the pumps identified in Table 2.3.2-3.	The pumps identified in Table 2.3.2-3 start after a signal is generated by the PLS.
		13. Displays of the parameters identified in Table 2.3.2-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.3.2-3 in the MCR.	Displays identified in Table 2.3.2-3 can be retrieved in the MCR.
***				
304	2.3.02.08b	Not used per Amendment No. XX ~~8.b) The CVS provides the pressurizer auxiliary spray.~~	~~Testing will be performed by aligning a flow path from each CVS makeup pump to the pressurizer auxiliary spray and measuring the flow rate in the makeup pump discharge line with each pump suction aligned to the boric acid storage tank and with RCS pressure greater than or equal to 2000 psia.~~	~~Each CVS makeup pump provides spray flow to the pressurizer.~~

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
305	2.3.02.09	Not used per Amendment No. XX ~~9. Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.~~
306	2.3.02.10a	Not used per Amendment No. XX ~~10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.~~	~~Stroke testing will be performed on the remotely operated valves identified in Table 2.3.2-1 using the controls in the MCR.~~	~~Controls in the MCR operate to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.~~
307	2.3.02.10b.i	Not used per Amendment No. XX ~~10.b) The valves identified in Table 2.3.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.~~	~~i) Testing will be performed using real or simulated signals into the PMS.~~	~~i) The valves identified in Table 2.3.2-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.~~
308	2.3.02.10b.ii	Not used per Amendment No. XX ~~10.b) The valves identified in Table 2.3.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.~~	~~ii) Testing will be performed to demonstrate that the remotely operated CVS isolation valves CVS-V090, V091, V136A/B close within the required response time.~~	~~ii) These valves close within the following times after receipt of an actuation signal:~~ ~~V090, V091 ≤ 30 sec~~ ~~V136A/B ≤ 20 sec~~
***				
311	2.3.02.11a.iii	Not used per Amendment No. XX ~~11.a) The motor-operated and check valves identified in Table 2.3.2-1 perform an active safety-related function to change position as indicated in the table.~~	~~iii) Tests of the motor-operated valves will be performed under pre-operational flow, differential pressure, and temperature conditions.~~	~~iii) Each motor-operated valve changes position as indicated in Table 2.3.2-1 under pre-operational test conditions.~~
312	2.3.02.11a.iv	Not used per Amendment No. XX ~~11.a) The motor-operated and check valves identified in Table 2.3.2-1 perform an active safety-related function to change position as indicated in the table.~~	~~iv) Exercise testing of the check valves with active safety functions identified in Table 2.3.2-1 will be performed under pre-operational test pressure, temperature and fluid flow conditions.~~	~~iv) Each check valve changes position as indicated in Table 2.3.2-1.~~
313	2.3.02.11b	Not used per Amendment No. XX ~~11.b) After loss of motive power, the remotely operated valves identified in Table 2.3.2-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~Upon loss of motive power, each remotely operated valve identified in Table 2.3.2-1 assumes the indicated loss of motive power position.~~

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
314	2.3.02.12a	Not used per Amendment No. XX ~~12.a) Controls exist in the MCR to cause the pumps identified in Table 2.3.2-3 to perform the listed function.~~	~~Testing will be performed to actuate the pumps identified in Table 2.3.2-3 using controls in the MCR.~~	~~Controls in the MCR cause pumps identified in Table 2.3.2-3 to perform the listed function.~~
315	2.3.02.12b	Not used per Amendment No. XX ~~12.b) The pumps identified in Table 2.3.2-3 start after receiving a signal from the PLS.~~	~~Testing will be performed to confirm starting of the pumps identified in Table 2.3.2-3.~~	~~The pumps identified in Table 2.3.2-3 start after a signal is generated by the PLS.~~
316	2.3.02.13	Not used per Amendment No. XX ~~13. Displays of the parameters identified in Table 2.3.2-3 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the displays identified in Table 2.3.2-3 in the MCR.~~	~~Displays identified in Table 2.3.2-3 can be retrieved in the MCR.~~
***				

Category 2 – "Initial Test Program and Related Inspections" ITAAC

In addition to the ITP ITAAC described in Category 1, there are ITAAC that require inspections related to the components or system being tested. These ITAAC, combined with one or more of the groupings of ITAAC in Category 1, are hereafter referred to collectively as ITP and Related Inspections ITAAC or ITAAC in Category 2. Within this category of ITP and related inspections ITAAC, 50 ITAAC are proposed to be consolidated into eight ITAAC. In this category of ITAAC being consolidated, one or more tests and related inspections are used to complete several related ITAAC within the same system and therefore consolidation into a single ITAAC is logical. Inspections of components, where the components have ITP ITAAC, can be combined with the ITP ITAAC because the inspections are related to the ITP ITAAC. The proposed changes include consolidation of inspections with one or more component testing ITAAC, pre-operational testing ITAAC, or a combination of one or more component testing ITAAC and pre-operational testing ITAAC.

Consolidation of these ITP and Related Inspections ITAAC reduces administrative and regulatory burden on the licensee and the regulator by combining multiple closure notifications into a single closure notification that more closely aligns to the technical processes and documentation required by the Vogtle 3&4 COLs for performing integrated testing and inspections, and generating the documentation needed to complete and close these ITAAC. In consolidating these ITAAC, it should be noted that the closure documentation will not change, and the scope of inspections, tests and analyses required for each system will not change. All of the proposed changes are administrative. Any ITAAC number proposed to be consolidated that has been previously identified as a targeted ITAAC is consolidated into another targeted ITAAC; therefore there is no reduction in regulatory oversight. Consolidation of ITP ITAAC in COL Appendix C (and plant-specific Tier 1, as applicable) does not alter the DC, ITA, or AC for the impacted ITAAC and, therefore, does not impact the scope of the 10 CFR 52.103(g) finding to be made by the Commission, indicating that the Acceptance Criteria (AC) in COL Appendix C are met.

Enclosure 4 provides a roadmap of the proposed consolidation changes and identifies where each ITAAC will now be hosted.

In the example below, five ITP and Related Inspections ITAAC in the Fuel Handling System (FHS) are proposed to be consolidated into one ITAAC, 2.1.01.04, based on the following:

- ITAAC Numbers 2.1.01.04 and 2.1.01.05 are completed in pre-operational system testing.
- ITAAC Numbers 2.1.01.02, 2.1.01.06.i, 2.1.01.07.ii are inspections related to the physical attributes of the system

After consolidation, ITAAC 2.1.01.04 would appear as shown in Table 2 below. The new, consolidated ITAAC retains all the text from the five individual ITAAC without any changes.

Table 2. Consolidated ITP and Related Inspections FHS ITAAC Example

Table 2.1.1-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
2	2.1.01.02	Not used per Amendment No. XX ~~2. The FHS has the refueling machine (RM), the fuel handling machine (FHM), and the new and spent fuel storage racks.~~	~~Inspection of the system will be performed.~~	~~The FHS has the RM, the FHM, and the new and spent fuel storage racks.~~
***				
4	2.1.01.04	2. The FHS has the refueling machine (RM), the fuel handling machine (FHM), and the new and spent fuel storage racks. 4. The RM and FHM/spent fuel handling tool (SFHT) gripper assemblies are designed to prevent opening while the weight of the fuel assembly is suspended from the grippers. 5. The lift height of the RM mast and FHM hoist(s) is limited such that the minimum required depth of water shielding is maintained. 6. The RM and FHM are designed to maintain their load carrying and structural integrity functions during a safe shutdown earthquake. 7. The new and spent fuel storage racks maintain the effective neutron multiplication factor required by 10 CFR 50.68 limits during normal operation, design basis seismic events, and design basis dropped spent fuel assembly accidents over the spent fuel storage racks.	Inspection of the system will be performed. The RM and FHM/SFHT gripper assemblies will be tested by operating the open controls of the gripper while suspending a dummy fuel assembly. The RM and FHM will be tested by attempting to raise a dummy fuel assembly. i) Inspection will be performed to verify that the RM and FHM are located on the nuclear island. ii) Inspection will be performed to verify that the new and spent fuel storage racks are located on the nuclear island.	The FHS has the RM, the FHM, and the new and spent fuel storage racks. The RM and FHM/SFHT gripper assemblies will not open while suspending a dummy test assembly. The bottom of the dummy fuel assembly cannot be raised to within 24 ft, 6 in. of the operating deck floor. i) The RM and FHM are located on the nuclear island. ii) The new and spent fuel storage racks are located on the nuclear island.
5	2.1.01.05	Not used per Amendment No. XX ~~5. The lift height of the RM mast and FHM hoist(s) is limited such that the minimum required depth of water shielding is maintained.~~	~~The RM and FHM will be tested by attempting to raise a dummy fuel assembly.~~	~~The bottom of the dummy fuel assembly cannot be raised to within 24 ft, 6 in. of the operating deck floor.~~
6	2.1.01.06.i	Not used per Amendment No. XX ~~6. The RM and FHM are designed to maintain their load carrying and structural integrity functions during a safe shutdown earthquake.~~	~~i) Inspection will be performed to verify that the RM and FHM are located on the nuclear island.~~	~~i) The RM and FHM are located on the nuclear island.~~

Table 2.1.1-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
9	2.1.01.07.ii	Not used per Amendment No. XX ~~7. The new and spent fuel storage racks maintain the effective neutron multiplication factor required by 10 CFR 50.68 limits during normal operation, design basis seismic events, and design basis dropped spent fuel assembly accidents over the spent fuel storage racks.~~	~~ii) Inspection will be performed to verify that the new and spent fuel storage racks are located on the nuclear island.~~	~~ii) The new and spent fuel storage racks are located on the nuclear island.~~
***				

Category 3 – "Related Inspections and Analyses" ITAAC

Similar to the ITAAC addressed in Category 2, there are ITAAC that require inspections, tests outside of ITP scope, or analyses on related components or systems. These ITAAC are hereafter referred to collectively as Related Inspections and Analyses ITAAC in Category 3. In this category of ITAAC being consolidated, one or more related inspections or analyses are used to complete several related ITAAC within the same system and therefore consolidation into a single ITAAC is logical. Inspections of physical attributes of the system, including components within the system, can be combined with other related inspections in the same system. Within this category 32 ITAAC are proposed to be consolidated into 11 ITAAC. The proposed changes include consolidation of inspections, tests outside of ITP scope, and analyses.

Consolidation of these Related Inspections and Analyses ITAAC reduces administrative and regulatory burden on the licensee and the regulator by combining multiple closure notifications into a single closure notification that more closely aligns to the technical processes and documentation required by the Vogtle 3&4 COLs for performing related testing, inspections, and analyses, and generating the documentation needed to complete and close these ITAAC. In consolidating these ITAAC, it should be noted that the closure documentation will not change, and the scope of inspections, tests and analyses required for each system will not change. All of the proposed changes are administrative. Any ITAAC number proposed to be consolidated that has been previously identified as a targeted ITAAC is consolidated into another targeted ITAAC; therefore there is no reduction in regulatory oversight. Consolidation of ITP ITAAC in COL Appendix C (and plant-specific Tier 1, as applicable) does not alter the DC, ITA, or AC for the impacted ITAAC and, therefore, does not impact the scope of the 10 CFR 52.103(g) finding to be made by the Commission, indicating that the Acceptance Criteria (AC) in COL Appendix C are met.

Enclosure 4 provides a roadmap of the proposed consolidation changes and identifies where each ITAAC will now be hosted.

In the example below, three Related Inspections and Analyses ITAAC in the Fire Protection System (FPS) are proposed to be consolidated into one ITAAC, 2.3.04.04.i, based on the following:

- ITAAC Numbers 2.3.04.04.i and 2.3.04.07 are all related requirements to perform inspections of volumes for a common system.
- ITAAC Number 2.3.04.06 requires an inspection of other components in the same system as the above ITAAC.
- The proposed ITAAC to be combined in the FPS system all require inspections of physical attributes required by the FPS system.

After consolidation, ITAAC 2.3.04.04.i would appear as shown in Table 3 below. The new, consolidated ITAAC retains all the text from the three individual ITAAC without any changes.

Table 3. Related Inspections and Analyses FPS ITAAC Example

Table 2.3.4-2 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
***				
330	2.3.04.04.i	4. The FPS provides for manual fire fighting capability in plant areas containing safety-related equipment. 6. The FPS provides nonsafety-related containment spray for severe accident management. 7. The FPS provides two fire water storage tanks, each capable of holding at least 300,000 gallons of water.	i) Inspection of the passive containment cooling system (PCS) storage tank will be performed. Inspection of the containment spray headers will be performed. Inspection of each fire water storage tank will be performed.	i) The volume of the PCS tank above the standpipe feeding the FPS and below the overflow is at least 18,000 gal. The FPS has spray headers and nozzles as follows: At least 44 nozzles at plant elevation of at least 260 feet, and 24 nozzles at plant elevation of at least 275 feet. The volume of each fire water storage tank supplying the FPS is at least 300,000 gallons.
***				
333	2.3.04.06	Not used per Amendment No. XX ~~6. The FPS provides nonsafety-related containment spray for severe accident management.~~	~~Inspection of the containment spray headers will be performed.~~	~~The FPS has spray headers and nozzles as follows:~~ ~~At least 44 nozzles at plant elevation of at least 260 feet, and 24 nozzles at plant elevation of at least 275 feet.~~

Table 2.3.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
334	2.3.04.07	Not used per Amendment No. XX ~~7. The FPS provides two fire water storage tanks, each capable of holding at least 300,000 gallons of water.~~	~~Inspection of each fire water storage tank will be performed.~~	~~The volume of each fire water storage tank supplying the FPS is at least 300,000 gallons.~~
***				

Category 4 – "Security Testing" ITAAC

Similar to the ITAAC addressed in Category 3, there are ITAAC that require inspections, tests outside of ITP scope or analyses in the security system. These ITAAC are hereafter referred to collectively as Security Testing ITAAC or ITAAC in Category 4. In this category of ITAAC being consolidated, one or more related inspections, tests, analyses, or some combination of the three, are used to complete several related ITAAC within the security system and therefore consolidation into a single ITAAC is logical. Within this category of Security Testing ITAAC, 18 ITAAC are proposed to be consolidated into eight ITAAC. Inspections of physical attributes of the security system, including components and features within the system, can be combined with other related inspections in the security system. The proposed changes include consolidation of inspections, tests outside of ITP scope, and analyses, or some combination of the three. Note that there are standard plant security system ITAAC and plant-specific security system ITAAC in the proposed changes. By combining standard plant ITAAC into plant-specific ITAAC, there is still no impact to the DC, ITA, and AC of the ITAAC between the two systems.

Consolidation of the Security Testing ITAAC reduces administrative and regulatory burden on the licensee and the regulator by combining multiple closure notifications into a single closure notification that more closely aligns to the technical processes and documentation required by the Vogtle 3&4 COLs for performing integrated security testing, inspections, and analyses, and generating the documentation needed to complete and close these ITAAC. In consolidating these ITAAC, it should be noted that the closure documentation will not change, and the scope of inspections, tests and analyses required for the security system will not change. All of the proposed changes are administrative. Any ITAAC number proposed to be consolidated that has been previously identified as a targeted ITAAC is consolidated into another targeted ITAAC; therefore there is no reduction in regulatory oversight. Consolidation of Security Testing ITAAC in COL Appendix C (and plant-specific Tier 1, as applicable) does not alter the DC, ITA, or AC for the impacted ITAAC and, therefore, does not impact the scope of the 10 CFR 52.103(g) finding to be made by the Commission, indicating that the Acceptance Criteria (AC) in COL Appendix C are met.

Enclosure 4 provides a roadmap of the proposed consolidation changes and identifies where each ITAAC will now be hosted.

In the example below, three Related Inspections and Analyses ITAAC in the Security System are proposed to be consolidated into one ITAAC, C.2.6.09.03b based on the following:

- ITAAC Number C.2.6.09.03b inspects the intrusion detection equipment for the isolation zones.
- ITAAC Numbers C.2.6.09.04a and C.2.6.09.04b then inspect and/or test the intrusion detection equipment for required capabilities.
- The proposed ITAAC are all related to intrusion detection equipment and its capabilities.

After consolidation, ITAAC C.2.6.09.03b would appear as shown in Table 4 below. The new, consolidated ITAAC retains all the text from the three individual ITAAC without any changes.

Table 4. Security ITAAC Example

Table C.2.6.9-2 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
***				
661	C.2.6.09.03b	3.b) The isolation zones are monitored with intrusion detection equipment that provides the capability to detect and assess unauthorized persons. <u>4. The intrusion detection and assessment equipment at the protected area perimeter:</u> <u>a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in both the Central Alarm Station and Secondary Alarm Station;</u> <u>b) remains operable from an uninterruptible power supply in the event of the loss of normal power.</u>	Inspections will be performed of the intrusion detection equipment within the isolation zones. <u>Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.</u> <u>Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.</u>	The isolation zones are equipped with intrusion detection equipment that provides the capability to detect and assess unauthorized persons. <u>The intrusion detection and assessment equipment at the protected area perimeter:</u> <u>a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in the Central Alarm Station and Secondary Alarm Station;</u> <u>b) remains operable from an uninterruptible power supply in the event of the loss of normal power.</u>

Table C.2.6.9-2 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
662	C.2.6.09.04a	Not used per Amendment No. XX ~~4. The intrusion detection and assessment equipment at the protected area perimeter:~~ ~~a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in both the Central Alarm Station and Secondary Alarm Station;~~	~~Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.~~	~~The intrusion detection and assessment equipment at the protected area perimeter:~~ ~~a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in the Central Alarm Station and Secondary Alarm Station;~~
663	C.2.6.09.04b	Not used per Amendment No. XX ~~4. The intrusion detection and assessment equipment at the protected area perimeter:~~ ~~b) remains operable from an uninterruptible power supply in the event of the loss of normal power.~~	~~Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.~~	~~The intrusion detection and assessment equipment at the protected area perimeter:~~ ~~b) remains operable from an uninterruptible power supply in the event of the loss of normal power.~~

Proposed Licensing Basis Changes

The following changes to COL Appendix C (and plant-specific Tier 1) ITAAC Tables are proposed:

Table 5. Proposed Licensing Basis Changes

Licensing Basis Section	ITAAC Number to Be Removed	Host ITAAC Number
COL Appendix C (and plant-specific Tier 1) Table 2.1.1-1	2.1.01.02 2.1.01.05 2.1.01.06.i 2.1.01.07.ii	2.1.01.04
COL Appendix C Table 2.1.2-4	2.1.02.08d.vi 2.1.02.08d.vii 2.1.02.08d.viii	2.1.02.08d.v
COL Appendix C (and plant-specific Tier 1) Table 2.1.2-4	2.1.02.10 2.1.02.11b.ii 2.1.02.11b.iii 2.1.02.12b	2.1.02.11a.ii

Licensing Basis Section	ITAAC Number to Be Removed	Host ITAAC Number
COL Appendix C (and plant-specific Tier 1) Table 2.1.3-2	2.1.03.02b 2.1.03.02c	2.1.03.02a
COL Appendix C (and plant-specific Tier 1) Table 2.1.3-2	2.1.03.07.ii 2.1.03.10	2.1.03.07.i
COL Appendix C (and plant-specific Tier 1) Table 2.2.1-3	2.2.01.10a 2.2.01.10b	2.2.01.09
COL Appendix C (and plant-specific Tier 1) Table 2.2.2-3	2.2.02.07f.ii 2.2.02.08a	2.2.02.07a.iii
COL Appendix C (and plant-specific Tier 1) Table 2.2.2-3	2.2.02.07a.i 2.2.02.07a.ii 2.2.02.07b.ii 2.2.02.07b.iii 2.2.02.07c 2.2.02.07d 2.2.02.07e.ii 2.2.02.09 2.2.02.10a 2.2.02.10b 2.2.02.11a.iii 2.2.02.11b	2.2.02.07b.i
COL Appendix C (and plant-specific Tier 1) Table 2.2.2-3	2.2.02.08b	2.2.02.07f.i
COL Appendix C (and plant-specific Tier 1) Table 2.2.3-4	2.2.03.08c.v.02	2.2.03.08c.iv.01
COL Appendix C (and plant-specific Tier 1) Table 2.2.3-4	2.2.03.11a.ii 2.2.03.11b.ii 2.2.03.11b.iii 2.2.03.12b 2.2.03.13	2.2.03.10
COL Appendix C (and plant-specific Tier 1) Table 2.2.2-4	2.2.04.09a.i 2.2.04.10 2.2.04.11a 2.2.04.11b.i 2.2.04.11b.ii 2.2.04.12b	2.2.04.12a.iii

Licensing Basis Section	ITAAC Number to Be Removed	Host ITAAC Number
COL Appendix C (and plant-specific Tier 1) Table 2.2.2-5	2.2.05.07a.iii 2.2.05.07b.i 2.2.05.07b.ii 2.2.05.07d 2.2.05.08 2.2.05.09a 2.2.05.09b 2.2.05.10 2.2.05.11 2.2.05.12	2.2.05.07a.i
COL Appendix C (and plant-specific Tier 1) Table 2.3.1-2	2.3.01.04 2.3.01.05	2.3.01.03.ii
COL Appendix C (and plant-specific Tier 1) Table 2.3.2-4	2.3.02.08b 2.3.02.09 2.3.02.10a 2.3.02.10b.i 2.3.02.10b.ii 2.3.02.11a.iii 2.3.02.11a.iv 2.3.02.11b 2.3.02.12a 2.3.02.12b 2.3.02.13	2.3.02.08a.i
COL Appendix C (and plant-specific Tier 1) Table 2.3.3-2	2.3.03.05	2.3.03.04
COL Appendix C (and plant-specific Tier 1) Table 2.3.4-2	2.3.04.06 2.3.04.07	2.3.04.04.i
COL Appendix C Table 2.3.5-2	2.3.05.03a.iii	2.3.05.03a.ii
COL Appendix C (and plant-specific Tier 1) Table 2.3.5-2	2.3.05.03b.ii 2.3.05.04	2.3.05.03b.iii
COL Appendix C (and plant-specific Tier 1) Table 2.3.6-4	2.3.06.09b.iii 2.3.06.09b.iv 2.3.06.09b.v 2.3.06.09c 2.3.06.09d 2.3.06.12a.iii 2.3.06.12a.iv	2.3.06.09b.ii

Licensing Basis Section	ITAAC Number to Be Removed	Host ITAAC Number
COL Appendix C (and plant-specific Tier 1) Table 2.3.6-4	2.3.06.10 2.3.06.11b 2.3.06.12b 2.3.06.13 2.3.06.14	2.3.06.11a
COL Appendix C (and plant-specific Tier 1) Table 2.3.7-4	2.3.07.08.ii 2.3.07.09 2.3.07.10 2.3.07.11	2.3.07.07c
COL Appendix C (and plant-specific Tier 1) Table 2.3.8-2	2.3.08.03 2.3.08.04	2.3.08.02.i
COL Appendix C (and plant-specific Tier 1) Table 2.3.9-3	2.3.09.03.i 2.3.09.04a 2.3.09.05	2.3.09.03.ii
COL Appendix C (and plant-specific Tier 1) Table 2.3.10-4	2.3.10.07a.i 2.3.10.07b 2.3.10.08 2.3.10.09 2.3.10.10	2.3.10.07a.ii
COL Appendix C (and plant-specific Tier 1) Table 2.3.13-3	2.3.13.09 2.3.13.10a 2.3.13.10b 2.3.13.11b 2.3.13.12	2.3.13.08
COL Appendix C (and plant-specific Tier 1) Table 2.3.19-2	2.3.19.01a 2.3.19.01b 2.3.19.02b	2.3.19.02a
COL Appendix C (and plant-specific Tier 1) Table 2.3.29-1	2.3.29.03	2.3.29.02
COL Appendix C (and plant-specific Tier 1) Table 2.4.1-2	2.4.01.03 2.4.01.04	2.4.01.02
COL Appendix C (and plant-specific Tier 1) Table 2.4.2-1	2.4.02.02c 2.4.02.03.ii	2.4.02.02a

Licensing Basis Section	ITAAC Number to Be Removed	Host ITAAC Number
COL Appendix C (and plant-specific Tier 1) Table 2.5.1-4	2.5.01.02b 2.5.01.02c.i 2.5.01.02c.ii 2.5.01.02d 2.5.01.03f 2.5.01.03g	2.5.01.02a
COL Appendix C (and plant-specific Tier 1) Table 2.5.2-8	2.5.02.06b 2.5.02.06c.ii 2.5.02.08a.i 2.5.02.08a.iii 2.5.02.08c 2.5.02.09a 2.5.02.09b 2.5.02.09c	2.5.02.06a.ii
COL Appendix C Table 2.5.4-2	2.5.04.02.ii 2.5.04.02.iii	2.5.04.02.i
COL Appendix C Table 2.5.4-2	C.2.5.04.04b C.2.5.04.04c	C.2.5.04.04a
COL Appendix C (and plant-specific Tier 1) Table 2.6.1-4	2.6.01.04a 2.6.01.04f 2.6.01.05 2.6.01.06	2.6.01.04e
COL Appendix C (and plant-specific Tier 1) Table 2.6.3-3	2.6.03.04d 2.6.03.04e 2.6.03.04f 2.6.03.04g 2.6.03.04h 2.6.03.05a 2.6.03.05b 2.6.03.05c 2.6.03.06 2.6.03.11	2.6.03.04c
COL Appendix C Table 2.6.3-3	2.6.03.05d.ii	2.6.03.05d.i
COL Appendix C (and plant-specific Tier 1) Table 2.6.3-3	2.6.03.08 2.6.03.09 2.6.03.10	2.6.03.07

Licensing Basis Section	ITAAC Number to Be Removed	Host ITAAC Number
COL Appendix C (and plant-specific Tier 1) Table 2.6.4-1	2.6.04.02b 2.6.04.03 2.6.04.04	2.6.04.02a
COL Appendix C (and plant-specific Tier 1) Table 2.6.5-1	2.6.05.02.i 2.6.05.05.i 2.6.05.05.ii 2.6.05.06.i 2.6.05.06.ii	2.6.05.02.ii
COL Appendix C Table 2.6.6-1	2.6.06.01.ii 2.6.06.01.iii 2.6.06.01.iv	2.6.06.01.i
COL Appendix C (and plant-specific Tier 1) Table 2.6.9-1	2.6.09.15b	2.6.09.05a
COL Appendix C (and plant-specific Tier 1) Table 2.6.9-1 and COL Appendix C Table C.2.6.9-2	2.6.09.07a 2.6.09.07b	C.2.6.09.07
COL Appendix C (and plant-specific Tier 1) Table 2.6.9-1 and COL Appendix C Table C.2.6.9-2	2.6.09.09	C.2.6.09.09
COL Appendix C (and plant-specific Tier 1) Table 2.6.9-1	2.6.09.13b	2.6.09.13a
COL Appendix C (and plant-specific Tier 1) Table 2.6.9-1	2.6.09.16	2.6.09.15a
COL Appendix C Table C.2.6.9-2	C.2.6.09.04a C.2.6.09.04b	C.2.6.09.03b
COL Appendix C Table C.2.6.9-2	C.2.6.09.05b	C.2.6.09.05a
COL Appendix C Table C.2.6.9-2	C.2.6.09.08b	C.2.6.09.08a
COL Appendix C (and plant-specific Tier 1) Table 2.7.1-4	2.7.01.08d 2.7.01.09 2.7.01.10a 2.7.01.10b 2.7.01.11 2.7.01.12 2.7.01.13	2.7.01.14

Licensing Basis Section	ITAAC Number to Be Removed	Host ITAAC Number
COL Appendix C (and plant-specific Tier 1) Table 2.7.2-2	2.7.02.04 2.7.02.05	2.7.02.03a
COL Appendix C (and plant-specific Tier 1) Table 2.7.3-2	2.7.03.04	2.7.03.03
COL Appendix C (and plant-specific Tier 1) Table 2.7.4-2	2.7.04.04	2.7.04.03
COL Appendix C (and plant-specific Tier 1) Table 2.7.5-2	2.7.05.02.ii 2.7.05.02.iii 2.7.05.03	2.7.05.02.i
COL Appendix C (and plant-specific Tier 1) Table 2.7.6-2	2.7.06.03.ii 2.7.06.03.iii 2.7.06.04 2.7.06.05	2.7.06.03.i
COL Appendix C Table 3.3-6	3.3.00.10.iii	3.3.00.10.ii
COL Appendix C (and plant-specific Tier 1) Table 3.5-6	3.5.00.04 3.5.00.05 3.5.00.07 3.5.00.08	3.5.00.06

In COL Appendix C, the ITAAC Index Number and ITAAC Number of each moved ITAAC will remain. The Design Commitment column next to each moved ITAAC will now state “Not used per Amendment No. XX” where “XX” is a placeholder for the Unit specific COL Amendment numbers assigned by the NRC for approval of this LAR and will be inserted in this text when this amendment is approved and implemented.

For the changes where ITAAC 2.6.09.09 is relocated to ITAAC C.2.6.09.09 and ITAAC 2.6.09.07a and 2.6.09.07b are relocated to ITAAC C.2.6.09.07, and since plant-specific Tier 1 is applicable to both Units 3 and 4, the Design Commitment will be replaced with “Not used per Amendment Nos. XX and XX for VEGP Units 3 and 4, respectively” where XX is a placeholder for the COL Amendment number assigned by the NRC for each unit for approval of this LAR.

Summary

The proposed changes consolidate 249 ITAAC into 57 ITAAC in COL Appendix C (and plant-specific Tier 1, as applicable). The proposed changes relocate multiple ITAAC entries into a single ITAAC in order to minimize the number of ITAAC completion packages and ICNs. The above-mentioned licensing basis changes will also result in a change to the COL Appendix C (and corresponding plant-specific Tier 1, as applicable) table of contents. These changes are considered administrative in nature since no technical changes are being made, all existing ITAAC text is retained, the required inspections, tests and analyses are still being performed and the margin of safety is not reduced. A standalone roadmap for the consolidation of each ITAAC identified in the LAR is included in Enclosure 4 of this submittal.

3. TECHNICAL EVALUATION (Included in Section 2)

4. REGULATORY EVALUATION

4.1 Applicable Regulatory Requirements/Criteria

10 CFR 52.98(c) requires NRC approval for any modification to, addition to, or deletion from the terms and conditions of a COL. This activity involves a departure from COL Appendix C information, and a corresponding change to plant-specific Tier 1 information; therefore, this activity requires an amendment to the COL. Accordingly, NRC approval is required prior to making the plant-specific changes in this license amendment request.

10 CFR 52.98(f) requires NRC approval for any modification to, addition to, or deletion from the terms and conditions of a COL. This activity involves a departure from COL Appendix C information, and a corresponding change to plant-specific Tier 1 information; therefore, this activity requires an amendment to the COL. Accordingly, NRC approval is required prior to making the plant-specific changes in this license amendment request.

10 CFR 52.97(b) requires that the Commission shall identify within the combined license the inspections, tests, and analyses, including those applicable to emergency planning, that the licensee shall perform, and the acceptance criteria that, if met, are necessary and sufficient to provide reasonable assurance that the facility has been constructed and will be operated in conformity with the license, the provisions of the Act, and the Commission's rules and regulations. Based on the technical evaluations provided in Section 2 above, the proposed changes to consolidate ITAAC continue to meet the requirements of 10 CFR 52.97(b).

4.2 Precedent

No precedent is identified.

4.3 Significant Hazards Consideration Determination

The proposed changes would require non-technical changes to COL Appendix C information. These changes consolidate and relocate Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) to improve efficiency of the ITAAC completion and closure process.

An evaluation to determine whether or not a significant hazards consideration is involved with the proposed amendment was completed by focusing on the three standards set forth in 10 CFR 50.92, “Issuance of amendment,” as discussed below:

4.3.1 Does the proposed amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No

The proposed non-technical change to COL Appendix C will consolidate ITAAC in order to improve and create a more efficient process for the ITAAC Closure Notification submittals. No structure, system, or component (SSC) design or function is affected. No design or safety analysis is affected. The proposed changes do not affect any accident initiating event or component failure, thus the probabilities of the accidents previously evaluated are not affected. No function used to mitigate a radioactive material release and no radioactive material release source term is involved, thus the radiological releases in the accident analyses are not affected.

Therefore, the proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

4.3.2 Does the proposed amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No

The proposed change to COL Appendix C does not affect the design or function of any SSC, but will consolidate ITAAC in order to improve efficiency of the ITAAC completion and closure process. The proposed changes would not introduce a new failure mode, fault or sequence of events that could result in a radioactive material release.

Therefore, the proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

4.3.3 Does the proposed amendment involve a significant reduction in a margin of safety?

Response: No

The proposed change to COL Appendix C to consolidate ITAAC in order to improve efficiency of the ITAAC completion and closure process is considered non-technical and would not affect any design parameter, function or analysis. There would be no change to an existing design basis, design function, regulatory criterion, or analysis. No safety analysis or design basis acceptance limit/criterion is involved. Therefore, the proposed amendment does not involve a significant reduction in a margin of safety.

Based on the above, it is concluded that the proposed amendment does not involve a significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of “no significant hazards consideration” is justified.

4.4 Conclusions

In conclusion, based on the considerations discussed above, (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public. Pursuant to 10 CFR 50.92, the requested change does not involve a Significant Hazards Consideration Determination.

5. ENVIRONMENTAL CONSIDERATIONS

The proposed changes would require non-technical changes to COL Appendix C information. The changes consolidate Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) to improve efficiency of the ITAAC completion and closure process.

A review has determined that the anticipated construction and operational effects of the proposed amendment meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9), in that:

(i) There is no significant hazards consideration.

As documented in Section 4.3, Significant Hazards Consideration Determination, of this license amendment request, an evaluation was completed to determine whether or not a significant hazards consideration is involved by focusing on the three standards set forth in 10 CFR 50.92, "Issuance of amendment." The Significant Hazards Consideration Determination determined that (1) the proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated; (2) the proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated; and (3) the proposed amendment does not involve a significant reduction in a margin of safety. Therefore, it is concluded that the proposed amendment does not involve a significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and accordingly, a finding of "no significant hazards consideration" is justified.

(ii) There is no significant change in the types or significant increase in the amounts of any effluents that may be released offsite.

The proposed changes to COL Appendix C are administrative changes to consolidate ITAAC in order to create a more efficient process for the ITAAC Closure Notification submittals. The proposed changes are unrelated to any aspect of plant construction or operation that would introduce any change to effluent types (e.g., effluents containing chemicals or biocides, sanitary system effluents, and other effluents), or affect any plant radiological or non-radiological effluent release quantities. Furthermore, the proposed changes do not affect any effluent release path or diminish the functionality of any design or operational features that are credited with controlling the release of effluents during plant operation. Therefore, it is concluded that the proposed amendment does not involve a significant change in the types or a significant increase in the amounts of any effluents that may be released offsite.

(iii) There is no significant increase in individual or cumulative occupational radiation

exposure.

The proposed changes to COL Appendix C are administrative changes to consolidate ITAAC in order to create a more efficient process for the ITAAC Closure Notification submittals. Plant radiation zones (addressed in UFSAR Section 12.3) are not affected, and controls under 10 CFR 20 preclude a significant increase in occupational radiation exposure. Therefore, the proposed amendment does not involve a significant increase in individual or cumulative occupational radiation exposure.

Based on the above review of the proposed amendment, it has been determined that anticipated construction and operational impacts of the proposed amendment do not involve (i) a significant hazards consideration, (ii) a significant change in the types or significant increase in the amounts of any effluents that may be released offsite, or (iii) a significant increase in the individual or cumulative occupational radiation exposure. Accordingly, the proposed amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), an environmental impact statement or environmental assessment of the proposed exemption is not required.

6. REFERENCES

1. Regulatory Guide 1.215, Revision 2, “Guidance for ITAAC Closure under 10 CFR Part 52”

2. NEI 08-01, Revision 5 - Corrected “Industry Guideline for the ITAAC Closure Process under 10 CFR Part 52”

Southern Nuclear Operating Company

ND-17-0XXX

Enclosure 2

Vogtle Electric Generating Plant (VEGP) Units 3 and 4

Exemption Request:

Testing Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) Consolidation

(LAR-17-038)

(This Enclosure consists of 12 pages, including this cover page.)

1.0 Purpose

Southern Nuclear Operating Company (SNC), the Licensee, requests a permanent exemption from the provisions of 10 CFR 52, Appendix D, Section III.B, "Design Certification Rule for the AP1000 Design, Scope and Contents," to allow a departure from elements of the certified information in Tier 1 of the generic AP1000 Design Control Document (DCD). The regulation, 10 CFR 52, Appendix D, Section III.B, requires an applicant or licensee referencing Appendix D to 10 CFR Part 52 to incorporate by reference and comply with the requirements of Appendix D, including certification information in DCD Tier 1. Tier 1 includes ITAAC that must be satisfactorily performed prior to fuel load. The design details to be verified by these ITAAC are specified in the text, tables, and figures that are referenced in each individual ITAAC. The generic Tier 1 information from which a plant-specific departure and exemption is requested includes ITAAC to be consolidated within plant-specific Tier 1, and plant-specific Tier 1 Security System ITAAC to be consolidated into VEGP 3 and 4 COL Appendix C site specific security system ITAAC. The specific information for which a plant-specific departure and exemption is being requested is as follows:

Tier 1 Section	Table ITAAC Item to Be Removed	Host ITAAC Item
Tier 1 Table 2.1.1-1	Item 2 Item 5 Item 6.i Item 7.ii	Item 4
Tier 1 Table 2.1.2-4	Item 10 Item 11b.ii Item 11b.iii Item 12b	Item 11a.ii
Tier 1 Table 2.1.3-2	Item 2b Item 2c	Item 2a
Tier 1 Table 2.1.3-2	Item 7.ii Item 10	Item 07.i
Tier 1 Table 2.2.1-3	Item 10a Item 10b	Item 09
Tier 1 Table 2.2.2-3	Item 7f.ii Item 8a	Item 7a.iii
Tier 1 Table 2.2.2-3	Item 7a.i Item 7a.ii Item 7b.ii Item 7b.iii Item 7c Item 7d	Item 7b.i

Tier 1 Section	**Table ITAAC Item to Be Removed**	**Host ITAAC Item**
	Item 7e.ii Item 9 Item 10a Item 10b Item 11a.iii Item 11b	
Tier 1 Table 2.2.2-3	Item 8b	Item 7f.i
Tier 1 Table 2.2.3-4	Item 8c.v.02	Item 8c.iv.01
Tier 1 Table 2.2.3-4	Item 11a.ii Item 11b.ii Item 11b.iii Item 12b Item 13	Item 10
Tier 1 Table 2.2.2-4	Item 9a.i Item 10 Item 11a Item 11b.i Item 11b.ii Item 12b	Item 12a.iii
Tier 1 Table 2.2.2-5	Item 7a.iii Item 7b.i Item 7b.ii Item 7d Item 08 Item 09a Item 09b Item 10 Item 11 Item 12	Item 7a.i
Tier 1 Table 2.3.1-2	Item 4 Item 5	Item 3.ii
Tier 1 Table 2.3.2-4	Item 8b Item 9 Item 10a	Item 8a.i

Tier 1 Section	Table ITAAC Item to Be Removed	Host ITAAC Item
	Item 10b.i Item 10b.ii Item 11a.iii Item 11a.iv Item 11b Item 12a Item 12b Item 13	
Tier 1 Table 2.3.3-2	Item 5	Item 4
Tier 1 Table 2.3.4-2	Item 6 Item 7	Item 4.i
Tier 1 Table 2.3.5-2	Item 3b.ii Item 4	Item 3b.iii
Tier 1 Table 2.3.6-4	Item 9b.iii Item 9b.iv Item 9b.v Item 9c Item 9d Item 12a.iii Item 12a.iv	Item 9b.ii
Tier 1 Table 2.3.6-4	Item 10 Item 11b Item 12b Item 13 Item 14	Item 11a
Tier 1 Table 2.3.7-4	Item 8.ii Item 9 Item 10 Item 11	Item 7c
Tier 1 Table 2.3.8-2	Item 3 Item 4	Item 2.i
Tier 1 Table 2.3.9-3	Item 3.i Item 4a Item 5	Item 3.ii
Tier 1 Table 2.3.10-4	Item 7a.i Item 7b	Item 7a.ii

Tier 1 Section	Table ITAAC Item to Be Removed	Host ITAAC Item
	Item 8 Item 9 Item 10	
Tier 1 Table 2.3.13-3	Item 9 Item 10a Item 10b Item 11b Item 12	Item 8
Tier 1 Table 2.3.19-2	Item 1a Item 1b Item 2b	Item 2a
Tier 1 Table 2.3.29-1	Item 3	Item 2
Tier 1 Table 2.4.1-2	Item 3 Item 4	Item 2
Tier 1 Table 2.4.2-1	Item 2c Item 3.ii	Item 2a
Tier 1 Table 2.5.1-4	Item 2b Item 2c.i Item 2c.ii Item 2d Item 3f Item 3g	Item 2a
Tier 1 Table 2.5.2-8	Item 6b Item 6c.ii Item 8a.i Item 8a.iii Item 8c Item 9a Item 9b Item 9c	Item 6a.ii
Tier 1 Table 2.6.1-4	Item 4a Item 4f Item 5 Item 6	Item 4e

Tier 1 Section	Table ITAAC Item to Be Removed	Host ITAAC Item
Tier 1 Table 2.6.3-3	Item 4d Item 4e Item 4f Item 4g Item 4h Item 5a Item 5b Item 5c Item 6 Item 11	Item 4c
Tier 1 Table 2.6.3-3	Item 8 Item 9 Item 10	Item 7
Tier 1 Table 2.6.4-1	Item 2b Item 3 Item 4	Item 2a
Tier 1 Table 2.6.5-1	Item 2.i Item 5.i Item 5.ii Item 6.i Item 6.ii	Item 2.ii
Tier 1 Table 2.6.9-1	Item 15b	Item 5a
Tier 1 Table 2.6.9-1	Item 7a Item 7b	COL Appendix C ITAAC C.2.6.09.07
Tier 1 Table 2.6.9-1	Item 9	COL Appendix C ITAAC C.2.6.09.09
Tier 1 Table 2.6.9-1	Item 13b	Item 13a
Tier 1 Table 2.6.9-1	Item 16	Item 15a
Tier 1 Table 2.7.1-4	Item 8d Item 9 Item 10a Item 10b	Item 14

Tier 1 Section	Table ITAAC Item to Be Removed	Host ITAAC Item
	Item 11 Item 12 Item 13	
Tier 1 Table 2.7.2-2	Item 4 Item 5	Item 3a
Tier 1 Table 2.7.3-2	Item 4	Item 3
Tier 1 Table 2.7.4-2	Item 4	Item 3
Tier 1 Table 2.7.5-2	Item 2.ii Item 2.iii Item 3	Item 2.i
Tier 1 Table 2.7.6-2	Item 3.ii Item 3.iii Item 4 Item 5	Item 3.i
Tier 1 Table 3.5-6	Item 4 Item 5 Item 7 Item 8	Item 6

This request for exemption provides the technical and regulatory basis to demonstrate that 10 CFR 52.63, §52.7, and §50.12 requirements are met and will apply the requirements of 10 CFR 52, Appendix D, Section VIII.A.4 to allow departures from generic Tier 1 information due to proposed consolidation, relocation and elimination of ITAAC.

2.0 Background

The Licensee is the holder of Combined License Nos. NPF-91 and NPF-92, which authorize construction and operation of two Westinghouse Electric Company AP1000 nuclear plants, named Vogtle Electric Generating Plant (VEGP) Units 3 and 4, respectively. The proposed changes would consolidate and relocate ITAAC.

During preparation and submittal of ITAAC Closure Notifications (ICNs), and through feedback by the Commission during review of the ICNs, SNC identified efficiencies to the ICN submittal process. Submittal of ICNs based upon the current plant-specific Tier 1 information creates additional regulatory burden on the Licensee and the NRC staff. The identified efficiencies would consolidate and relocate ITAAC to improve efficiency of the ITAAC completion and closure process. This activity requests exemption from the Generic DCD Tier 1 tables which support the associated COL Appendix C ITAAC.

An exemption from elements of the AP1000 certified (Tier 1) design information is requested to allow plant-specific departures to be taken from the Tier 1 ITAAC Tables listed in Section 1.0 of this Enclosure.

3.0 Technical Justification of Acceptability

An exemption is requested to depart from AP1000 Generic DCD Tier 1 material in regard to the AP1000 by consolidating and relocating ITAAC. Consolidation and relocation of ITAAC reduces documentation by reducing the number of ICNs because redundant documentation is not submitted. The proposed ITAAC consolidation continues to meet the intent of 10 CFR Part 52 Appendix D and plant-specific Tier 1 design descriptions, tables and figures. The proposed exemption would allow a change to the plant-specific Tier 1 ITAAC information consistent with existing plant-specific DCD Tier 2 information. The proposed changes to the description information presented in plant-specific Tier 1 are at a level of detail that is consistent with the information currently provided therein.

The proposed changes neither adversely impact the ability to meet the design functions of the SSCs nor involve a significant decrease in the level of safety provided by the structures, systems, or components. Because the proposed consolidations are consistent with plant-specific DCD Tier 2 information, the changes do not affect a structure, system or component. The proposed changes to information in plant- specific DCD Tier 1 continue to provide the detail necessary to implement the corresponding ITAAC.

Detailed technical justification supporting this request for exemption is provided in Section 2 of the associated License Amendment Request in Enclosure 1 of this letter

4.0 Justification of Exemption

10 CFR 52, Appendix D, Section VIII.A.4, 10 CFR 52.63(b)(1), and 52.98(f) govern the issuance of exemptions from elements of the certified design information for AP1000 nuclear power plants. Since SNC has identified changes to the Tier 1 information as discussed in Enclosure 1 of the accompanying License Amendment Request, an exemption to the certified design information in Tier 1 is needed.

10 CFR 52, Appendix D, and 10 CFR 50.12, §52.7, and §52.63 state that the NRC may grant exemptions from the requirements of the regulations provided six conditions are met: 1) the exemption is authorized by law [§50.12(a)(1)]; 2) the exemption will not present an undue risk to the health and safety of the public [§50.12(a)(1)]; 3) the exemption is consistent with the common defense and security [§50.12(a)(1)]; 4) special circumstances are present [§50.12(a)(2)(ii)]; 5) the special circumstances outweigh any decrease in safety that may result from the reduction in standardization caused by the exemption [§52.63(b)(1)]; and 6) the design change will not result in a significant decrease in the level of safety [Part 52, App. D, VIII.A.4].

The requested exemption satisfies the criteria for granting specific exemptions, as described below.

1. This exemption is authorized by law

The NRC has authority under 10 CFR 52.63, §52.7, and §50.12 to grant exemptions from the requirements of NRC regulations. Specifically, 10 CFR 50.12 and §52.7 state that the NRC may grant exemptions from the requirements of 10 CFR Part 52 upon a proper showing. No law exists that would preclude the changes covered by this exemption request. Additionally, granting of the proposed exemption does not result in a violation of the Atomic Energy Act of 1954, as amended, or the Commission's regulations.

Accordingly, this requested exemption is "authorized by law," as required by 10 CFR 50.12(a)(1).

2. **This exemption will not present an undue risk to the health and safety of the public**

   The proposed exemption from the requirements of 10 CFR 52, Appendix D, Section III.B, would allow changes to elements of the plant-specific DCD Tier 1 to depart from the AP1000 certified (Tier 1) design information. The plant-specific DCD Tier 1 will continue to reflect the approved licensing basis for VEGP Units 3 and 4 and will maintain a consistent level of detail with that which is currently provided elsewhere in Tier 1 of the DCD. Therefore, the affected plant-specific DCD Tier 1 ITAAC will continue to serve its required purpose.

   These changes will not impact the ability of the SSCs to perform their design functions. Because the changes will not alter the operation of any plant equipment or systems, these changes do not present an undue risk to existing equipment or systems. These changes do not add any new equipment or system interfaces to the current plant design. The description changes do not introduce any new industrial, chemical, or radiological hazards that would represent a public health or safety risk, nor do they modify or remove any design or operational controls or safeguards that are intended to mitigate any existing on-site hazards. Furthermore, the proposed changes would not allow for a new fission product release path, result in a new fission product barrier failure mode, or create a new sequence of events that would result in significant fuel cladding failures. Accordingly, these changes do not present an undue risk from any new equipment or systems.

   Therefore, the requested exemption from 10 CFR 52, Appendix D, Section III.B, would not present an undue risk to the health and safety of the public.

3. **The exemption is consistent with the common defense and security**

   The requested exemption from the requirements of 10 CFR 52, Appendix D, Section III.B, would allow the Licensee to depart from elements of the plant-specific DCD Tier 1 design information. The requested exemption does not alter the design, function, or operation of any structure or plant equipment that is necessary to maintain a safe and secure status of the plant. The requested exemption has no impact on plant security or safeguards procedures.

   Therefore, the requested exemption is consistent with the common defense and security.

4. **Special circumstances are present**

   10 CFR 50.12(a)(2) lists six "special circumstances" for which an exemption may be granted. Pursuant to the regulation, it is necessary for one of these special

circumstances to be present in order for the NRC to consider granting an exemption request. The requested exemption meets the special circumstances of 10 CFR 50.12(a)(2)(ii). That subsection defines special circumstances as when “Application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule.”

The rule under consideration in this request for exemption is 10 CFR 52, Appendix D, Section III.B, which requires that a licensee referencing the AP1000 Design Certification Rule (10 CFR Part 52, Appendix D) shall incorporate by reference and comply with the requirements of Appendix D, including Tier 1 information. The VEGP Units 3 and 4 COLs reference the AP1000 Design Certification Rule and incorporate by reference the requirements of 10 CFR Part 52, Appendix D, including Tier 1 information. The underlying purpose of Appendix D, Section III.B, is to describe and define the scope and contents of the AP1000 design certification, and to require compliance with the design certification information in Appendix D.

The proposed changes to consolidate and relocate ITAAC and subsume redundant ITA and AC maintain the design functions of these systems. This change does not impact the ability of any SSCs to perform their functions or negatively impact safety. Accordingly, this exemption from the certification information will enable the licensee to safely construct and operate the AP1000 facility consistent with the design certified by the NRC in 10 CFR 52, Appendix D.

Therefore, special circumstances are present, because application of the current generic certified design information in Tier 1 as required by 10 CFR Part 52, Appendix D, Section III.B, in the particular circumstances discussed in this request is not necessary to achieve the underlying purpose of the rule.

5. The special circumstances outweigh any decrease in safety that may result from the reduction in standardization caused by the exemption

Based on the nature of the changes to the plant-specific Tier 1 information in this area and the understanding that these changes are not related to system functions, these changes will not have a negative impact. Nevertheless, if other AP1000 licensees do not elect to request this exemption, the special circumstances continue to outweigh any decrease in safety from the reduction in standardization because the key design functions associated with this request will continue to be maintained. This exemption request and the associated marked-up table demonstrate that there is a minimal change from the generic AP1000 DCD, minimizing the reduction in standardization and, consequently, the safety impact from the reduction.

Therefore, the special circumstances associated with the requested exemption outweigh any decrease in safety that may result from the reduction in standardization caused by the exemption.

6. The design change will not result in a significant decrease in the level of safety

The proposed exemption would allow changes to consolidate and relocate ITAAC in plant-specific Tier 1. The consolidation will not impact the functional capabilities of the components identified in the affected ITAAC. Because the consolidation of ITAAC associated with this exemption request will not modify the design or operation of any systems or equipment, there are no new failure modes introduced by these changes and the level of safety provided by the current structures, systems, and components will be unchanged.

Because the proposed changes to the AP1000 certified (Tier 1) design information will not adversely affect the ability of the structures, systems or components to perform their design functions and the level of safety provided by the structures, systems, and components is unchanged, it is concluded that the changes associated with proposed exemption will not result in a significant decrease in the level of safety.

5.0 RISK ASSESSMENT

A risk assessment was not determined to be applicable to address the acceptability of this proposal.

6.0 PRECEDENT

None.

7.0 ENVIRONMENTAL CONSIDERATION

The Licensee requests a departure from elements of the certified information in Tier 1 of the generic AP1000 DCD. The Licensee has determined that the proposed departure would require a permanent exemption from the requirements of 10 CFR 52, Appendix D, Section III.B, Design Certification Rule for the AP1000 Design, Scope and Contents, with respect to installation or use of facility components located within the restricted area, as defined in 10 CFR Part 20, or which changes an inspection or a surveillance requirement; however, the Licensee evaluation of the proposed exemption has determined that the proposed exemption meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9).

Based on the above review of the proposed exemption, the Licensee has determined that the proposed activity does not involve (i) a significant hazards consideration, (ii) a significant change in the types or significant increase in the amounts of any effluents that may be released offsite, or (iii) a significant increase in the individual or cumulative occupational radiation exposure. Accordingly, the proposed exemption meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), an environmental impact statement or environmental assessment of the proposed exemption is not required.

Specific details of the environmental considerations supporting this request for exemption are provided in Section 5 of the associated License Amendment Request provided in Enclosure 1 of this letter.

8.0 CONCLUSION

The proposed changes to Tier 1 are necessary to consolidate information in ITAAC Tables in plant-specific DCD Tier 1 to improve efficiency of the ITAAC completion and closure process. The exemption request meets the requirements of 10 CFR 52.63, “Finality of Design Certifications,” 10 CFR 52.7, “Specific Exemptions,” 10 CFR 50.12, “Specific Exemptions,” and 10 CFR 52 Appendix D, “Design Certification Rule for the AP1000.”

Specifically, the exemption request meets the criteria of 10 CFR 50.12(a)(1) in that the request is authorized by law, presents no undue risk to public health and safety, and is consistent with the common defense and security. Furthermore, approval of this request does not result in a significant decrease in the level of safety, satisfies the underlying purpose of the AP1000 Design Certification Rule, and does not present a significant decrease in safety as a result of a reduction in standardization.

9.0 REFERENCES

None.

Southern Nuclear Operating Company

ND-17-0XXX

Enclosure 3

Vogtle Electric Generating Plant (VEGP) Units 3 and 4

Proposed Changes to COL Appendix C and plant-specific Tier 1

(LAR-17-038)

Insertions Denoted by Blue Underline and Deletions by Red Strikethrough
Omitted text is identified by three asterisks (*)**

(This Enclosure consists of 95 pages, including this cover page.)

Revise COL Appendix C (and plant-specific Tier 1), Section 2.1.1, Fuel Handling and Refueling System, Table 2.1.1-1, as shown below:

Table 2.1.1-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
2	2.1.01.02	Not used per Amendment No. XX ~~2. The FHS has the refueling machine (RM), the fuel handling machine (FHM), and the new and spent fuel storage racks.~~	~~Inspection of the system will be performed.~~	~~The FHS has the RM, the FHM, and the new and spent fuel storage racks.~~
***				
4	2.1.01.04	2. The FHS has the refueling machine (RM), the fuel handling machine (FHM), and the new and spent fuel storage racks.	Inspection of the system will be performed.	The FHS has the RM, the FHM, and the new and spent fuel storage racks.
		4. The RM and FHM/spent fuel handling tool (SFHT) gripper assemblies are designed to prevent opening while the weight of the fuel assembly is suspended from the grippers.	The RM and FHM/SFHT gripper assemblies will be tested by operating the open controls of the gripper while suspending a dummy fuel assembly.	The RM and FHM/SFHT gripper assemblies will not open while suspending a dummy test assembly.
		5. The lift height of the RM mast and FHM hoist(s) is limited such that the minimum required depth of water shielding is maintained.	The RM and FHM will be tested by attempting to raise a dummy fuel assembly.	The bottom of the dummy fuel assembly cannot be raised to within 24 ft, 6 in. of the operating deck floor.
		6. The RM and FHM are designed to maintain their load carrying and structural integrity functions during a safe shutdown earthquake.	i) Inspection will be performed to verify that the RM and FHM are located on the nuclear island.	i) The RM and FHM are located on the nuclear island.
		7. The new and spent fuel storage racks maintain the effective neutron multiplication factor required by 10 CFR 50.68 limits during normal operation, design basis seismic events, and design basis dropped spent fuel assembly accidents over the spent fuel storage racks.	ii) Inspection will be performed to verify that the new and spent fuel storage racks are located on the nuclear island.	ii) The new and spent fuel storage racks are located on the nuclear island.
5	2.1.01.05	Not used per Amendment No. XX ~~5. The lift height of the RM mast and FHM hoist(s) is limited such that the minimum required depth of water shielding is maintained.~~	~~The RM and FHM will be tested by attempting to raise a dummy fuel assembly.~~	~~The bottom of the dummy fuel assembly cannot be raised to within 24 ft, 6 in. of the operating deck floor.~~

Table 2.1.1-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
6	2.1.01.06.i	Not used per Amendment No. XX ~~6. The RM and FHM are designed to maintain their load carrying and structural integrity functions during a safe shutdown earthquake.~~	~~i) Inspection will be performed to verify that the RM and FHM are located on the nuclear island.~~	~~i) The RM and FHM are located on the nuclear island.~~
***				
9	2.1.01.07.ii	Not used per Amendment No. XX ~~7. The new and spent fuel storage racks maintain the effective neutron multiplication factor required by 10 CFR 50.68 limits during normal operation, design basis seismic events, and design basis dropped spent fuel assembly accidents over the spent fuel storage racks.~~	~~ii) Inspection will be performed to verify that the new and spent fuel storage racks are located on the nuclear island.~~	~~ii) The new and spent fuel storage racks are located on the nuclear island.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.1.2, Reactor Coolant System, Table 2.1.2-4 as shown below:

*Note: Changes to ITAAC Nos. 2.1.02.08d.vi, 2.1.02.08d.vii and 2.1.2.02.08d.viii in Table 2.1.2-4, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 8d.vi, 8d.vii and 8d.viii in Table 2.1.2-4 is needed.

Table 2.1.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
36	2.1.02.08d.v	8.d) The RCS provides automatic depressurization during design basis events.	v) Inspections of the elevation of the ADS stage 4 valve discharge will be conducted. vi) Inspections of the ADS stage 4 valve discharge will be conducted. vii) Inspection of each ADS sparger will be conducted to determine the flow area through the sparger holes. viii) Inspection of the elevation of each ADS sparger will be conducted.	v) The minimum elevation of the bottom inside surface of the outlet of these valves is greater than plant elevation 110 feet. vi) The discharge of the ADS stage 4 valves is directed into the steam generator compartments. vii) The flow area through the holes in each ADS sparger is ≥ 274 in^2. viii) The centerline of the connection of the sparger arms to the sparger hub is ≤ 11.5 feet below the IRWST overflow level.
37	2.1.02.08d.vi	Not used per Amendment No. XX ~~8.d) The RCS provides automatic depressurization during design basis events.~~	~~vi) Inspections of the ADS stage 4 valve discharge will be conducted.~~	~~vi) The discharge of the ADS stage 4 valves is directed into the steam generator compartments.~~
38	2.1.02.08d.vii	Not used per Amendment No. XX ~~8.d) The RCS provides automatic depressurization during design basis events.~~	~~vii) Inspection of each ADS sparger will be conducted to determine the flow area through the sparger holes.~~	~~vii) The flow area through the holes in each ADS sparger is ≥ 274 in².~~
39	2.1.02.08d.viii	Not used per Amendment No. XX ~~8.d) The RCS provides automatic depressurization during design basis events.~~	~~viii) Inspection of the elevation of each ADS sparger will be conducted.~~	~~viii) The centerline of the connection of the sparger arms to the sparger hub is ≤ 11.5 feet below the IRWST overflow level.~~
***				

Table 2.1.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
45	2.1.02.10	Not used per Amendment No. XX ~~10. Safety-related displays identified in Table 2.1.2-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.1.2-1 can be retrieved in the MCR.~~
***				
47	2.1.02.11a.ii	10. Safety-related displays identified in Table 2.1.2-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.1.2-1 can be retrieved in the MCR.
		11.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.1.2-1 to perform active functions.	ii) Stroke testing will be performed on the other remotely operated valves listed in Table 2.1.2-1 using controls in the MCR.	ii) Controls in the MCR operate to cause the remotely operated valves (other than squib valves) to perform active functions.
		11.b) The valves identified in Table 2.1.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	ii) Testing will be performed on the other remotely operated valves identified in Table 2.1.2-1 using real or simulated signals into the PMS.	ii) The other remotely operated valves identified in Table 2.1.2-1 as having PMS control perform the active function identified in the table after receiving a signal from PMS.
		11.b) The valves identified in Table 2.1.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	iii) Testing will be performed to demonstrate that remotely operated RCS valves RCS-V001A/B, V002A/B, V003A/B, V011A/B, V012A/B, V013A/B open within the required response times.	iii) These valves open within the following times after receipt of an actuation signal: V001A/B < 40 sec V002A/B, V003A/B < 100 sec V011A/B < 30 sec V012A/B, V013A/B < 60 sec
		12.b) After loss of motive power, the remotely operated valves identified in Table 2.1.2-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	Upon loss of motive power, each remotely operated valve identified in Table 2.1.2-1 assumes the indicated loss of motive power position.

Table 2.1.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
49	2.1.02.11b.ii	Not used per Amendment No. XX ~~11.b) The valves identified in Table 2.1.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.~~	~~ii) Testing will be performed on the other remotely operated valves identified in Table 2.1.2-1 using real or simulated signals into the PMS.~~	~~ii) The other remotely operated valves identified in Table 2.1.2-1 as having PMS control perform the active function identified in the table after receiving a signal from PMS.~~
50	2.1.02.11b.iii	Not used per Amendment No. XX ~~11.b) The valves identified in Table 2.1.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.~~	~~iii) Testing will be performed to demonstrate that remotely operated RCS valves RCS-V001A/B, V002A/B, V003A/B, V011A/B, V012A/B, V013A/B open within the required response times.~~	~~iii) These valves open within the following times after receipt of an actuation signal:~~ ~~V001A/B ≤ 40 sec~~ ~~V002A/B, V003A/B ≤ 100 sec~~ ~~V011A/B ≤ 30 sec~~ ~~V012A/B, V013A/B ≤ 60 sec~~
***				
62	2.1.02.12b	Not used per Amendment No. XX ~~12.b) After loss of motive power, the remotely operated valves identified in Table 2.1.2-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~Upon loss of motive power, each remotely operated valve identified in Table 2.1.2-1 assumes the indicated loss of motive power position.~~
***				

Revise COL Appendix C (and plant-specific Tier 1) Section 2.1.3, Reactor System, Table 2.1.3-2 as shown below:

Table 2.1.3-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
69	2.1.03.02a	2.a) The reactor upper internals rod guide arrangement is as shown in Figure 2.1.3-1. 2.b) The control assemblies (rod cluster and gray rod) and drive rod arrangement is as shown in Figure 2.1.3-2. 2.c) The reactor vessel arrangement is as shown in Figure 2.1.3-3.	Inspection of the as-built system will be performed. Inspection of the as-built system will be performed. Inspection of the as-built system will be performed.	The as-built RXS will accommodate the fuel assembly and control rod drive mechanism pattern shown in Figure 2.1.3-1. The as-built RXS will accommodate the control assemblies (rod cluster and gray rod) and drive rod arrangement shown in Figure 2.1.3-2. The as-built RXS will accommodate the reactor vessel arrangement shown in Figure 2.1.3-3.
70	2.1.03.02b	Not used per Amendment No. XX ~~2.b) The control assemblies (rod cluster and gray rod) and drive rod arrangement is as shown in Figure 2.1.3-2.~~	~~Inspection of the as-built system will be performed.~~	~~The as-built RXS will accommodate the control assemblies (rod cluster and gray rod) and drive rod arrangement shown in Figure 2.1.3-2.~~
71	2.1.03.02c	Not used per Amendment No. XX ~~2.c) The reactor vessel arrangement is as shown in Figure 2.1.3-3.~~	~~Inspection of the as-built system will be performed.~~	~~The as-built RXS will accommodate the reactor vessel arrangement shown in Figure 2.1.3-3.~~
***				

Table 2.1.3-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
78	2.1.03.07.i	7. The reactor internals will withstand the effects of flow induced vibration.	i) A vibration type test will be conducted on the (first unit) reactor internals representative of AP1000.	i) A report exists and concludes that the (first unit) reactor internals have no observable damage or loose parts as a result of the vibration type test.
			ii) A pre-test inspection, a flow test and a post-test inspection will be conducted on the as-built reactor internals.	ii) The as-built reactor internals have no observable damage or loose parts.
		10. The reactor lower internals assembly is equipped with holders for at least eight capsules for storing material surveillance specimens.	Inspection of the reactor lower internals assembly for the presence of capsules will be performed.	At least eight capsules are in the reactor lower internals assembly.
79	2.1.03.07.ii	Not used per Amendment No. XX ~~7. The reactor internals will withstand the effects of flow induced vibration.~~	~~ii) A pre-test inspection, a flow test and a post-test inspection will be conducted on the as-built reactor internals.~~	~~ii) The as-built reactor internals have no observable damage or loose parts.~~
***				
85	2.1.03.10	Not used per Amendment No. XX ~~10. The reactor lower internals assembly is equipped with holders for at least eight capsules for storing material surveillance specimens.~~	~~Inspection of the reactor lower internals assembly for the presence of capsules will be performed.~~	~~At least eight capsules are in the reactor lower internals assembly.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.2.1, Containment System, Table 2.2.1-3 as shown below:

Table 2.2.1-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
110	2.2.01.09	9. Safety-related displays identified in Table 2.2.1-1 can be retrieved in the MCR. 10.a) Controls exist in the MCR to cause those remotely operated valves identified in Table 2.2.1-1 to perform active functions. 10.b) The valves identified in Table 2.2.1-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	Inspection will be performed for retrievability of the safety-related displays in the MCR. Stroke testing will be performed on remotely operated valves identified in Table 2.2.1-1 using the controls in the MCR. Testing will be performed on remotely operated valves listed in Table 2.2.1-1 using real or simulated signals into the PMS.	Safety-related displays identified in Table 2.2.1-1 can be retrieved in the MCR. Controls in the MCR operate to cause remotely operated valves identified in Table 2.2.1-1 to perform active safety functions. The remotely operated valves identified in Table 2.2.1-1 as having PMS control perform the active function identified in the table after receiving a signal from PMS.
111	2.2.01.10a	Not used per Amendment No. XX ~~10.a) Controls exist in the MCR to cause those remotely operated valves identified in Table 2.2.1-1 to perform active functions.~~	~~Stroke testing will be performed on remotely operated valves identified in Table 2.2.1-1 using the controls in the MCR.~~	~~Controls in the MCR operate to cause remotely operated valves identified in Table 2.2.1-1 to perform active safety functions.~~
112	2.2.01.10b	Not used per Amendment No. XX ~~10.b) The valves identified in Table 2.2.1-1 as having PMS control perform an active safety function after receiving a signal from the PMS.~~	~~Testing will be performed on remotely operated valves listed in Table 2.2.1-1 using real or simulated signals into the PMS.~~	~~The remotely operated valves identified in Table 2.2.1-1 as having PMS control perform the active function identified in the table after receiving a signal from PMS.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.2.2, Passive Containment Cooling System, Table 2.2.2-3 as shown below:

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
135	2.2.02.07a.i	Not used per Amendment No. XX ~~7.a) The PCS delivers water from the PCCWST to the outside, top of the containment vessel.~~	~~i) Testing will be performed to measure the PCCWST delivery rate from each one of the three parallel flow paths.~~	~~i) When tested, each one of the three flow paths delivers water at greater than or equal to:~~ ~~– 469.1 gpm at a PCCWST water level of 27.4 ft + 0.2, – 0.0 ft above the tank floor~~ ~~– 226.6 gpm when the PCCWST water level uncovers the first (i.e. tallest) standpipe~~ ~~– 176.3 gpm when the PCCWST water level uncovers the second tallest standpipe~~ ~~– 144.2 gpm when the PCCWST water level uncovers the third tallest standpipe~~
136	2.2.02.07a.ii	Not used per Amendment No. XX ~~7.a) The PCS delivers water from the PCCWST to the outside, top of the containment vessel.~~	~~ii) Testing and or analysis will be performed to demonstrate the PCCWST inventory provides 72 hours of adequate water flow.~~	~~ii) When tested and/or analyzed with all flow paths delivering and an initial water level at 27.4 + 0.2, – 0.00 ft, the PCCWST water inventory provides greater than or equal to 72 hours of flow, and the flow rate at 72 hours is greater than or equal to 100.7 gpm.~~

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
137	2.2.02.07a.iii	7.a) The PCS delivers water from the PCCWST to the outside, top of the containment vessel.	iii) Inspection will be performed to determine the PCCWST standpipes elevations.	iii) The elevations of the standpipes above the tank floor are: – 16.8 ft ± 0.2 ft – 20.3 ft ± 0.2 ft – 24.1 ft ± 0.2 ft
		7.f) The PCS provides a flow path for long-term water makeup from the PCCWST to the spent fuel pool.	ii) Inspection of the PCCWST will be performed.	ii) The volume of the PCCWST is greater than 756,700 gallons.
		8.a) The PCCAWST contains an inventory of cooling water sufficient for PCS containment cooling from hour 72 through day 7.	Inspection of the PCCAWST will be performed.	The volume of the PCCAWST is greater than 780,000 gallons.
138	2.2.02.07b.i	7.a) The PCS delivers water from the PCCWST to the outside, top of the containment vessel.	i) Testing will be performed to measure the PCCWST delivery rate from each one of the three parallel flow paths.	i) When tested, each one of the three flow paths delivers water at greater than or equal to: – 469.1 gpm at a PCCWST water level of 27.4 ft + 0.2, - 0.0 ft above the tank floor – 226.6 gpm when the PCCWST water level uncovers the first (i.e. tallest) standpipe – 176.3 gpm when the PCCWST water level uncovers the second tallest standpipe – 144.2 gpm when the PCCWST water level uncovers the third tallest standpipe
			ii) Testing and or analysis will be performed to demonstrate the PCCWST inventory provides 72 hours of adequate water flow.	ii) When tested and/or analyzed with all flow paths delivering and an initial water level at 27.4 + 0.2, - 0.00 ft, the PCCWST water inventory provides greater than or equal to 72 hours of flow, and the flow rate at 72 hours is greater than or equal to 100.7 gpm.

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		7.b) The PCS wets the outside surface of the containment vessel. The inside and the outside of the containment vessel above the operating deck are coated with an inorganic zinc material.	i) Testing will be performed to measure the outside wetted surface of the containment vessel with one of the three parallel flow paths delivering water to the top of the containment vessel.	i) A report exists and concludes that when the water in the PCCWST uncovers the standpipes at the following levels, the water delivered by one of the three parallel flow paths to the containment shell provides coverage measured at the spring line that is equal to or greater than the stated coverages. - 24.1 ± 0.2 ft above the tank floor; at least 90% of the perimeter is wetted. - 20.3 ± 0.2 ft above the tank floor; at least 72.9% of the perimeter is wetted. - 16.8 ± 0.2 ft above the tank floor; at least 59.6% of the perimeter is wetted.
			ii) Inspection of the containment vessel exterior coating will be conducted.	ii) A report exists and concludes that the containment vessel exterior surface is coated with an inorganic zinc coating above elevation 135'-3".
			iii) Inspection of the containment vessel interior coating will be conducted.	iii) A report exists and concludes that the containment vessel interior surface is coated with an inorganic zinc coating above 7' above the operating deck.
		7.c) The PCS provides air flow over the outside of the containment vessel by a natural circulation air flow path from the air inlets to the air discharge structure.	Inspections of the air flow path segments will be performed.	Flow paths exist at each of the following locations: – Air inlets – Base of the outer annulus – Base of the inner annulus – Discharge structure
		7.d) The PCS drains the excess water from the outside of the containment vessel through the two upper annulus drains.	Testing will be performed to verify the upper annulus drain flow performance.	With a water level within the upper annulus 10" + 1" above the annulus drain inlet, the flow rate through each drain is greater than or equal to 525 gpm.

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		7.e) The PCS provides a flow path for long-term water makeup to the PCCWST. 9. Safety-related displays identified in Table 2.2.2-1 can be retrieved in the MCR. 10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.2.2-1 to perform active functions. 10.b) The valves identified in Table 2.2.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS. 11.a) The motor-operated valves identified in Table 2.2.2-1 perform an active safety-related function to change position as indicated in the table. 11.b) After loss of motive power, the remotely operated valves identified in Table 2.2.2-1 assume the indicated loss of motive power position.	ii) Testing will be performed to measure the delivery rate from the long-term makeup connection to the PCCWST. Inspection will be performed for retrievability of the safety-related displays in the MCR. Stroke testing will be performed on the remotely operated valves identified in Table 2.2.2-1 using the controls in the MCR. Testing will be performed on the remotely operated valves in Table 2.2.2-1 using real or simulated signals into the PMS. iii) Tests of the motor-operated valves will be performed under preoperational flow, differential pressure, and temperature conditions. Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	ii) With a water supply connected to the PCS long-term makeup connection, each PCS recirculation pump delivers greater than or equal to 100 gpm when tested separately. Safety-related displays identified in Table 2.2.2-1 can be retrieved in the MCR. Controls in the MCR operate to cause remotely operated valves identified in Table 2.2.2-1 to perform active functions. The remotely operated valves identified in Table 2.2.2-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS. iii) Each motor-operated valve changes position as indicated in Table 2.2.2-1 under preoperational test conditions. After loss of motive power, each remotely operated valve identified in Table 2.2.2-1 assumes the indicated loss of motive power position.
139	2.2.02.07b.ii	Not used per Amendment No. XX ~~7.b) The PCS wets the outside surface of the containment vessel. The inside and the outside of the containment vessel above the operating deck are coated with an inorganic zinc material.~~	~~ii) Inspection of the containment vessel exterior coating will be conducted.~~	~~ii) A report exists and concludes that the containment vessel exterior surface is coated with an inorganic zinc coating above elevation 135'-3".~~
140	2.2.02.07b.iii	Not used per Amendment No. XX ~~7.b) The PCS wets the outside surface of the containment vessel. The inside and the outside of the containment vessel above the operating deck are coated with an inorganic zinc material.~~	~~iii) Inspection of the containment vessel interior coating will be conducted.~~	~~iii) A report exists and concludes that the containment vessel interior surface is coated with an inorganic zinc coating above 7' above the operating deck.~~

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
141	2.2.02.07c	Not used per Amendment No. XX ~~7.c) The PCS provides air flow over the outside of the containment vessel by a natural circulation air flow path from the air inlets to the air discharge structure.~~	~~Inspections of the air flow path segments will be performed.~~	~~Flow paths exist at each of the following locations:~~ ~~– Air inlets~~ ~~– Base of the outer annulus~~ ~~– Base of the inner annulus~~ ~~– Discharge structure~~
142	2.2.02.07d	Not used per Amendment No. XX ~~7.d) The PCS drains the excess water from the outside of the containment vessel through the two upper annulus drains.~~	~~Testing will be performed to verify the upper annulus drain flow performance.~~	~~With a water level within the upper annulus 10" ± 1" above the annulus drain inlet, the flow rate through each drain is greater than or equal to 525 gpm.~~
***				
144	2.2.02.07e.ii	Not used per Amendment No. XX ~~7.e) The PCS provides a flow path for long-term water makeup to the PCCWST.~~	~~ii) Testing will be performed to measure the delivery rate from the long-term makeup connection to the PCCWST.~~	~~ii) With a water supply connected to the PCS long-term makeup connection, each PCS recirculation pump delivers greater than or equal to 100 gpm when tested separately.~~
145	2.2.02.07f.i	7.f) The PCS provides a flow path for long-term water makeup from the PCCWST to the spent fuel pool. 8.b) The PCS delivers water from the PCCAWST to the PCCWST and spent fuel pool simultaneously.	i) Testing will be performed to measure the delivery rate from the PCCWST to the spent fuel pool. Testing will be performed to measure the delivery rate from the PCCAWST to the PCCWST and spent fuel pool simultaneously.	i) With the PCCWST water level at 27.4 ft + 0.2, - 0.0 ft above the bottom of the tank, the flow path from the PCCWST to the spent fuel pool delivers greater than or equal to 118 gpm. With PCCAWST aligned to the suction of the recirculation pumps, each pump delivers greater than or equal to 100 gpm to the PCCWST and 35 gpm to the spent fuel pool simultaneously when each pump is tested separately.
146	2.2.02.07f.ii	Not used per Amendment No. XX ~~7.f) The PCS provides a flow path for long-term water makeup from the PCCWST to the spent fuel pool.~~	~~ii) Inspection of the PCCWST will be performed.~~	~~ii) The volume of the PCCWST is greater than 756,700 gallons.~~

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
147	2.2.02.08a	Not used per Amendment No. XX ~~8.a) The PCCAWST contains an inventory of cooling water sufficient for PCS containment cooling from hour 72 through day 7.~~	~~Inspection of the PCCAWST will be performed.~~	~~The volume of the PCCAWST is greater than 780,000 gallons.~~
148	2.2.02.08b	Not used per Amendment No. XX ~~8.b) The PCS delivers water from the PCCAWST to the PCCWST and spent fuel pool simultaneously.~~	~~Testing will be performed to measure the delivery rate from the PCCAWST to the PCCWST and spent fuel pool simultaneously.~~	~~With PCCASWST aligned to the suction of the recirculation pumps, each pump delivers greater than or equal to 100 gpm to the PCCWST and 35 gpm to the spent fuel pool simultaneously when each pump is tested separately.~~
***				
150	2.2.02.09	Not used per Amendment No. XX ~~9. Safety-related displays identified in Table 2.2.2-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.2.2-1 can be retrieved in the MCR.~~
151	2.2.02.10a	Not used per Amendment No. XX ~~10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.2.2-1 to perform active functions.~~	~~Stroke testing will be performed on the remotely operated valves identified in Table 2.2.2-1 using the controls in the MCR.~~	~~Controls in the MCR operate to cause remotely operated valves identified in Table 2.2.2-1 to perform active functions.~~
152	2.2.02.10b	Not used per Amendment No. XX ~~10.b) The valves identified in Table 2.2.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.~~	~~Testing will be performed on the remotely operated valves in Table 2.2.2-1 using real or simulated signals into the PMS.~~	~~The remotely operated valves identified in Table 2.2.2-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.~~
***				
156	2.2.02.11a.iii	Not used per Amendment No. XX ~~11.a) The motor-operated valves identified in Table 2.2.2-1 perform an active safety-related function to change position as indicated in the table.~~	~~iii) Tests of the motor-operated valves will be performed under preoperational flow, differential pressure, and temperature conditions.~~	~~iii) Each motor-operated valve changes position as indicated in Table 2.2.2-1 under preoperational test conditions.~~
157	2.2.02.11b	Not used per Amendment No. XX ~~11.b) After loss of motive power, the remotely operated valves identified in Table 2.2.2-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~After loss of motive power, each remotely operated valve identified in Table 2.2.2-1 assumes the indicated loss of motive power position.~~

Revise COL Appendix C (and plant-specific Tier 1) Section 2.2.3, Passive Core Cooling System, Table 2.2.3-4 as shown below:

Table 2.2.3-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
183	2.2.03.08c.iv.01	8.c) The PXS provides RCS makeup, boration, and safety injection during design basis events.	iv) Inspections of the elevation of the following pipe lines will be conducted: 1. IRWST injection lines; IRWST connection to DVI nozzles v) Inspections of the elevation of the following tanks will be conducted: 2. IRWST	iv) The maximum elevation of the top inside surface of these lines is less than the elevation of: 1. IRWST bottom inside surface v) The elevation of the bottom inside tank surface is higher than the direct vessel injection nozzle centerline by the following: 2. IRWST ≥ 3.4 ft
***				
188	2.2.03.08c.v.02	Not used per Amendment No. XX ~~8.c) The PXS provides RCS makeup, boration, and safety injection during design basis events.~~	~~v) Inspections of the elevation of the following tanks will be conducted:~~ ~~2. IRWST~~	~~v) The elevation of the bottom inside tank surface is higher than the direct vessel injection nozzle centerline by the following:~~ ~~2. IRWST ≥ 3.4 ft~~
***				

Table 2.2.3-4 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
206	2.2.03.10	10. Safety-related displays of the parameters identified in Table 2.2.3-1 can be retrieved in the MCR.	Inspection will be performed for the retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.2.3-1 can be retrieved in the MCR.
		11.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.2.3-1 to perform their active function(s).	ii) Stroke testing will be performed on remotely operated valves other than squib valves identified in Table 2.2.3-1 using the controls in the MCR.	ii) Controls in the MCR operate to cause remotely operated valves other than squib valves to perform their active functions.
		11.b) The valves identified in Table 2.2.3-1 as having PMS control perform their active function after receiving a signal from the PMS.	ii) Testing will be performed on the remotely operated valves other than squib valves identified in Table 2.2.3-1 using real or simulated signals into the PMS.	ii) Remotely operated valves other than squib valves perform the active function identified in the table after a signal is input to the PMS.
			iii) Testing will be performed to demonstrate that remotely operated PXS isolation valves PXS-V014A/B, V015A/B, V108A/B open within the required response times.	iii) These valves open within 20 seconds after receipt of an actuation signal.
		12.b) After loss of motive power, the remotely operated valves identified in Table 2.2.3-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	After loss of motive power, each remotely operated valve identified in Table 2.2.3-1 assumes the indicated loss of motive power position.
		13. Displays of the parameters identified in Table 2.2.3-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.2.3-3 in the MCR.	Displays identified in Table 2.2.3-3 can be retrieved in the MCR.
***				
208	2.2.03.11a.ii	Not used per Amendment No. XX ~~11.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.2.3-1 to perform their active function(s).~~	~~ii) Stroke testing will be performed on remotely operated valves other than squib valves identified in Table 2.2.3-1 using the controls in the MCR.~~	~~ii) Controls in the MCR operate to cause remotely operated valves other than squib valves to perform their active functions.~~

Table 2.2.3-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
210	2.2.03.11b.ii	Not used per Amendment No. XX ~~11.b) The valves identified in Table 2.2.3-1 as having PMS control perform their active function after receiving a signal from the PMS.~~	~~ii) Testing will be performed on the remotely operated valves other than squib valves identified in Table 2.2.3-1 using real or simulated signals into the PMS.~~	~~ii) Remotely operated valves other than squib valves perform the active function identified in the table after a signal is input to the PMS.~~
211	2.2.03.11b.iii	Not used per Amendment No. XX ~~11.b) The valves identified in Table 2.2.3-1 as having PMS control perform their active function after receiving a signal from the PMS.~~	~~iii) Testing will be performed to demonstrate that remotely operated PXS isolation valves PXS-V014A/B, V015A/B, V108A/B open within the required response times.~~	~~iii) These valves open within 20 seconds after receipt of an actuation signal.~~
***				
217	2.2.03.12b	Not used per Amendment No. XX ~~12.b) After loss of motive power, the remotely operated valves identified in Table 2.2.3-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~After loss of motive power, each remotely operated valve identified in Table 2.2.3-1 assumes the indicated loss of motive power position.~~
218	2.2.03.13	Not used per Amendment No. XX ~~13. Displays of the parameters identified in Table 2.2.3-3 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the displays identified in Table 2.2.3-3 in the MCR.~~	~~Displays identified in Table 2.2.3-3 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.2.4, Steam Generator System, Table 2.2.4-4 as shown below:

Table 2.2.4-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
240	2.2.04.09a.i	Not used per Amendment No. XX ~~9.a) Components within the main steam system, main and startup feedwater system, and the main turbine system identified in Table 2.2.4-3 provide backup isolation of the SGS to limit steam generator blowdown and feedwater flow to the steam generator.~~	~~i) Testing will be performed to confirm closure of the valves identified in Table 2.2.4-3.~~	~~i) The valves identified in Table 2.2.4-3 close after a signal is generated by the PMS.~~
***				
244	2.2.04.10	Not used per Amendment No. XX ~~10. Safety-related displays identified in Table 2.2.4-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.2.4-1 can be retrieved in the MCR.~~
245	2.2.04.11a	Not used per Amendment No. XX ~~11.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.2.4-1 to perform active functions.~~	~~Stroke testing will be performed on the remotely operated valves listed in Table 2.2.4-1 using controls in the MCR.~~	~~Controls in the MCR operate to cause the remotely operated valves to perform active safety functions.~~
246	2.2.04.11b.i	Not used per Amendment No. XX ~~11.b) The valves identified in Table 2.2.4-1 as having PMS control perform an active safety function after receiving a signal from PMS.~~	~~i) Testing will be performed on the remotely operated valves listed in Table 2.2.4-1 using real or simulated signals into the PMS.~~	~~i) The remotely-operated valves identified in Table 2.2.4-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.~~
247	2.2.04.11b.ii	Not used per Amendment No. XX ~~11.b) The valves identified in Table 2.2.4-1 as having PMS control perform an active safety function after receiving a signal from PMS.~~	~~ii) Testing will be performed to demonstrate that remotely operated SGS isolation valves SGS-V027A/B, V040A/B, V057A/B, V250A/B close within the required response times.~~	~~ii) These valves close within the following times after receipt of an actuation signal:~~ ~~V027A/B < 44 sec~~ ~~V040A/B, V057A/B < 5 sec~~ ~~V250A/B < 5 sec~~
***				

Table 2.2.4-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
250	2.2.04.12a.iii	9.a) Components within the main steam system, main and startup feedwater system, and the main turbine system identified in Table 2.2.4-3 provide backup isolation of the SGS to limit steam generator blowdown and feedwater flow to the steam generator.	i) Testing will be performed to confirm closure of the valves identified in Table 2.2.4-3.	i) The valves identified in Table 2.2.4-3 close after a signal is generated by the PMS.
		10. Safety-related displays identified in Table 2.2.4-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.2.4-1 can be retrieved in the MCR.
		11.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.2.4-1 to perform active functions.	Stroke testing will be performed on the remotely operated valves listed in Table 2.2.4-1 using controls in the MCR.	Controls in the MCR operate to cause the remotely operated valves to perform active safety functions.
		11.b) The valves identified in Table 2.2.4-1 as having PMS control perform an active safety function after receiving a signal from PMS.	i) Testing will be performed on the remotely operated valves listed in Table 2.2.4-1 using real or simulated signals into the PMS.	i) The remotely-operated valves identified in Table 2.2.4-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.
			ii) Testing will be performed to demonstrate that remotely operated SGS isolation valves SGS-V027A/B, V040A/B, V057A/B, V250A/B close within the required response times.	ii) These valves close within the following times after receipt of an actuation signal: V027A/B < 44 sec V040A/B, V057A/B < 5 sec V250A/B < 5 sec
		12.a) The motor-operated valves identified in Table 2.2.4-1 perform an active safety-related function to change position as indicated in the table.	iii) Tests of the motor-operated valves will be performed under pre-operational flow, differential pressure, and temperature conditions.	iii) Each motor-operated valve changes position as indicated in Table 2.2.4-1 under pre-operational test conditions.
		12.b) After loss of motive power, the remotely operated valves identified in Table 2.2.4-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	After loss of motive power, each remotely operated valve identified in Table 2.2.4-1 assumes the indicated loss of motive power position. Motive power to SGS-PL-V040A/B and SGS-PL-V057A/B is electric power to the actuator from plant services.

Table 2.2.4-4 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
251	2.2.04.12b	Not used per Amendment No. XX ~~12.b) After loss of motive power, the remotely operated valves identified in Table 2.2.4-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~After loss of motive power, each remotely operated valve identified in Table 2.2.4-1 assumes the indicated loss of motive power position. Motive power to SGS-PL-V040A/B and SGS-PL-V057A/B is electric power to the actuator from plant services.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.2.5, Main Control Room Emergency Habitability System, Table 2.2.5-5 as shown below:

Table 2.2.5-5
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
265	2.2.05.07a.i	7.a) The VES provides a 72-hour supply of breathable quality air for the occupants of the MCR.	i) Testing will be performed to confirm that the required amount of air flow is delivered to the MCR.	i) The air flow rate from the VES is at least 60 scfm and not more than 70 scfm.
			iii) MCR air samples will be taken during VES testing and analyzed for quality.	iii) The MCR air is of breathable quality.
		7.b) The VES maintains the MCR pressure boundary at a positive pressure with respect to the surrounding areas.	i) Testing will be performed with VES flow rate between 60 and 70 scfm to confirm that the MCR is capable of maintaining the required pressurization of the pressure boundary.	i) The MCR pressure boundary is pressurized to greater than or equal to 1/8-in. water gauge with respect to the surrounding area.
			ii) Air leakage into the MCR will be measured during VES testing using a tracer gas.	ii) Air leakage into the MCR is less than or equal to 10 cfm.
		7.d) The system provides a passive recirculation flow of MCR air to maintain main control room dose rates below an acceptable level during VES operation.	Testing will be performed to confirm that the required amount of air flow circulates through the MCR passive filtration system.	The air flow rate at the outlet of the MCR passive filtration system is at least 600 cfm greater than the flow measured by VES-003A/B.
		8. Safety-related displays identified in Table 2.2.5-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.2.5-1 can be retrieved in the MCR.
		9.a) Controls exist in the MCR to cause remotely operated valves identified in Table 2.2.5-1 to perform their active functions.	Stroke testing will be performed on remotely operated valves identified in Table 2.2.5-1 using the controls in the MCR.	Controls in the MCR operate to cause remotely operated valves identified in Table 2.2.5-1 to perform their active safety functions.
		9.b) The valves identified in Table 2.2.5-1 as having PMS control perform their active safety function after receiving a signal from the PMS.	Testing will be performed on remotely operated valves listed in Table 2.2.5-1 using real or simulated signals into the PMS.	The remotely operated valves identified in Table 2.2.5-1 as having PMS control perform the active safety function identified in the table after receiving a signal from the PMS.

Table 2.2.5-5
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		10. After loss of motive power, the remotely operated valves identified in Table 2.2.5-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	After loss of motive power, each remotely operated valve identified in Table 2.2.5-1 assumes the indicated loss of motive power position.
		11. Displays of the parameters identified in Table 2.2.5-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.2.5-3 can be retrieved in the MCR.
		12. The background noise level in the MCR does not exceed 65 dB(A) at the operator workstations when VES is operating.	The as-built VES will be operated, and background noise levels in the MCR will be measured at the operator work stations with the plant not operating.	The background noise level in the MCR does not exceed 65 dB(A) at the operator work stations when the VES is operating.
***				
267	2.2.05.07a.iii	Not used per Amendment No. XX ~~7.a) The VES provides a 72-hour supply of breathable quality air for the occupants of the MCR.~~	~~iii) MCR air samples will be taken during VES testing and analyzed for quality.~~	~~iii) The MCR air is of breathable quality.~~
268	2.2.05.07b.i	Not used per Amendment No. XX ~~7.b) The VES maintains the MCR pressure boundary at a positive pressure with respect to the surrounding areas.~~	~~i) Testing will be performed with VES flow rate between 60 and 70 scfm to confirm that the MCR is capable of maintaining the required pressurization of the pressure boundary.~~	~~i) The MCR pressure boundary is pressurized to greater than or equal to 1/8-in. water gauge with respect to the surrounding area.~~
269	2.2.05.07b.ii	Not used per Amendment No. XX ~~7.b) The VES maintains the MCR pressure boundary at a positive pressure with respect to the surrounding areas.~~	~~ii) Air leakage into the MCR will be measured during VES testing using a tracer gas.~~	~~ii) Air leakage into the MCR is less than or equal to 10 cfm.~~
***				
271	2.2.05.07d	Not used per Amendment No. XX ~~7.d) The system provides a passive recirculation flow of MCR air to maintain main control room dose rates below an acceptable level during VES operation.~~	~~Testing will be performed to confirm that the required amount of air flow circulates through the MCR passive filtration system.~~	~~The air flow rate at the outlet of the MCR passive filtration system is at least 600 cfm greater than the flow measured by VES-003A/B.~~

Table 2.2.5-5
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
272	2.2.05.08	Not used per Amendment No. XX ~~8. Safety-related displays identified in Table 2.2.5-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.2.5-1 can be retrieved in the MCR.~~
273	2.2.05.09a	Not used per Amendment No. XX ~~9.a) Controls exist in the MCR to cause remotely operated valves identified in Table 2.2.5-1 to perform their active functions.~~	~~Stroke testing will be performed on remotely operated valves identified in Table 2.2.5-1 using the controls in the MCR.~~	~~Controls in the MCR operate to cause remotely operated valves identified in Table 2.2.5-1 to perform their active safety functions.~~
274	2.2.05.09b	Not used per Amendment No. XX ~~9.b) The valves identified in Table 2.2.5-1 as having PMS control perform their active safety function after receiving a signal from the PMS.~~	~~Testing will be performed on remotely operated valves listed in Table 2.2.5-1 using real or simulated signals into the PMS.~~	~~The remotely operated valves identified in Table 2.2.5-1 as having PMS control perform the active safety function identified in the table after receiving a signal from the PMS.~~
275	2.2.05.10	Not used per Amendment No. XX ~~10. After loss of motive power, the remotely operated valves identified in Table 2.2.5-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~After loss of motive power, each remotely operated valve identified in Table 2.2.5-1 assumes the indicated loss of motive power position.~~
276	2.2.05.11	Not used per Amendment No. XX ~~11. Displays of the parameters identified in Table 2.2.5-3 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the parameters in the MCR.~~	~~The displays identified in Table 2.2.5-3 can be retrieved in the MCR.~~
277	2.2.05.12	Not used per Amendment No. XX ~~12. The background noise level in the MCR does not exceed 65 dB(A) at the operator workstations when VES is operating.~~	~~The as-built VES will be operated, and background noise levels in the MCR will be measured at the operator work stations with the plant not operating.~~	~~The background noise level in the MCR does not exceed 65 dB(A) at the operator work stations when the VES is operating.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.1, Component Cooling Water System, Table 2.3.1-2 as shown below:

Table 2.3.1-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
281	2.3.01.03.ii	3. The CCS provides the nonsafety-related functions of transferring heat from the RNS during shutdown and the spent fuel pool cooling system during all modes of operation to the SWS. 4. Controls exist in the MCR to cause the pumps identified in Table 2.3.1-1 to perform the listed functions. 5. Displays of the parameters identified in Table 2.3.1-1 can be retrieved in the MCR.	ii) Testing will be performed to confirm that the CCS can provide cooling water to the RNS HXs while providing cooling water to the SFS HXs. Testing will be performed to actuate the pumps identified in Table 2.3.1-1 using controls in the MCR. Inspection will be performed for retrievability of the parameters in the MCR.	ii) Each pump of the CCS can provide at least 2685 gpm of cooling water to one RNS HX and at least 1200 gpm of cooling water to one SFS HX while providing at least 4415 gpm to other users of cooling water. Controls in the MCR operate to cause pumps listed in Table 2.3.1-1 to perform the listed functions. Displays identified in Table 2.3.1-1 can be retrieved in the MCR.
282	2.3.01.04	Not used per Amendment No. XX ~~4. Controls exist in the MCR to cause the pumps identified in Table 2.3.1-1 to perform the listed functions.~~	~~Testing will be performed to actuate the pumps identified in Table 2.3.1-1 using controls in the MCR.~~	~~Controls in the MCR operate to cause pumps listed in Table 2.3.1-1 to perform the listed functions.~~
283	2.3.01.05	Not used per Amendment No. XX ~~5. Displays of the parameters identified in Table 2.3.1-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the parameters in the MCR.~~	~~Displays identified in Table 2.3.1-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.2, Chemical and Volume Control System, Table 2.3.2-4 as shown below:

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
301	2.3.02.08a.i	8.a) The CVS provides makeup water to the RCS.	i) Testing will be performed by aligning a flow path from each CVS makeup pump, actuating makeup flow to the RCS at pressure greater than or equal to 2000 psia, and measuring the flow rate in the makeup pump discharge line with each pump suction aligned to the boric acid storage tank.	i) Each CVS makeup pump provides a flow rate of greater than or equal to 100 gpm.
		8.b) The CVS provides the pressurizer auxiliary spray.	Testing will be performed by aligning a flow path from each CVS makeup pump to the pressurizer auxiliary spray and measuring the flow rate in the makeup pump discharge line with each pump suction aligned to the boric acid storage tank and with RCS pressure greater than or equal to 2000 psia.	Each CVS makeup pump provides spray flow to the pressurizer.
		9. Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.
		10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.	Stroke testing will be performed on the remotely operated valves identified in Table 2.3.2-1 using the controls in the MCR.	Controls in the MCR operate to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.
		10.b) The valves identified in Table 2.3.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	i) Testing will be performed using real or simulated signals into the PMS.	i) The valves identified in Table 2.3.2-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
			ii) Testing will be performed to demonstrate that the remotely operated CVS isolation valves CVS-V090, V091, V136A/B close within the required response time.	ii) These valves close within the following times after receipt of an actuation signal: V090, V091 < 30 sec V136A/B < 20 sec
		11.a) The motor-operated and check valves identified in Table 2.3.2-1 perform an active safety-related function to change position as indicated in the table.	iii) Tests of the motor-operated valves will be performed under pre-operational flow, differential pressure, and temperature conditions. iv) Exercise testing of the check valves with active safety functions identified in Table 2.3.2-1 will be performed under pre-operational test pressure, temperature and fluid flow conditions.	iii) Each motor-operated valve changes position as indicated in Table 2.3.2-1 under pre-operational test conditions. iv) Each check valve changes position as indicated in Table 2.3.2-1.
		11.b) After loss of motive power, the remotely operated valves identified in Table 2.3.2-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	Upon loss of motive power, each remotely operated valve identified in Table 2.3.2-1 assumes the indicated loss of motive power position.
		12.a) Controls exist in the MCR to cause the pumps identified in Table 2.3.2-3 to perform the listed function.	Testing will be performed to actuate the pumps identified in Table 2.3.2-3 using controls in the MCR.	Controls in the MCR cause pumps identified in Table 2.3.2-3 to perform the listed function.
		12.b) The pumps identified in Table 2.3.2-3 start after receiving a signal from the PLS.	Testing will be performed to confirm starting of the pumps identified in Table 2.3.2-3.	The pumps identified in Table 2.3.2-3 start after a signal is generated by the PLS.
		13. Displays of the parameters identified in Table 2.3.2-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.3.2-3 in the MCR.	Displays identified in Table 2.3.2-3 can be retrieved in the MCR.

***

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
304	2.3.02.08b	Not used per Amendment No. XX ~~8.b) The CVS provides the pressurizer auxiliary spray.~~	~~Testing will be performed by aligning a flow path from each CVS makeup pump to the pressurizer auxiliary spray and measuring the flow rate in the makeup pump discharge line with each pump suction aligned to the boric acid storage tank and with RCS pressure greater than or equal to 2000 psia.~~	~~Each CVS makeup pump provides spray flow to the pressurizer.~~
305	2.3.02.09	Not used per Amendment No. XX ~~9. Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.~~
306	2.3.02.10a	Not used per Amendment No. XX ~~10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.~~	~~Stroke testing will be performed on the remotely operated valves identified in Table 2.3.2-1 using the controls in the MCR.~~	~~Controls in the MCR operate to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.~~
307	2.3.02.10b.i	Not used per Amendment No. XX ~~10.b) The valves identified in Table 2.3.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.~~	~~i) Testing will be performed using real or simulated signals into the PMS.~~	~~i) The valves identified in Table 2.3.2-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.~~
308	2.3.02.10b.ii	Not used per Amendment No. XX ~~10.b) The valves identified in Table 2.3.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.~~	~~ii) Testing will be performed to demonstrate that the remotely operated CVS isolation valves CVS-V090, V091, V136A/B close within the required response time.~~	~~ii) These valves close within the following times after receipt of an actuation signal:~~ ~~V090, V091 ≤ 30 sec~~ ~~V136A/B ≤ 20 sec~~
***				
311	2.3.02.11a.iii	Not used per Amendment No. XX ~~11.a) The motor-operated and check valves identified in Table 2.3.2-1 perform an active safety-related function to change position as indicated in the table.~~	~~iii) Tests of the motor-operated valves will be performed under pre-operational flow, differential pressure, and temperature conditions.~~	~~iii) Each motor-operated valve changes position as indicated in Table 2.3.2-1 under pre-operational test conditions.~~

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
312	2.3.02.11a.iv	Not used per Amendment No. XX ~~11.a) The motor-operated and check valves identified in Table 2.3.2-1 perform an active safety-related function to change position as indicated in the table.~~	~~iv) Exercise testing of the check valves with active safety functions identified in Table 2.3.2-1 will be performed under pre-operational test pressure, temperature and fluid flow conditions.~~	~~iv) Each check valve changes position as indicated in Table 2.3.2-1.~~
313	2.3.02.11b	Not used per Amendment No. XX ~~11.b) After loss of motive power, the remotely operated valves identified in Table 2.3.2-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~Upon loss of motive power, each remotely operated valve identified in Table 2.3.2-1 assumes the indicated loss of motive power position.~~
314	2.3.02.12a	Not used per Amendment No. XX ~~12.a) Controls exist in the MCR to cause the pumps identified in Table 2.3.2-3 to perform the listed function.~~	~~Testing will be performed to actuate the pumps identified in Table 2.3.2-3 using controls in the MCR.~~	~~Controls in the MCR cause pumps identified in Table 2.3.2-3 to perform the listed function.~~
315	2.3.02.12b	Not used per Amendment No. XX ~~12.b) The pumps identified in Table 2.3.2-3 start after receiving a signal from the PLS.~~	~~Testing will be performed to confirm starting of the pumps identified in Table 2.3.2-3.~~	~~The pumps identified in Table 2.3.2-3 start after a signal is generated by the PLS.~~
316	2.3.02.13	Not used per Amendment No. XX ~~13. Displays of the parameters identified in Table 2.3.2-3 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the displays identified in Table 2.3.2-3 in the MCR.~~	~~Displays identified in Table 2.3.2-3 can be retrieved in the MCR.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.3, Standby Diesel Fuel Oil System, Table 2.3.3-2 as shown below:

Table 2.3.3-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
324	2.3.03.04	4. Controls exist in the MCR to cause the components identified in Table 2.3.3-1 to perform the listed function. 5. Displays of the parameters identified in Table 2.3.3-1 can be retrieved in the MCR.	Testing will be performed on the components in Table 2.3.3-1 using controls in the MCR. Inspection will be performed for retrievability of parameters in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.3.3-1 to perform the listed functions. The displays identified in Table 2.3.3-1 can be retrieved in the MCR.
325	2.3.03.05	Not used per Amendment No. XX ~~5. Displays of the parameters identified in Table 2.3.3-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of parameters in the MCR.~~	~~The displays identified in Table 2.3.3-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.4, Fire Protection System, Table 2.3.4-2 as shown below:

Table 2.3.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
330	2.3.04.04.i	4. The FPS provides for manual fire fighting capability in plant areas containing safety-related equipment. 6. The FPS provides nonsafety-related containment spray for severe accident management. 7. The FPS provides two fire water storage tanks, each capable of holding at least 300,000 gallons of water.	i) Inspection of the passive containment cooling system (PCS) storage tank will be performed. Inspection of the containment spray headers will be performed. Inspection of each fire water storage tank will be performed.	i) The volume of the PCS tank above the standpipe feeding the FPS and below the overflow is at least 18,000 gal. The FPS has spray headers and nozzles as follows: At least 44 nozzles at plant elevation of at least 260 feet, and 24 nozzles at plant elevation of at least 275 feet. The volume of each fire water storage tank supplying the FPS is at least 300,000 gallons.
***				
333	2.3.04.06	Not used per Amendment No. XX ~~6. The FPS provides nonsafety-related containment spray for severe accident management.~~	~~Inspection of the containment spray headers will be performed.~~	~~The FPS has spray headers and nozzles as follows:~~ ~~At least 44 nozzles at plant elevation of at least 260 feet, and 24 nozzles at plant elevation of at least 275 feet.~~
334	2.3.04.07	Not used per Amendment No. XX ~~7. The FPS provides two fire water storage tanks, each capable of holding at least 300,000 gallons of water.~~	~~Inspection of each fire water storage tank will be performed.~~	~~The volume of each fire water storage tank supplying the FPS is at least 300,000 gallons.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.5, Mechanical Handling System, Table 2.3.5-2 as shown below:

*Note: Changes to ITAAC Nos. 2.3.05.03a.ii and 2.3.05.03a.iii in Table 2.3.5-2, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 03a.ii and 03a.iii in Table 2.3.5-2 is needed.

Table 2.3.5-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
344	2.3.05.03a.ii	3.a) The polar crane is single failure proof.	ii) Testing of the polar crane is performed. iii) Testing of the polar crane is performed.	ii) The polar crane shall be static-load tested to 125% of the rated load. iii) The polar crane shall lift a test load that is 100% of the rated load. Then it shall lower, stop, and hold the test load.
345	2.3.05.03a.iii	Not used per Amendment No. XX ~~3.a) The polar crane is single failure proof.~~	~~iii) Testing of the polar crane is performed.~~	~~iii) The polar crane shall lift a test load that is 100% of the rated load. Then it shall lower, stop, and hold the test load.~~
***				
347	2.3.05.03b.ii	Not used per Amendment No. XX ~~3.b) The cask handling crane is single failure proof.~~	~~ii) Testing of the cask handling crane is performed.~~	~~ii) The cask handling crane shall be static load tested to 125% of the rated load.~~
348	2.3.05.03b.iii	3.b) The cask handling crane is single failure proof. 4. The cask handling crane cannot move over the spent fuel pool.	ii) Testing of the cask handling crane is performed. iii) Testing of the cask handling crane is performed. Testing of the cask handling crane is performed.	ii) The cask handling crane shall be static load tested to 125% of the rated load. iii) The cask handling crane shall lift a test load that is 100% of the rated load. Then it shall lower, stop, and hold the test load. The cask handling crane does not move over the spent fuel pool.
***				
353	2.3.05.04	Not used per Amendment No. XX ~~4. The cask handling crane cannot move over the spent fuel pool.~~	~~Testing of the cask handling crane is performed.~~	~~The cask handling crane does not move over the spent fuel pool.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.6 Normal Residual Heat Removal System, Table 2.3.6-4 as shown below:

Table 2.3.6-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
375	2.3.06.09b.ii	9.b) The RNS provides heat removal from the reactor coolant during shutdown operations.	ii) Testing will be performed to confirm that the RNS can provide flow through the RNS heat exchangers when the pump suction is aligned to the RCS hot leg and the discharge is aligned to both PXS DVI lines with the RCS at atmospheric pressure.	ii) Each RNS pump provides at least 1400 gpm net flow to the RCS when the hot leg water level is at an elevation 15.5 inches ± 2 inches above the bottom of the hot leg.
			iii) Inspection will be performed of the reactor coolant loop piping.	iii) The RCS cold legs piping centerline is 17.5 inches ± 2 inches above the hot legs piping centerline.
			iv) Inspection will be performed of the RNS pump suction piping.	iv) The RNS pump suction piping from the hot leg to the pump suction piping low point does not form a local high point (defined as an upward slope with a vertical rise greater than 3 inches).
			v) Inspection will be performed of the RNS pump suction nozzle connection to the RCS hot leg.	v) The RNS suction line connection to the RCS is constructed from 20-inch Schedule 140 pipe.
		9.c) The RNS provides low pressure makeup flow from the cask loading pit to the RCS for scenarios following actuation of the ADS.	Testing will be performed to confirm that the RNS can provide low pressure makeup flow from the cask loading pit to the RCS when the pump suction is aligned to the cask loading pit and the discharge is aligned to both PXS DVI lines with RCS at atmospheric pressure.	Each RNS pump provides at least 1100 gpm net flow to the RCS when the water level above the bottom of the cask loading pit is 1 foot ± 6 inches.
		9.d) The RNS provides heat removal from the in-containment refueling water storage tank (IRWST).	Testing will be performed to confirm that the RNS can provide flow through the RNS heat exchangers when the pump suction is aligned to the IRWST and the discharge is aligned to the IRWST.	Two operating RNS pumps provide at least 2000 gpm to the IRWST.

Table 2.3.6-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		12.a) The motor-operated and check valves identified in Table 2.3.6-1 perform an active safety-related function to change position as indicated in the table.	iii) Tests of the motor-operated valves will be performed under preoperational flow, differential pressure and temperature conditions. iv) Exercise testing of the check valves active safety functions identified in Table 2.3.6-1 will be performed under preoperational test pressure, temperature and fluid flow conditions.	iii) Each motor-operated valve changes position as indicated in Table 2.3.6-1 under preoperational test conditions. iv) Each check valve changes position as indicated in Table 2.3.6-1.
376	2.3.06.09b.iii	Not used per Amendment No. XX ~~9.b) The RNS provides heat removal from the reactor coolant during shutdown operations.~~	~~iii) Inspection will be performed of the reactor coolant loop piping.~~	~~iii) The RCS cold legs piping centerline is 17.5 inches ± 2 inches above the hot legs piping centerline.~~
377	2.3.06.09b.iv	Not used per Amendment No. XX ~~9.b) The RNS provides heat removal from the reactor coolant during shutdown operations.~~	~~iv) Inspection will be performed of the RNS pump suction piping.~~	~~iv) The RNS pump suction piping from the hot leg to the pump suction piping low point does not form a local high point (defined as an upward slope with a vertical rise greater than 3 inches).~~
378	2.3.06.09b.v	Not used per Amendment No. XX ~~9.b) The RNS provides heat removal from the reactor coolant during shutdown operations.~~	~~v) Inspection will be performed of the RNS pump suction nozzle connection to the RCS hot leg.~~	~~v) The RNS suction line connection to the RCS is constructed from 20-inch Schedule 140 pipe.~~
379	2.3.06.09c	Not used per Amendment No. XX ~~9.c) The RNS provides low pressure makeup flow from the cask loading pit to the RCS for scenarios following actuation of the ADS.~~	~~Testing will be performed to confirm that the RNS can provide low pressure makeup flow from the cask loading pit to the RCS when the pump suction is aligned to the cask loading pit and the discharge is aligned to both PXS DVI lines with RCS at atmospheric pressure.~~	~~Each RNS pump provides at least 1100 gpm net flow to the RCS when the water level above the bottom of the cask loading pit is 1 foot ± 6 inches.~~
380	2.3.06.09d	Not used per Amendment No. XX ~~9.d) The RNS provides heat removal from the in-containment refueling water storage tank (IRWST).~~	~~Testing will be performed to confirm that the RNS can provide flow through the RNS heat exchangers when the pump suction is aligned to the IRWST and the discharge is aligned to the IRWST.~~	~~Two operating RNS pumps provide at least 2000 gpm to the IRWST.~~

Table 2.3.6-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
381	2.3.06.10	Not used per Amendment No. XX ~~10. Safety-related displays identified in Table 2.3.6-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.3.6-1 can be retrieved in the MCR.~~
382	2.3.06.11a	10. Safety-related displays identified in Table 2.3.6-1 can be retrieved in the MCR. 11.a) Controls exist in the MCR to cause those remotely operated valves identified in Table 2.3.6-1 to perform active functions. 11.b) The valves identified in Table 2.3.6-1 as having PMS control perform active safety functions after receiving a signal from the PMS. 12.b) After loss of motive power, the remotely operated valves identified in Table 2.3.6-1 assume the indicated loss of motive power position. 13. Controls exist in the MCR to cause the pumps identified in Table 2.3.6-3 to perform the listed function. 14. Displays of the RNS parameters identified in Table 2.3.6-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR. Stroke testing will be performed on the remotely operated valves identified in Table 2.3.6-1 using the controls in the MCR. Testing will be performed using real or simulated signals into the PMS. Testing of the remotely operated valves will be performed under the conditions of loss of motive power. Testing will be performed to actuate the pumps identified in Table 2.3.6-3 using controls in the MCR. Inspection will be performed for retrievability in the MCR of the displays identified in Table 2.3.6-3.	Safety-related displays identified in Table 2.3.6-1 can be retrieved in the MCR. Controls in the MCR operate to cause those remotely operated valves identified in Table 2.3.6-1 to perform active functions. The valves identified in Table 2.3.6-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS. Upon loss of motive power, each remotely operated valve identified in Table 2.3.6-1 assumes the indicated loss of motive power position. Controls in the MCR cause pumps identified in Table 2.3.6-3 to perform the listed action. Displays of the RNS parameters identified in Table 2.3.6-3 are retrieved in the MCR.
383	2.3.06.11b	Not used per Amendment No. XX ~~11.b) The valves identified in Table 2.3.6-1 as having PMS control perform active safety functions after receiving a signal from the PMS.~~	~~Testing will be performed using real or simulated signals into the PMS.~~	~~The valves identified in Table 2.3.6-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.~~

Table 2.3.6-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
386	2.3.06.12a.iii	Not used per Amendment No. XX ~~12.a) The motor-operated and check valves identified in Table 2.3.6-1 perform an active safety-related function to change position as indicated in the table.~~	~~iii) Tests of the motor-operated valves will be performed under preoperational flow, differential pressure and temperature conditions.~~	~~iii) Each motor-operated valve changes position as indicated in Table 2.3.6-1 under preoperational test conditions.~~
387	2.3.06.12a.iv	Not used per Amendment No. XX ~~12.a) The motor-operated and check valves identified in Table 2.3.6-1 perform an active safety-related function to change position as indicated in the table.~~	~~iv) Exercise testing of the check valves active safety functions identified in Table 2.3.6-1 will be performed under preoperational test pressure, temperature and fluid flow conditions.~~	~~iv) Each check valve changes position as indicated in Table 2.3.6-1.~~
388	2.3.06.12b	Not used per Amendment No. XX ~~12.b) After loss of motive power, the remotely operated valves identified in Table 2.3.6-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~Upon loss of motive power, each remotely operated valve identified in Table 2.3.6-1 assumes the indicated loss of motive power position.~~
389	2.3.06.13	Not used per Amendment No. XX ~~13. Controls exist in the MCR to cause the pumps identified in Table 2.3.6-3 to perform the listed function.~~	~~Testing will be performed to actuate the pumps identified in Table 2.3.6-3 using controls in the MCR.~~	~~Controls in the MCR cause pumps identified in Table 2.3.6-3 to perform the listed action.~~
390	2.3.06.14	Not used per Amendment No. XX ~~14. Displays of the RNS parameters identified in Table 2.3.6-3 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability in the MCR of the displays identified in Table 2.3.6-3.~~	~~Displays of the RNS parameters identified in Table 2.3.6-3 are retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.7, Spent Fuel Pool Cooling System, Table 2.3.7-4 as shown below:

Table 2.3.7-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
408	2.3.07.07c	7c) The SFS provides check valves in the drain line from the refueling cavity to prevent flooding of the refueling cavity during containment flooding. 8. The SFS provides the nonsafety-related function of removing spent fuel decay heat using pumped flow through a heat exchanger. 9. Safety-related displays identified in Table 2.3.7-1 can be retrieved in the MCR. 10. Controls exist in the MCR to cause the pumps identified in Table 2.3.7-3 to perform their listed functions. 11. Displays of the SFS parameters identified in Table 2.3.7-3 can be retrieved in the MCR.	Exercise testing of the check valves with active safety-functions identified in Table 2.3.7-1 will be performed under pre-operational test pressure, temperature and flow conditions. ii) Testing will be performed to confirm that each SFS pump provides flow through its heat exchanger when taking suction from the SFP and returning flow to the SFP. Inspection will be performed for retrievability of the safety-related displays in the MCR. Testing will be performed to actuate the pumps identified in Table 2.3.7-3 using controls in the MCR. Inspection will be performed for retrievability in the MCR of the displays identified in Table 2.3.7-3.	Each check valve changes position as indicated on Table 2.3.7-1. ii) Each SFS pump produces at least 900 gpm through its heat exchanger. Safety-related displays identified in Table 2.3.7-1 can be retrieved in the MCR. Controls in the MCR cause pumps identified in Table 2.3.7-3 to perform the listed functions. Displays of the SFS parameters identified in Table 2.3.7-3 are retrieved in the MCR.
***				
410	2.3.07.08.ii	Not used per Amendment No. XX ~~8. The SFS provides the nonsafety-related function of removing spent fuel decay heat using pumped flow through a heat exchanger.~~	~~ii) Testing will be performed to confirm that each SFS pump provides flow through its heat exchanger when taking suction from the SFP and returning flow to the SFP.~~	~~ii) Each SFS pump produces at least 900 gpm through its heat exchanger.~~
411	2.3.07.09	Not used per Amendment No. XX ~~9. Safety-related displays identified in Table 2.3.7-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.3.7-1 can be retrieved in the MCR.~~

Table 2.3.7-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
412	2.3.07.10	Not used per Amendment No. XX ~~10. Controls exist in the MCR to cause the pumps identified in Table 2.3.7-3 to perform their listed functions.~~	~~Testing will be performed to actuate the pumps identified in Table 2.3.7-3 using controls in the MCR.~~	~~Controls in the MCR cause pumps identified in Table 2.3.7-3 to perform the listed functions.~~
413	2.3.07.11	Not used per Amendment No. XX ~~11. Displays of the SFS parameters identified in Table 2.3.7-3 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability in the MCR of the displays identified in Table 2.3.7-3.~~	~~Displays of the SFS parameters identified in Table 2.3.7-3 are retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.8, Service Water System, Table 2.3.8-2 as shown below:

Table 2.3.8-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
415	2.3.08.02.i	2. The SWS provides the nonsafety-related function of transferring heat from the component cooling water system to the surrounding atmosphere to support plant shutdown and spent fuel pool cooling. 3. Controls exist in the MCR to cause the components identified in Table 2.3.8-1 to perform the listed function. 4. Displays of the parameters identified in Table 2.3.8-1 can be retrieved in the MCR.	i) Testing will be performed to confirm that the SWS can provide cooling water to the CCS heat exchangers. Testing will be performed on the components in Table 2.3.8-1 using controls in the MCR. Inspection will be performed for retrievability of parameters in the MCR.	i) Each SWS pump can provide at least 10,000 gpm of cooling water through its CCS heat exchanger. Controls in the MCR operate to cause the components listed in Table 2.3.8-1 to perform the listed functions. The displays identified in Table 2.3.8-1 can be retrieved in the MCR.
***				
418	2.3.08.03	Not used per Amendment No. XX ~~3. Controls exist in the MCR to cause the components identified in Table 2.3.8-1 to perform the listed function.~~	~~Testing will be performed on the components in Table 2.3.8-1 using controls in the MCR.~~	~~Controls in the MCR operate to cause the components listed in Table 2.3.8-1 to perform the listed functions.~~
419	2.3.08.04	Not used per Amendment No. XX ~~4. Displays of the parameters identified in Table 2.3.8-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of parameters in the MCR.~~	~~The displays identified in Table 2.3.8-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1) Section 2.3.9, Containment Hydrogen Control System Table 2.3.9-3 as shown below:

Table 2.3.9-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
423	2.3.09.03.i	Not used per Amendment No. XX ~~3. The VLS provides the nonsafety-related function to control the containment hydrogen concentration for beyond design basis accidents.~~	~~i) Inspection for the number of igniters will be performed.~~	~~i) At least 66 hydrogen igniters are provided inside containment at the locations specified in Table 2.3.9-2.~~
424	2.3.09.03.ii	3. The VLS provides the nonsafety-related function to control the containment hydrogen concentration for beyond design basis accidents.	i) Inspection for the number of igniters will be performed. ii) Operability testing will be performed on the igniters.	i) At least 66 hydrogen igniters are provided inside containment at the locations specified in Table 2.3.9-2. ii) The surface temperature of the igniter meets or exceeds 1700°F.
		4.a) Controls exist in the MCR to cause the components identified in Table 2.3.9-2 to perform the listed function.	Testing will be performed on the igniters using the controls in the MCR.	Controls in the MCR operate to energize the igniters.
		5. Displays of the parameters identified in Table 2.3.9-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.3.9-1 in the MCR.	Displays identified in Table 2.3.9-1 can be retrieved in the MCR.
***				
427	2.3.09.04a	Not used per Amendment No. XX ~~4.a) Controls exist in the MCR to cause the components identified in Table 2.3.9-2 to perform the listed function.~~	~~Testing will be performed on the igniters using the controls in the MCR.~~	~~Controls in the MCR operate to energize the igniters.~~
***				
429	2.3.09.05	Not used per Amendment No. XX ~~5. Displays of the parameters identified in Table 2.3.9-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the displays identified in Table 2.3.9-1 in the MCR.~~	~~Displays identified in Table 2.3.9-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.10, Liquid Radwaste System, Table 2.3.10-4 as shown below:

Table 2.3.10-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
443	2.3.10.07a.i	Not used per Amendment No. XX ~~7.a) The WLS provides the nonsafety-related function of detecting leaks within containment to the containment sump.~~	~~i) Inspection will be performed for retrievability of the displays of containment sump level channels WLS-034, WLS-035, and WLS-036 in the MCR.~~	~~i) Nonsafety-related displays of WLS containment sump level channels WLS-034, WLS-035, and WLS-036 can be retrieved in the MCR.~~
444	2.3.10.07a.ii	7.a) The WLS provides the nonsafety-related function of detecting leaks within containment to the containment sump.	i) Inspection will be performed for retrievability of the displays of containment sump level channels WLS-034, WLS-035, and WLS-036 in the MCR.	i) Nonsafety-related displays of WLS containment sump level channels WLS-034, WLS-035, and WLS-036 can be retrieved in the MCR.
			ii) Testing will be performed by adding water to the sump and observing display of sump level.	ii) A report exists and concludes that sump level channels WLS-034, WLS-035, and WLS-036 can detect a change of 1.75 ± 0.1 inches.
		7.b) The WLS provides the nonsafety-related function of controlling releases of radioactive materials in liquid effluents.	Tests will be performed to confirm that a simulated high radiation signal from the discharge radiation monitor, WLS-RE-229, causes the discharge isolation valve WLS-PL-V223 to close.	A simulated high radiation signal causes the discharge control isolation valve WLS-PL-V223 to close.
		8. Controls exist in the MCR to cause the remotely operated valve identified in Table 2.3.10-3 to perform its active function.	Stroke testing will be performed on the remotely operated valve listed in Table 2.3.10-3 using controls in the MCR.	Controls in the MCR operate to cause the remotely operated valve to perform its active function.
		9. The check valves identified in Table 2.3.10-1 perform an active safety-related function to change position as indicated in the table.	Exercise testing of the check valves with active safety functions identified in Table 2.3.10-1 will be performed under pre-operational test pressure, temperature and flow conditions.	Each check valve changes position as indicated on Table 2.3.10-1.
		10. Displays of the parameters identified in Table 2.3.10-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.3.10-3 in the MCR.	Displays identified in Table 2.3.10-3 can be retrieved in the MCR.

Table 2.3.10-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
445	2.3.10.07b	Not used per Amendment No. XX ~~7.b) The WLS provides the nonsafety-related function of controlling releases of radioactive materials in liquid effluents.~~	~~Tests will be performed to confirm that a simulated high radiation signal from the discharge radiation monitor, WLS-RE-229, causes the discharge isolation valve WLS-PL-V223 to close.~~	~~A simulated high radiation signal causes the discharge control isolation valve WLS-PL-V223 to close.~~
446	2.3.10.08	Not used per Amendment No. XX ~~8. Controls exist in the MCR to cause the remotely operated valve identified in Table 2.3.10-3 to perform its active function.~~	~~Stroke testing will be performed on the remotely operated valve listed in Table 2.3.10-3 using controls in the MCR.~~	~~Controls in the MCR operate to cause the remotely operated valve to perform its active function.~~
447	2.3.10.09	Not used per Amendment No. XX ~~9. The check valves identified in Table 2.3.10-1 perform an active safety-related function to change position as indicated in the table.~~	~~Exercise testing of the check valves with active safety functions identified in Table 2.3.10-1 will be performed under pre-operational test pressure, temperature and flow conditions.~~	~~Each check valve changes position as indicated on Table 2.3.10-1.~~
448	2.3.10.10	Not used per Amendment No. XX ~~10. Displays of the parameters identified in Table 2.3.10-3 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the displays identified in Table 2.3.10-3 in the MCR.~~	~~Displays identified in Table 2.3.10-3 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.13, Primary Sampling System, Table 2.3.13-3 as shown below:

Table 2.3.13-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
470	2.3.13.08	8. The PSS provides the nonsafety-related function of providing the capability of obtaining reactor coolant and containment atmosphere samples. 9. Safety-related displays identified in Table 2.3.13-1 can be retrieved in the MCR. 10.a) Controls exist in the MCR to cause those remotely operated valves identified in Table 2.3.13-1 to perform active functions. 10.b) The valves identified in Table 2.3.13-1 as having PMS control perform an active function after receiving a signal from the PMS. 11.b) After loss of motive power, the remotely operated valves identified in Table 2.3.13-1 assume the indicated loss of motive power position. 12. Controls exist in the MCR to cause the valves identified in Table 2.3.13-2 to perform the listed function.	Testing will be performed to obtain samples of the reactor coolant and containment atmosphere. Inspection will be performed for retrievability of the safety-related displays in the MCR. Stroke testing will be performed on the remotely operated valves identified in Table 2.3.13-1 using the controls in the MCR. Testing will be performed on remotely operated valves listed in Table 2.3.13-1 using real or simulated signals into the PMS. Testing of the remotely operated valves will be performed under the conditions of loss of motive power. Testing will be performed on the components in Table 2.3.13-2 using controls in the MCR.	A sample is drawn from the reactor coolant and the containment atmosphere. The safety-related displays identified in Table 2.3.13-1 can be retrieved in the MCR. Controls in the MCR operate to cause those remotely operated valves identified in Table 2.3.13-1 to perform active functions. The remotely operated valves identified in Table 2.3.13-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS. After loss of motive power, each remotely operated valve identified in Table 2.3.13-1 assumes the indicated loss of motive power position. Controls in the MCR cause valves identified in Table 2.3.13-2 to perform the listed functions.
471	2.3.13.09	Not used per Amendment No. XX ~~9. Safety-related displays identified in Table 2.3.13-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~The safety-related displays identified in Table 2.3.13-1 can be retrieved in the MCR.~~
472	2.3.13.10a	Not used per Amendment No. XX ~~10.a) Controls exist in the MCR to cause those remotely operated valves identified in Table 2.3.13-1 to perform active functions.~~	~~Stroke testing will be performed on the remotely operated valves identified in Table 2.3.13-1 using the controls in the MCR.~~	~~Controls in the MCR operate to cause those remotely operated valves identified in Table 2.3.13-1 to perform active functions.~~

Table 2.3.13-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
473	2.3.13.10b	Not used per Amendment No. XX ~~10.b) The valves identified in Table 2.3.13-1 as having PMS control perform an active function after receiving a signal from the PMS.~~	~~Testing will be performed on remotely operated valves listed in Table 2.3.13-1 using real or simulated signals into the PMS.~~	~~The remotely operated valves identified in Table 2.3.13-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.~~
***				
475	2.3.13.11b	Not used per Amendment No. XX ~~11.b) After loss of motive power, the remotely operated valves identified in Table 2.3.13-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~After loss of motive power, each remotely operated valve identified in Table 2.3.13-1 assumes the indicated loss of motive power position.~~
476	2.3.13.12	Not used per Amendment No. XX ~~12. Controls exist in the MCR to cause the valves identified in Table 2.3.13-2 to perform the listed function.~~	~~Testing will be performed on the components in Table 2.3.13-2 using controls in the MCR.~~	~~Controls in the MCR cause valves identified in Table 2.3.13-2 to perform the listed functions.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.19, Communication System, Table 2.3.19-2 as shown below:

Table 2.3.19-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
484	2.3.19.01a	Not used per Amendment No. XX ~~1.a) The EFS has handsets, amplifiers, loudspeakers, and siren tone generators connected as a telephone/page system.~~	~~Inspection of the as-built system will be performed.~~	~~The as-built EFS has handsets, amplifiers, loudspeakers, and siren tone generators connected as a telephone/page system.~~
485	2.3.19.01b	Not used per Amendment No. XX ~~1.b) The EFS has sound-powered equipment connected as a system.~~	~~Inspection of the as-built system will be performed.~~	~~The as-built EFS has sound-powered equipment connected as a system.~~
486	2.3.19.02a	1.a) The EFS has handsets, amplifiers, loudspeakers, and siren tone generators connected as a telephone/page system.	Inspection of the as-built system will be performed.	The as-built EFS has handsets, amplifiers, loudspeakers, and siren tone generators connected as a telephone/page system.
		1.b) The EFS has sound-powered equipment connected as a system.	Inspection of the as-built system will be performed.	The as-built EFS has sound-powered equipment connected as a system.
		2.a) The EFS telephone/page system provides intraplant, station-to-station communications and area broadcasting between the MCR and the locations listed in Table 2.3.19-1.	An inspection and test will be performed on the telephone/page communication equipment.	Telephone/page equipment is installed and voice transmission and reception from the MCR are accomplished.
		2.b) EFS provides sound-powered communications between the MCR, the RSW, the Division A, B, C, D dc equipment rooms (Rooms 12201/12203/12205/ 12207), the Division A, B, C, D I&C rooms (Rooms 12301/12302/ 12304/12305), and the diesel generator building (Rooms 60310/60320) without external power.	An inspection and test will be performed of the sound-powered communication equipment.	Sound-powered equipment is installed and voice transmission and reception are accomplished.

Table 2.3.19-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
487	2.3.19.02b	Not used per Amendment No. XX ~~2.b) EFS provides sound-powered communications between the MCR, the RSW, the Division A, B, C, D dc equipment rooms (Rooms 12201/12203/12205/ 12207), the Division A, B, C, D I&C rooms (Rooms 12301/12302/ 12304/12305), and the diesel generator building (Rooms 60310/60320) without external power.~~	~~An inspection and test will be performed of the sound-powered communication equipment.~~	~~Sound-powered equipment is installed and voice transmission and reception are accomplished.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.29, Radioactive Waste Drain System, Table 2.3.29-1 as shown below:

Table 2.3.29-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
489	2.3.29.02	2. The WRS collects liquid wastes from the equipment and floor drainage of the radioactive portions of the auxiliary building, annex building, and radwaste building and directs these wastes to a WRS sump or WLS waste holdup tanks located in the auxiliary building. 3. The WRS collects chemical wastes from the auxiliary building chemical laboratory drains and the decontamination solution drains in the annex building and directs these wastes to the chemical waste tank of the liquid radwaste system.	A test is performed by pouring water into the equipment and floor drains in the radioactive portions of the auxiliary building, annex building, and radwaste building. A test is performed by pouring water into the auxiliary building chemical laboratory and the decontamination solution drains in the annex building.	The water poured into these drains is collected either in the auxiliary building radioactive drains sump or the WLS waste holdup tanks. The water poured into these drains is collected in the chemical waste tank of the liquid radwaste system.
490	2.3.29.03	Not used per Amendment No. XX ~~3. The WRS collects chemical wastes from the auxiliary building chemical laboratory drains and the decontamination solution drains in the annex building and directs these wastes to the chemical waste tank of the liquid radwaste system.~~	~~A test is performed by pouring water into the auxiliary building chemical laboratory and the decontamination solution drains in the annex building.~~	~~The water poured into these drains is collected in the chemical waste tank of the liquid radwaste system.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.4.1, Main and Startup Feedwater System, Table 2.4.1-2 as shown below:

Table 2.4.1-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
493	2.4.01.02	2. The FWS provides startup feedwater flow from the CST to the SGS for heat removal from the RCS. 3. Controls exist in the MCR to cause the components identified in Table 2.4.1-1 to perform the listed function. 4. Displays of the parameters identified in Table 2.4.1-1 can be retrieved in the MCR.	Testing will be performed to confirm that each of the startup feedwater pumps can provide water from the CST to both steam generators. Testing will be performed on the components in Table 2.4.1-1 using controls in the MCR. Inspection will be performed for retrievability of parameters in the MCR.	Each FWS startup feedwater pump provides a flow rate greater than or equal to 260 gpm to each steam generator system at a steam generator secondary side pressure of at least 1106 psia. Controls in the MCR operate to cause the components listed in Table 2.4.1-1 to perform the listed functions. The displays identified in Table 2.4.1-1 can be retrieved in the MCR.
494	2.4.01.03	Not used per Amendment No. XX ~~3. Controls exist in the MCR to cause the components identified in Table 2.4.1-1 to perform the listed function.~~	~~Testing will be performed on the components in Table 2.4.1-1 using controls in the MCR.~~	~~Controls in the MCR operate to cause the components listed in Table 2.4.1-1 to perform the listed functions.~~
495	2.4.01.04	Not used per Amendment No. XX ~~4. Displays of the parameters identified in Table 2.4.1-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of parameters in the MCR.~~	~~The displays identified in Table 2.4.1-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.4.2, Main Turbine System, Table 2.4.2-1 as shown below:

Table 2.4.2-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
497	2.4.02.02a	2.a) Controls exist in the MCR to trip the main turbine-generator. 2.c) The main turbine-generator trips after receiving a signal from the DAS. 3) The trip signals from the two turbine electrical overspeed protection trip systems are isolated from, and independent of, each other.	Testing will be performed on the main turbine-generator using controls in the MCR. Testing will be performed using real or simulated signals into the DAS. ii) Testing of the as-built system will be performed using simulated signals from the turbine speed sensors.	Controls in the MCR operate to trip the main turbine-generator. The main turbine-generator trips after receiving a signal from the DAS. ii) The main turbine-generator trips after overspeed signals are received from the speed sensors of the 110% emergency electrical overspeed trip system, and the main turbine-generator trips after overspeed signals are received from the speed sensors of the 111% backup electrical overspeed trip system.
***				
499	2.4.02.02c	Not used per Amendment No. XX ~~2.c) The main turbine-generator trips after receiving a signal from the DAS.~~	~~Testing will be performed using real or simulated signals into the DAS.~~	~~The main turbine-generator trips after receiving a signal from the DAS.~~
***				
501	2.4.02.03.ii	Not used per Amendment No. XX ~~3) The trip signals from the two turbine electrical overspeed protection trip systems are isolated from, and independent of, each other.~~	~~ii) Testing of the as-built system will be performed using simulated signals from the turbine speed sensors.~~	~~ii) The main turbine-generator trips after overspeed signals are received from the speed sensors of the 110% emergency electrical overspeed trip system, and the main turbine-generator trips after overspeed signals are received from the speed sensors of the 111% backup electrical overspeed trip system.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.5.1, Diverse Actuation System, Table 2.5.1-4 as shown below:

Table 2.5.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
506	2.5.01.02a	2.a) The DAS provides an automatic reactor trip on low wide-range steam generator water level, or on low pressurizer water level, or on high hot leg temperature, separate from the PMS.	Electrical power to the PMS equipment will be disconnected and an operational test of the as-built DAS will be performed using real or simulated test signals.	The generator field control relays (contained in the control cabinets for the rod drive motor-generator sets) open after the test signal reaches the specified limit.
		2.b) The DAS provides automatic actuation of selected functions, as identified in Table 2.5.1-1, separate from the PMS.	Electrical power to the PMS equipment will be disconnected and an operational test of the as-built DAS will be performed using real or simulated test signals.	Appropriate DAS output signals are generated after the test signal reaches the specified limit.
		2.c) The DAS provides manual initiation of reactor trip, and selected functions, as identified in Table 2.5.1-2, separate from the PMS. These manual initiation functions are implemented in a manner that bypasses the control room multiplexers, if any; the PMS cabinets; and the signal processing equipment of the DAS.	Electrical power to the control room multiplexers, if any, and PMS equipment will be disconnected and the outputs from the DAS signal processing equipment will be disabled. While in this configuration, an operational test of the as-built system will be performed using the DAS manual actuation controls.	i) The generator field control relays (contained in the control cabinets for the rod drive motor-generator sets) open after reactor and turbine trip manual initiation controls are actuated.
			Electrical power to the control room multiplexers, if any, and PMS equipment will be disconnected and the outputs from the DAS signal processing equipment will be disabled. While in this configuration, an operational test of the as-built system will be performed using the DAS manual actuation controls.	ii) DAS output signals are generated for the selected functions, as identified in Table 2.5.1-2, after manual initiation controls are actuated.
		2.d) The DAS provides MCR displays of selected plant parameters, as identified in Table 2.5.1-3, separate from the PMS.	Electrical power to the PMS equipment will be disconnected and inspection will be performed for retrievability of the selected plant parameters in the MCR.	The selected plant parameters can be retrieved in the MCR.

Table 2.5.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		3.f) The DAS is powered by non-Class 1E uninterruptible power supplies that are independent and separate from the power supplies which power the PMS. 3.g) The DAS signal processing cabinets are provided with the capability for channel testing without actuating the controlled components.	Electrical power to the PMS equipment will be disconnected. While in this configuration, a test will be performed by providing simulated test signals in the non-Class 1E uninterruptible power supplies. Channel tests will be performed on the as built system.	A simulated test signal exists at the DAS equipment when the assigned non-Class 1E uninterruptible power supply is provided the test signal. The capability exists for testing individual DAS channels without propagating an actuation signal to a DAS controlled component.
507	2.5.01.02b	Not used per Amendment No. XX ~~2.b) The DAS provides automatic actuation of selected functions, as identified in Table 2.5.1-1, separate from the PMS.~~	~~Electrical power to the PMS equipment will be disconnected and an operational test of the as-built DAS will be performed using real or simulated test signals.~~	~~Appropriate DAS output signals are generated after the test signal reaches the specified limit.~~
508	2.5.01.02c.i	Not used per Amendment No. XX ~~2.c) The DAS provides manual initiation of reactor trip, and selected functions, as identified in Table 2.5.1-2, separate from the PMS. These manual initiation functions are implemented in a manner that bypasses the control room multiplexers, if any; the PMS cabinets; and the signal processing equipment of the DAS.~~	~~Electrical power to the control room multiplexers, if any, and PMS equipment will be disconnected and the outputs from the DAS signal processing equipment will be disabled. While in this configuration, an operational test of the as-built system will be performed using the DAS manual actuation controls.~~	~~i) The generator field control relays (contained in the control cabinets for the rod drive motor-generator sets) open after reactor and turbine trip manual initiation controls are actuated.~~
509	2.5.01.02c.ii	Not used per Amendment No. XX ~~2.c) The DAS provides manual initiation of reactor trip, and selected functions, as identified in Table 2.5.1-2, separate from the PMS. These manual initiation functions are implemented in a manner that bypasses the control room multiplexers, if any; the PMS cabinets; and the signal processing equipment of the DAS.~~	~~Electrical power to the control room multiplexers, if any, and PMS equipment will be disconnected and the outputs from the DAS signal processing equipment will be disabled. While in this configuration, an operational test of the as-built system will be performed using the DAS manual actuation controls.~~	~~ii) DAS output signals are generated for the selected functions, as identified in Table 2.5.1-2, after manual initiation controls are actuated.~~
510	2.5.01.02d	Not used per Amendment No. XX ~~2.d) The DAS provides MCR displays of selected plant parameters, as identified in Table 2.5.1-3, separate from the PMS.~~	~~Electrical power to the PMS equipment will be disconnected and inspection will be performed for retrievability of the selected plant parameters in the MCR.~~	~~The selected plant parameters can be retrieved in the MCR.~~

***

Table 2.5.1-4 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
516	2.5.01.03f	Not used per Amendment No. XX ~~3.f) The DAS is powered by non-Class 1E uninterruptible power supplies that are independent and separate from the power supplies which power the PMS.~~	~~Electrical power to the PMS equipment will be disconnected. While in this configuration, a test will be performed by providing simulated test signals in the non-Class 1E uninterruptible power supplies.~~	~~A simulated test signal exists at the DAS equipment when the assigned non-Class 1E uninterruptible power supply is provided the test signal.~~
517	2.5.01.03g	Not used per Amendment No. XX ~~3.g) The DAS signal processing cabinets are provided with the capability for channel testing without actuating the controlled components.~~	~~Channel tests will be performed on the as built system.~~	~~The capability exists for testing individual DAS channels without propagating an actuation signal to a DAS controlled component.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.5.2, Protection and Safety Monitoring System, Table 2.5.2-8 as shown below:

Table 2.5.2-8
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
530	2.5.02.06a.ii	6.a) The PMS initiates an automatic reactor trip, as identified in Table 2.5.2-2, when plant process signals reach specified limits.	An operational test of the as-built PMS will be performed using real or simulated test signals.	ii) PMS output signals to the reactor trip switchgear are generated after the test signal reaches the specified limit. This needs to be verified for each automatic reactor trip function.
		6.b) The PMS initiates automatic actuation of engineered safety features, as identified in Table 2.5.2-3, when plant process signals reach specified limits.	An operational test of the as-built PMS will be performed using real or simulated test signals.	Appropriate PMS output signals are generated after the test signal reaches the specified limit. These output signals remain following removal of the test signal. Tests from the actuation signal to the actuated device(s) are performed as part of the system-related inspection, test, analysis, and acceptance criteria.
		6.c) The PMS provides manual initiation of reactor trip and selected engineered safety features as identified in Table 2.5.2-4.	An operational test of the as-built PMS will be performed using the PMS manual actuation controls.	ii) PMS output signals are generated for reactor trip and selected engineered safety features as identified in Table 2.5.2-4 after the manual initiation controls are actuated.
		8.a) The PMS provides for the minimum inventory of displays, visual alerts, and fixed position controls, as identified in Table 2.5.2-5. The plant parameters listed with a "Yes" in the "Display" column and visual alerts listed with a "Yes" in the "Alert" column can be retrieved in the MCR. The fixed position controls listed with a "Yes" in the "Control" column are provided in the MCR.	i) An inspection will be performed for retrievability of plant parameters in the MCR.	i) The plant parameters listed in Table 2.5.2-5 with a "Yes" in the "Display" column, can be retrieved in the MCR.

Table 2.5.2-8
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
			iii) An operational test of the as-built system will be performed using each MCR fixed position control.	iii) For each test of an as-built fixed position control listed in Table 2.5.2-5 with a "Yes" in the "Control" column, an actuation signal is generated. Tests from the actuation signal to the actuated device(s) are performed as part of the system-related inspection, test, analysis and acceptance criteria.
		8.c) Displays of the open/closed status of the reactor trip breakers can be retrieved in the MCR.	Inspection will be performed for retrievability of displays of the open/closed status of the reactor trip breakers in the MCR.	Displays of the open/closed status of the reactor trip breakers can be retrieved in the MCR.
		9.a) The PMS automatically removes blocks of reactor trip and engineered safety features actuation when the plant approaches conditions for which the associated function is designed to provide protection. These blocks are identified in Table 2.5.2-6.	An operational test of the as-built PMS will be performed using real or simulated test signals.	The PMS blocks are automatically removed when the test signal reaches the specified limit.
		9.b) The PMS two-out-of-four initiation logic reverts to a two-out-of-three coincidence logic if one of the four channels is bypassed. All bypassed channels are alarmed in the MCR.	An operational test of the as-built PMS will be performed.	The PMS two-out-of-four initiation logic reverts to a two-out-of-three coincidence logic if one of the four channels is bypassed. All bypassed channels are alarmed in the MCR.
		9.c) The PMS does not allow simultaneous bypass of two redundant channels.	An operational test of the as-built PMS will be performed. With one channel in bypass, an attempt will be made to place a redundant channel in bypass.	The redundant channel cannot be placed in bypass.

Table 2.5.2-8
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
531	2.5.02.06b	Not used per Amendment No. XX ~~6.b) The PMS initiates automatic actuation of engineered safety features, as identified in Table 2.5.2-3, when plant process signals reach specified limits.~~	~~An operational test of the as-built PMS will be performed using real or simulated test signals.~~	~~Appropriate PMS output signals are generated after the test signal reaches the specified limit. These output signals remain following removal of the test signal. Tests from the actuation signal to the actuated device(s) are performed as part of the system-related inspection, test, analysis, and acceptance criteria.~~
***				
533	2.5.02.06c.ii	Not used per Amendment No. XX ~~6.c) The PMS provides manual initiation of reactor trip and selected engineered safety features as identified in Table 2.5.2-4.~~	~~An operational test of the as-built PMS will be performed using the PMS manual actuation controls.~~	~~ii) PMS output signals are generated for reactor trip and selected engineered safety features as identified in Table 2.5.2-4 after the manual initiation controls are actuated.~~
***				
539	2.5.02.08a.i	Not used per Amendment No. XX ~~8.a) The PMS provides for the minimum inventory of displays, visual alerts, and fixed position controls, as identified in Table 2.5.2-5. The plant parameters listed with a "Yes" in the "Display" column and visual alerts listed with a "Yes" in the "Alert" column can be retrieved in the MCR. The fixed position controls listed with a "Yes" in the "Control" column are provided in the MCR.~~	~~i) An inspection will be performed for retrievability of plant parameters in the MCR.~~	~~i) The plant parameters listed in Table 2.5.2-5 with a "Yes" in the "Display" column, can be retrieved in the MCR.~~
***				

Table 2.5.2-8
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
541	2.5.02.08a.iii	Not used per Amendment No. XX ~~8.a) The PMS provides for the minimum inventory of displays, visual alerts, and fixed position controls, as identified in Table 2.5.2-5. The plant parameters listed with a "Yes" in the "Display" column and visual alerts listed with a "Yes" in the "Alert" column can be retrieved in the MCR. The fixed position controls listed with a "Yes" in the "Control" column are provided in the MCR.~~	~~iii) An operational test of the as-built system will be performed using each MCR fixed position control.~~	~~iii) For each test of an as-built fixed position control listed in Table 2.5.2-5 with a "Yes" in the "Control" column, an actuation signal is generated. Tests from the actuation signal to the actuated device(s) are performed as part of the system-related inspection, test, analysis and acceptance criteria.~~
***				
544	2.5.02.08c	Not used per Amendment No. XX ~~8.c) Displays of the open/closed status of the reactor trip breakers can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of displays of the open/closed status of the reactor trip breakers in the MCR.~~	~~Displays of the open/closed status of the reactor trip breakers can be retrieved in the MCR.~~
545	2.5.02.09a	Not used per Amendment No. XX ~~9.a) The PMS automatically removes blocks of reactor trip and engineered safety features actuation when the plant approaches conditions for which the associated function is designed to provide protection. These blocks are identified in Table 2.5.2-6.~~	~~An operational test of the as-built PMS will be performed using real or simulated test signals.~~	~~The PMS blocks are automatically removed when the test signal reaches the specified limit.~~
546	2.5.02.09b	Not used per Amendment No. XX ~~9.b) The PMS two-out-of-four initiation logic reverts to a two-out-of-three coincidence logic if one of the four channels is bypassed. All bypassed channels are alarmed in the MCR.~~	~~An operational test of the as-built PMS will be performed.~~	~~The PMS two-out-of-four initiation logic reverts to a two-out-of-three coincidence logic if one of the four channels is bypassed. All bypassed channels are alarmed in the MCR.~~
547	2.5.02.09c	Not used per Amendment No. XX ~~9.c) The PMS does not allow simultaneous bypass of two redundant channels.~~	~~An operational test of the as-built PMS will be performed. With one channel in bypass, an attempt will be made to place a redundant channel in bypass.~~	~~The redundant channel cannot be placed in bypass.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.5.4, Data Display and Processing System, Table 2.5.4-2 as shown below:

*Note: Changes to ITAAC Nos. 2.5.04.02.i, 2.5.04.02.ii and 2.5.04.02.iii in Table 2.5.4-2, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 2.i, 2ii and 2.iii in Table 2.5.4-2 is needed.

Table 2.5.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
557	2.5.04.02.i	2. The DDS provides for the minimum inventory of displays, visual alerts, and fixed position controls, as identified in Table 2.5.4-1. The plant parameters listed with a "Yes" in the "Display" column and visual alerts listed with a "Yes" in the "Alert" column can be retrieved at the RSW. The controls listed with a "Yes" in the "Control" column are provided at the RSW.	i) An inspection will be performed for retrievability of plant parameters at the RSW. ii) An inspection and test will be performed to verify that the plant parameters are used to generate visual alerts that identify challenges to critical safety functions. iii) An operational test of the as-built system will be performed using each RSW control.	i) The plant parameters listed in Table 2.5.4-1 with a "Yes" in the "Display" column can be retrieved at the RSW. ii) The plant parameters listed in Table 2.5.4-1 with a "Yes" in the "Alert" column are used to generate visual alerts that identify challenges to critical safety functions. The visual alerts actuate in accordance with their logic and values. iii) For each test of a control listed in Table 2.5.4-1 with a "Yes" in the "Control" column, an actuation signal is generated. Tests from the actuation signal to the actuated device(s) are performed as part of the system-related inspection, test, analysis and acceptance criteria.

Table 2.5.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
558	2.5.04.02.ii	Not used per Amendment No. XX ~~2. The DDS provides for the minimum inventory of displays, visual alerts, and fixed position controls, as identified in Table 2.5.4-1. The plant parameters listed with a "Yes" in the "Display" column and visual alerts listed with a "Yes" in the "Alert" column can be retrieved at the RSW. The controls listed with a "Yes" in the "Control" column are provided at the RSW.~~	~~ii) An inspection and test will be performed to verify that the plant parameters are used to generate visual alerts that identify challenges to critical safety functions.~~	~~ii) The plant parameters listed in Table 2.5.4-1 with a "Yes" in the "Alert" column are used to generate visual alerts that identify challenges to critical safety functions. The visual alerts actuate in accordance with their logic and values.~~
559	2.5.04.02.iii	Not used per Amendment No. XX ~~2. The DDS provides for the minimum inventory of displays, visual alerts, and fixed position controls, as identified in Table 2.5.4-1. The plant parameters listed with a "Yes" in the "Display" column and visual alerts listed with a "Yes" in the "Alert" column can be retrieved at the RSW. The controls listed with a "Yes" in the "Control" column are provided at the RSW.~~	~~iii) An operational test of the as-built system will be performed using each RSW control.~~	~~iii) For each test of a control listed in Table 2.5.4-1 with a "Yes" in the "Control" column, an actuation signal is generated. Tests from the actuation signal to the actuated device(s) are performed as part of the system-related inspection, test, analysis and acceptance criteria.~~
***				
561	C.2.5.04.04a	4. The plant calorimetric uncertainty and plant instrumentation performance is bounded by the 1% calorimetric uncertainty value assumed for the initial reactor power in the safety analysis.	Inspection will be performed of the plant operating instrumentation installed for feedwater flow measurement, its associated power calorimetric uncertainty calculation, and the calculated calorimetric values. Inspection will be performed of the plant operating instrumentation installed for feedwater flow measurement, its associated power calorimetric uncertainty calculation, and the calculated calorimetric values.	a) The as-built system takes input for feedwater flow measurement from a Caldon [Cameron] LEFM CheckPlus™ System; b) the power calorimetric uncertainty calculation documented for that instrumentation is based on an accepted Westinghouse methodology and the uncertainty values for that instrumentation are not lower than those for the actual installed instrumentation; and

Table 2.5.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
			Inspection will be performed of the plant operating instrumentation installed for feedwater flow measurement, its associated power calorimetric uncertainty calculation, and the calculated calorimetric values.	c) the calculated calorimetric power uncertainty measurement values are bounded by the 1% uncertainty value assumed for the initial reactor power in the safety analysis.
562	C.2.5.04.04b	Not used per Amendment No. XX ~~4. The plant calorimetric uncertainty and plant instrumentation performance is bounded by the 1% calorimetric uncertainty value assumed for the initial reactor power in the safety analysis.~~	~~Inspection will be performed of the plant operating instrumentation installed for feedwater flow measurement, its associated power calorimetric uncertainty calculation, and the calculated calorimetric values.~~	~~b) the power calorimetric uncertainty calculation documented for that instrumentation is based on an accepted Westinghouse methodology and the uncertainty values for that instrumentation are not lower than those for the actual installed instrumentation; and~~
563	C.2.5.04.04c	Not used per Amendment No. XX ~~4. The plant calorimetric uncertainty and plant instrumentation performance is bounded by the 1% calorimetric uncertainty value assumed for the initial reactor power in the safety analysis.~~	~~Inspection will be performed of the plant operating instrumentation installed for feedwater flow measurement, its associated power calorimetric uncertainty calculation, and the calculated calorimetric values.~~	~~c) the calculated calorimetric power uncertainty measurement values are bounded by the 1% uncertainty value assumed for the initial reactor power in the safety analysis.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.1, Main ac Power System, Table 2.6.1-4 as shown below:

Table 2.6.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
584	2.6.01.04a	Not used per Amendment No. XX ~~4.a) The ECS provides the capability for distributing non-Class 1E ac power from onsite sources (ZOS) to nonsafety-related loads listed in Table 2.6.1-2.~~	~~Tests will be performed using a test signal to confirm that an electrical path exists for each selected load listed in Table 2.6.1-2 from an ECS-ES-1 or ECS-ES-2 bus. Each test may be a single test or a series of over-lapping tests.~~	~~A test signal exists at the terminals of each selected load.~~
***				
588	2.6.01.04e	4.a) The ECS provides the capability for distributing non-Class 1E ac power from onsite sources (ZOS) to nonsafety-related loads listed in Table 2.6.1-2.	Tests will be performed using a test signal to confirm that an electrical path exists for each selected load listed in Table 2.6.1-2 from an ECS-ES-1 or ECS-ES-2 bus. Each test may be a single test or a series of over-lapping tests.	A test signal exists at the terminals of each selected load.
		4.e) The ECS provides two loss-of-voltage signals to the onsite standby power system (ZOS), one for each diesel-backed 6900 Vac switchgear bus.	Tests on the as-built ECS system will be conducted by simulating a loss-of-voltage condition on each diesel-backed 6900 Vac switchgear bus.	A loss-of-voltage signal is generated when the loss-of-voltage condition is simulated.
		4.f) The ECS provides a reverse-power trip of the generator circuit breaker which is blocked for at least 15 seconds following a turbine trip.	Tests on the as-built ECS system will be conducted by simulating a turbine trip signal followed by a simulated reverse-power condition. The generator circuit breaker trip signal will be monitored.	The generator circuit breaker trip signal does not occur until at least 15 seconds after the simulated turbine trip.
		5. Controls exist in the MCR to cause the circuit breakers identified in Table 2.6.1-3 to perform the listed functions.	Tests will be performed to verify that controls in the MCR can operate the circuit breakers identified in Table 2.6.1-3.	Controls in the MCR cause the circuit breakers identified in Table 2.6.1-3 to operate.
		6. Displays of the parameters identified in Table 2.6.1-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.6.1-3 in the MCR.	Displays identified in Table 2.6.1-3 can be retrieved in the MCR.

Table 2.6.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
589	2.6.01.04f	Not used per Amendment No. XX ~~4.f) The ECS provides a reverse-power trip of the generator circuit breaker which is blocked for at least 15 seconds following a turbine trip.~~	~~Tests on the as-built ECS system will be conducted by simulating a turbine trip signal followed by a simulated reverse-power condition. The generator circuit breaker trip signal will be monitored.~~	~~The generator circuit breaker trip signal does not occur until at least 15 seconds after the simulated turbine trip.~~
590	2.6.01.05	Not used per Amendment No. XX ~~5. Controls exist in the MCR to cause the circuit breakers identified in Table 2.6.1-3 to perform the listed functions.~~	~~Tests will be performed to verify that controls in the MCR can operate the circuit breakers identified in Table 2.6.1-3.~~	~~Controls in the MCR cause the circuit breakers identified in Table 2.6.1-3 to operate.~~
591	2.6.01.06	Not used per Amendment No. XX ~~6. Displays of the parameters identified in Table 2.6.1-3 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the displays identified in Table 2.6.1-3 in the MCR.~~	~~Displays identified in Table 2.6.1-3 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.3, Class 1E dc and Uninterruptible Power Supply System, Table 2.6.3-3 as shown below:

*Note: Changes to ITAAC Nos. 2.6.03.05d.i and 2.6.03.05d.ii in Table 2.6.3-3, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 5d.i and 5d.ii in Table 2.6.3-3 is needed.

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
603	2.6.03.04c	4.c) Each IDS 24-hour battery bank supplies a dc switchboard bus load for a period of 24 hours without recharging.	Testing of each 24-hour as-built battery bank will be performed by applying a simulated or real load, or a combination of simulated or real loads which envelope the battery bank design duty cycle. The test will be conducted on a battery bank that has been fully charged and has been connected to a battery charger maintained at 270±2 V for a period of no less than 24 hours prior to the test.	The battery terminal voltage is greater than or equal to 210 V after a period of no less than 24 hours with an equivalent load that equals or exceeds the battery bank design duty cycle capacity.
		4.d) Each IDS 72-hour battery bank supplies a dc switchboard bus load for a period of 72 hours without recharging.	Testing of each 72-hour as-built battery bank will be performed by applying a simulated or real load, or a combination of simulated or real loads which envelope the battery bank design duty cycle. The test will be conducted on a battery bank that has been fully charged and has been connected to a battery charger maintained at 270±2 V for a period of no less than 24 hours prior to the test.	The battery terminal voltage is greater than or equal to 210 V after a period of no less than 72 hours with an equivalent load that equals or exceeds the battery bank design duty cycle capacity.

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		4.e) The IDS spare battery bank supplies a dc load equal to or greater than the most severe switchboard bus load for the required period without recharging.	Testing of the as-built spare battery bank will be performed by applying a simulated or real load, or a combination of simulated or real loads which envelope the most severe of the division batteries design duty cycle. The test will be conducted on a battery bank that has been fully charged and has been connected to a battery charger maintained at 270±2 V for a period of no less than 24 hours prior to the test.	The battery terminal voltage is greater than or equal to 210 V after a period with a load and duration that equals or exceeds the most severe battery bank design duty cycle capacity.
		4.f) Each IDS 24-hour inverter supplies its ac load.	Testing of each 24-hour as-built inverter will be performed by applying a simulated or real load, or a combination of simulated or real loads, equivalent to a resistive load greater than 12 kW. The inverter input voltage will be no more than 210 Vdc during the test.	Each 24-hour inverter supplies a line-to-line output voltage of 208 ± 2% V at a frequency of 60 ± 0.5% Hz.
		4.g) Each IDS 72-hour inverter supplies its ac load.	Testing of each 72-hour as-built inverter will be performed by applying a simulated or real load, or a combination of simulated or real loads, equivalent to a resistive load greater than 7 kW. The inverter input voltage will be no more than 210 Vdc during the test.	Each 72-hour inverter supplies a line-to-line output voltage of 208 ± 2% V at a frequency of 60 ± 0.5% Hz.
		4.h) Each IDS 24-hour battery charger provides the PMS with two loss-of-ac input voltage signals.	Testing will be performed by simulating a loss of input voltage to each 24-hour battery charger.	Two PMS input signals exist from each 24-hour battery charger indicating loss of ac input voltage when the loss-of-input voltage condition is simulated.
		5.a) Each IDS 24-hour battery charger supplies a dc switchboard bus load while maintaining the corresponding battery charged.	Testing of each as-built 24-hour battery charger will be performed by applying a simulated or real load, or a combination of simulated or real loads.	Each 24-hour battery charger provides an output current of at least 150 A with an output voltage in the range 210 to 280 V.

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		5.b) Each IDS 72-hour battery charger supplies a dc switchboard bus load while maintaining the corresponding battery charged.	Testing of each 72-hour as-built battery charger will be performed by applying a simulated or real load, or a combination of simulated or real loads.	Each 72-hour battery charger provides an output current of at least 125 A with an output voltage in the range 210 to 280 V.
		5.c) Each IDS regulating transformer supplies an ac load when powered from the 480 V MCC.	Testing of each as-built regulating transformer will be performed by applying a simulated or real load, or a combination of simulated or real loads, equivalent to a resistive load greater than 30 kW when powered from the 480 V MCC.	Each regulating transformer supplies a line-to-line output voltage of 208 ± 2% V.
		6. Safety-related displays identified in Table 2.6.3-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.6.3-1 can be retrieved in the MCR.
		11. Displays of the parameters identified in Table 2.6.3-2 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.6.3-2 in the MCR.	Displays identified in Table 2.6.3-2 can be retrieved in the MCR.
604	2.6.03.04d	Not used per Amendment No. XX ~~4.d) Each IDS 72-hour battery bank supplies a dc switchboard bus load for a period of 72 hours without recharging.~~	~~Testing of each 72-hour as-built battery bank will be performed by applying a simulated or real load, or a combination of simulated or real loads which envelope the battery bank design duty cycle. The test will be conducted on a battery bank that has been fully charged and has been connected to a battery charger maintained at 270±2 V for a period of no less than 24 hours prior to the test.~~	~~The battery terminal voltage is greater than or equal to 210 V after a period of no less than 72 hours with an equivalent load that equals or exceeds the battery bank design duty cycle capacity.~~

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
605	2.6.03.04e	Not used per Amendment No. XX ~~4.e) The IDS spare battery bank supplies a dc load equal to or greater than the most severe switchboard bus load for the required period without recharging.~~	~~Testing of the as-built spare battery bank will be performed by applying a simulated or real load, or a combination of simulated or real loads which envelope the most severe of the division batteries design duty cycle. The test will be conducted on a battery bank that has been fully charged and has been connected to a battery charger maintained at 270±2 V for a period of no less than 24 hours prior to the test.~~	~~The battery terminal voltage is greater than or equal to 210 V after a period with a load and duration that equals or exceeds the most severe battery bank design duty cycle capacity.~~
606	2.6.03.04f	Not used per Amendment No. XX ~~4.f) Each IDS 24-hour inverter supplies its ac load.~~	~~Testing of each 24-hour as-built inverter will be performed by applying a simulated or real load, or a combination of simulated or real loads, equivalent to a resistive load greater than 12 kW. The inverter input voltage will be no more than 210 Vdc during the test.~~	~~Each 24-hour inverter supplies a line-to-line output voltage of 208 ± 2% V at a frequency of 60 ± 0.5% Hz.~~
607	2.6.03.04g	Not used per Amendment No. XX ~~4.g) Each IDS 72-hour inverter supplies its ac load.~~	~~Testing of each 72-hour as-built inverter will be performed by applying a simulated or real load, or a combination of simulated or real loads, equivalent to a resistive load greater than 7 kW. The inverter input voltage will be no more than 210 Vdc during the test.~~	~~Each 72-hour inverter supplies a line-to-line output voltage of 208 ± 2% V at a frequency of 60 ± 0.5% Hz.~~
608	2.6.03.04h	Not used per Amendment No. XX ~~4.h) Each IDS 24-hour battery charger provides the PMS with two loss-of-ac input voltage signals.~~	~~Testing will be performed by simulating a loss of input voltage to each 24-hour battery charger.~~	~~Two PMS input signals exist from each 24-hour battery charger indicating loss of ac input voltage when the loss-of-input voltage condition is simulated.~~

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Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
610	2.6.03.05a	Not used per Amendment No. XX ~~5.a) Each IDS 24-hour battery charger supplies a dc switchboard bus load while maintaining the corresponding battery charged.~~	~~Testing of each as-built 24-hour battery charger will be performed by applying a simulated or real load, or a combination of simulated or real loads.~~	~~Each 24-hour battery charger provides an output current of at least 150 A with an output voltage in the range 210 to 280 V.~~
611	2.6.03.05b	Not used per Amendment No. XX ~~5.b) Each IDS 72-hour battery charger supplies a dc switchboard bus load while maintaining the corresponding battery charged.~~	~~Testing of each 72-hour as-built battery charger will be performed by applying a simulated or real load, or a combination of simulated or real loads.~~	~~Each 72-hour battery charger provides an output current of at least 125 A with an output voltage in the range 210 to 280 V.~~
612	2.6.03.05c	Not used per Amendment No. XX ~~5.c) Each IDS regulating transformer supplies an ac load when powered from the 480 V MCC.~~	~~Testing of each as-built regulating transformer will be performed by applying a simulated or real load, or a combination of simulated or real loads, equivalent to a resistive load greater than 30 kW when powered from the 480 V MCC.~~	~~Each regulating transformer supplies a line-to-line output voltage of 208 ± 2% V.~~
613	2.6.03.05d.i	5.d) The IDS Divisions B and C regulating transformers supply their post-72-hour ac loads when powered from an ancillary diesel generator.	Inspection of the as-built system will be performed. Inspection of the as-built system will be performed.	i) Ancillary diesel generator 1 is electrically connected to regulating transformer IDSC-DT-1 ii) Ancillary diesel generator 2 is electrically connected to regulating transformer IDSB-DT-1.
614	2.6.03.05d.ii	Not used per Amendment No. XX ~~5.d) The IDS Divisions B and C regulating transformers supply their post-72-hour ac loads when powered from an ancillary diesel generator.~~	~~Inspection of the as-built system will be performed.~~	~~ii) Ancillary diesel generator 2 is electrically connected to regulating transformer IDSB-DT-1.~~
615	2.6.03.06	Not used per Amendment No. XX ~~6. Safety-related displays identified in Table 2.6.3-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.6.3-1 can be retrieved in the MCR.~~

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
616	2.6.03.07	7. The IDS dc battery fuses and battery charger circuit breakers, and dc distribution panels, MCCs, and their circuit breakers and fuses, are sized to supply their load requirements.	Analyses for the as-built IDS dc electrical distribution system to determine the capacities of the battery fuses and battery charger circuit breakers, and dc distribution panels, MCCs, and their circuit breakers and fuses, will be performed.	Analyses for the as-built IDS dc electrical distribution system exist and conclude that the capacities of as-built IDS battery fuses and battery charger circuit breakers, and dc distribution panels, MCCs, and their circuit breakers and fuses, as determined by their nameplate ratings, exceed their analyzed load requirements.
		8. Circuit breakers and fuses in IDS battery, battery charger, dc distribution panel, and MCC circuits are rated to interrupt fault currents.	Analyses for the as-built IDS dc electrical distribution system to determine fault currents will be performed.	Analyses for the as-built IDS dc electrical distribution system exist and conclude that the analyzed fault currents do not exceed the interrupt capacity of circuit breakers and fuses in the battery, battery charger, dc distribution panel, and MCC circuits, as determined by their nameplate ratings.
		9. The IDS batteries, battery chargers, dc distribution panels, and MCCs are rated to withstand fault currents for the time required to clear the fault from its power source.	Analyses for the as-built IDS dc electrical distribution system to determine fault currents will be performed.	Analyses for the as-built IDS dc electrical distribution system exist and conclude that the fault current capacities of as-built IDS batteries, battery chargers, dc distribution panels, and MCCs, as determined by manufacturer's ratings, exceed their analyzed fault currents for the time required to clear the fault from its power source as determined by the circuit interrupting device coordination analyses.

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		10. The IDS electrical distribution system cables are rated to withstand fault currents for the time required to clear the fault from its power source.	Analyses for the as-built IDS dc electrical distribution system to determine fault currents will be performed.	Analyses for the as-built IDS dc electrical distribution system exist and conclude that the IDS dc electrical distribution system cables will withstand the analyzed fault currents, as determined by manufacturer's ratings, for the time required to clear the fault from its power source as determined by the circuit interrupting device coordination analyses.
617	2.6.03.08	Not used per Amendment No. XX ~~8. Circuit breakers and fuses in IDS battery, battery charger, dc distribution panel, and MCC circuits are rated to interrupt fault currents.~~	~~Analyses for the as-built IDS dc electrical distribution system to determine fault currents will be performed.~~	~~Analyses for the as-built IDS dc electrical distribution system exist and conclude that the analyzed fault currents do not exceed the interrupt capacity of circuit breakers and fuses in the battery, battery charger, dc distribution panel, and MCC circuits, as determined by their nameplate ratings.~~
618	2.6.03.09	Not used per Amendment No. XX ~~9. The IDS batteries, battery chargers, dc distribution panels, and MCCs are rated to withstand fault currents for the time required to clear the fault from its power source.~~	~~Analyses for the as-built IDS dc electrical distribution system to determine fault currents will be performed.~~	~~Analyses for the as-built IDS dc electrical distribution system exist and conclude that the fault current capacities of as-built IDS batteries, battery chargers, dc distribution panels, and MCCs, as determined by manufacturer's ratings, exceed their analyzed fault currents for the time required to clear the fault from its power source as determined by the circuit interrupting device coordination analyses.~~

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
619	2.6.03.10	Not used per Amendment No. XX ~~10. The IDS electrical distribution system cables are rated to withstand fault currents for the time required to clear the fault from its power source.~~	~~Analyses for the as-built IDS dc electrical distribution system to determine fault currents will be performed.~~	~~Analyses for the as-built IDS dc electrical distribution system exist and conclude that the IDS dc electrical distribution system cables will withstand the analyzed fault currents, as determined by manufacturer's ratings, for the time required to clear the fault from its power source as determined by the circuit interrupting device coordination analyses.~~
620	2.6.03.11	Not used per Amendment No. XX ~~11. Displays of the parameters identified in Table 2.6.3-2 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the displays identified in Table 2.6.3-2 in the MCR.~~	~~Displays identified in Table 2.6.3-2 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), 2.6.4, Onsite Standby Power System, Table 2.6.4-1 as shown below:

Table 2.6.4-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
622	2.6.04.02a	2.a) On loss of power to a 6900 volt diesel-backed bus, the associated diesel generator automatically starts and produces ac power at rated voltage and frequency. The source circuit breakers and bus load circuit breakers are opened, and the generator is connected to the bus. 2.b) Each diesel generator unit is sized to supply power to the selected nonsafety-related electrical components. 3. Displays of diesel generator status (running/not running) and electrical output power (watts) can be retrieved in the MCR. 4. Controls exist in the MCR to start and stop each diesel generator.	Tests on the as-built ZOS system will be conducted by providing a simulated loss-of-voltage signal. The starting air supply receiver will not be replenished during the test. Each diesel generator will be operated with a load of 4000 kW or greater and a power factor between 0.9 and 1.0 for a time period required to reach engine temperature equilibrium plus 2.5 hours. Inspection will be performed for retrievability of the displays in the MCR. A test will be performed to verify that controls in the MCR can start and stop each diesel generator.	Each as-built diesel generator automatically starts on receiving a simulated loss-of-voltage signal and attains a voltage of 6900 ± 10% V and frequency 60 ± 5% Hz after the start signal is initiated and opens ac power system breakers on the associated 6900 V bus. Each diesel generator provides power to the load with a generator terminal voltage of 6900 ± 10% V and a frequency of 60 + 5% Hz. Displays of diesel generator status and electrical output power can be retrieved in the MCR. Controls in the MCR operate to start and stop each diesel generator.
623	2.6.04.02b	Not used per Amendment No. XX ~~2.b) Each diesel generator unit is sized to supply power to the selected nonsafety-related electrical components.~~	~~Each diesel generator will be operated with a load of 4000 kW or greater and a power factor between 0.9 and 1.0 for a time period required to reach engine temperature equilibrium plus 2.5 hours.~~	~~Each diesel generator provides power to the load with a generator terminal voltage of 6900 ± 10% V and a frequency of 60 ± 5% Hz.~~
***				
625	2.6.04.03	Not used per Amendment No. XX ~~3. Displays of diesel generator status (running/not running) and electrical output power (watts) can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the displays in the MCR.~~	~~Displays of diesel generator status and electrical output power can be retrieved in the MCR.~~

Table 2.6.4-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
626	2.6.04.04	Not used per Amendment No. XX ~~4. Controls exist in the MCR to start and stop each diesel generator.~~	~~A test will be performed to verify that controls in the MCR can start and stop each diesel generator.~~	~~Controls in the MCR operate to start and stop each diesel generator.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.5, Lighting System, Table 2.6.5-1 as shown below:

Table 2.6.5-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
628	2.6.05.02.i	Not used per Amendment No. XX ~~2. The ELS has six groups of emergency lighting fixtures located in the MCR and at the RSW. Each group is powered by one of the Class 1E inverters. The ELS has four groups of panel lighting fixtures located on or near safety panels in the MCR. Each group is powered by one of the Class 1E inverters in Divisions B and C (one 24-hour and one 72-hour inverter in each Division).~~	~~i) Inspection of the as-built system will be performed.~~	~~i) The as-built ELS has six groups of emergency lighting fixtures located in the MCR and at the RSW. The ELS has four groups of panel lighting fixtures located on or near safety panels in the MCR.~~
629	2.6.05.02.ii	2. The ELS has six groups of emergency lighting fixtures located in the MCR and at the RSW. Each group is powered by one of the Class 1E inverters. The ELS has four groups of panel lighting fixtures located on or near safety panels in the MCR. Each group is powered by one of the Class 1E inverters in Divisions B and C (one 24-hour and one 72-hour inverter in each Division).	i) Inspection of the as-built system will be performed.	i) The as-built ELS has six groups of emergency lighting fixtures located in the MCR and at the RSW. The ELS has four groups of panel lighting fixtures located on or near safety panels in the MCR.
			ii) Testing of the as-built system will be performed using one Class 1E inverter at a time.	ii) Each of the six as-built emergency lighting groups is supplied power from its respective Class 1E inverter and each of the four as-built panel lighting groups is supplied power from its respective Class 1E inverter.
		5. The normal lighting can provide 50 foot candles at the safety panel and at the workstations in the MCR and at the RSW.	i) Testing of the as-built normal lighting in the MCR will be performed.	i) When adjusted for maximum illumination and powered by the main ac power system, the normal lighting in the MCR provides at least 50 foot candles at the safety panel and at the workstations.

Table 2.6.5-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
			ii) Testing of the as-built normal lighting at the RSW will be performed.	ii) When adjusted for maximum illumination and powered by the main ac power system, the normal lighting in the RSW provides at least 50 foot candles at the safety panel and at the workstations.
		6. The emergency lighting can provide 10 foot candles at the safety panel and at the workstations in the MCR and at the RSW.	i) Testing of the as-built emergency lighting in the MCR will be performed.	i) When adjusted for maximum illumination and powered by the six Class 1E inverters, the emergency lighting in the MCR provides at least 10 foot candles at the safety panel and at the workstations.
			ii) Testing of the as-built emergency lighting at the RSW will be performed.	ii) When adjusted for maximum illumination and powered by the six Class 1E inverters, the emergency lighting provides at least 10 foot candles at the RSW.
***				
633	2.6.05.05.i	Not used per Amendment No. XX ~~5. The normal lighting can provide 50 foot candles at the safety panel and at the workstations in the MCR and at the RSW.~~	~~i) Testing of the as-built normal lighting in the MCR will be performed.~~	~~i) When adjusted for maximum illumination and powered by the main ac power system, the normal lighting in the MCR provides at least 50 foot candles at the safety panel and at the workstations.~~
634	2.6.05.05.ii	Not used per Amendment No. XX ~~5. The normal lighting can provide 50 foot candles at the safety panel and at the workstations in the MCR and at the RSW.~~	~~ii) Testing of the as-built normal lighting at the RSW will be performed.~~	~~ii) When adjusted for maximum illumination and powered by the main ac power system, the normal lighting in the RSW provides at least 50 foot candles at the safety panel and at the workstations.~~

Table 2.6.5-1 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
635	2.6.05.06.i	Not used per Amendment No. XX ~~6. The emergency lighting can provide 10 foot candles at the safety panel and at the workstations in the MCR and at the RSW.~~	~~i) Testing of the as-built emergency lighting in the MCR will be performed.~~	~~i) When adjusted for maximum illumination and powered by the six Class 1E inverters, the emergency lighting in the MCR provides at least 10 foot candles at the safety panel and at the workstations.~~
636	2.6.05.06.ii	Not used per Amendment No. XX ~~6. The emergency lighting can provide 10 foot candles at the safety panel and at the workstations in the MCR and at the RSW.~~	~~ii) Testing of the as-built emergency lighting at the RSW will be performed.~~	~~ii) When adjusted for maximum illumination and powered by the six Class 1E inverters, the emergency lighting provides at least 10 foot candles at the RSW.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.6, Grounding and Lightning Protection System, Table 2.6.6-1 as shown below:

*Note: Changes to ITAAC Nos. 2.6.06.01.i, 2.6.06.01.ii, 2.6.06.01.iii and 2.6.06.01.iv in Table 2.6.6-1, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 1.i, 1ii, 1.iii and 1.iv in Table 2.6.6-1 is needed.

Table 2.6.6-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
637	2.6.06.01.i	1. The EGS provides an electrical grounding system for: (1) instrument/computer grounding; (2) electrical system grounding of the neutral points of the main generator, main step-up transformers, auxiliary transformers, load center transformers, auxiliary and onsite standby diesel generators; and (3) equipment grounding of equipment enclosures, metal structures, metallic tanks, ground bus of switchgear assemblies, load centers, motor control centers, and control cabinets. Lightning protection is provided for exposed structures and buildings housing safety-related and fire protection equipment. Each grounding system and lightning protection system is grounded to the station grounding grid.	i) An inspection for the instrument/computer grounding system connection to the station grounding grid will be performed.	i) A connection exists between the instrument/computer grounding system and the station grounding grid.
			ii) An inspection for the electrical system grounding connection to the station grounding grid will be performed.	ii) A connection exists between the electrical system grounding and the station grounding grid.
			iii) An inspection for the equipment grounding system connection to the station grounding grid will be performed.	iii) A connection exists between the equipment grounding system and the station grounding grid.
			iv) An inspection for the lightning protection system connection to the station grounding grid will be performed.	iv) A connection exists between the lightning protection system and the station grounding grid.

Table 2.6.6-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
638	2.6.06.01.ii	Not used per Amendment No. XX ~~1. The EGS provides an electrical grounding system for: (1) instrument/computer grounding; (2) electrical system grounding of the neutral points of the main generator, main step-up transformers, auxiliary transformers, load center transformers, auxiliary and onsite standby diesel generators; and (3) equipment grounding of equipment enclosures, metal structures, metallic tanks, ground bus of switchgear assemblies, load centers, motor control centers, and control cabinets. Lightning protection is provided for exposed structures and buildings housing safety-related and fire protection equipment. Each grounding system and lightning protection system is grounded to the station grounding grid.~~	~~ii) An inspection for the electrical system grounding connection to the station grounding grid will be performed.~~	~~ii) A connection exists between the electrical system grounding and the station grounding grid.~~
639	2.6.06.01.iii	Not used per Amendment No. XX ~~1. The EGS provides an electrical grounding system for: (1) instrument/computer grounding; (2) electrical system grounding of the neutral points of the main generator, main step-up transformers, auxiliary transformers, load center transformers, auxiliary and onsite standby diesel generators; and (3) equipment grounding of equipment enclosures, metal structures, metallic tanks, ground bus of switchgear assemblies, load centers, motor control centers, and control cabinets. Lightning protection is provided for exposed structures and buildings housing safety-related and fire protection equipment. Each grounding system and lightning protection system is grounded to the station grounding grid.~~	~~iii) An inspection for the equipment grounding system connection to the station grounding grid will be performed.~~	~~iii) A connection exists between the equipment grounding system and the station grounding grid.~~

Table 2.6.6-1 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
640	2.6.06.01.iv	Not used per Amendment No. XX ~~1. The EGS provides an electrical grounding system for: (1) instrument/computer grounding; (2) electrical system grounding of the neutral points of the main generator, main step-up transformers, auxiliary transformers, load center transformers, auxiliary and onsite standby diesel generators; and (3) equipment grounding of equipment enclosures, metal structures, metallic tanks, ground bus of switchgear assemblies, load centers, motor control centers, and control cabinets. Lightning protection is provided for exposed structures and buildings housing safety-related and fire protection equipment. Each grounding system and lightning protection system is grounded to the station grounding grid.~~	~~iv) An inspection for the lightning protection system connection to the station grounding grid will be performed.~~	~~iv) A connection exists between the lightning protection system and the station grounding grid.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.9, Plant Security System, Table 2.6.9-1 as shown below:

Table 2.6.9-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		***		
644	2.6.09.05a	5.a) Security alarm annunciation and video assessment information is displayed concurrently in the central alarm station and the secondary alarm station, and the video image recording with real time playback capability can provide assessment of activities before and after each alarm annunciation within the perimeter area barrier. 15.b) Intrusion detection and assessment systems concurrently provide visual displays and audible annunciation of alarms in the central and secondary alarm stations.	Test, inspection, or a combination of test and inspections of the installed systems will be performed. Tests will be performed on intrusion detection and assessment equipment.	Security alarm annunciation and video assessment information is displayed concurrently in the central alarm station and the secondary alarm station, and the video image recording with real time playback capability provides assessment of activities before and after alarm annunciation within the perimeter barrier. The intrusion detection system concurrently provides visual displays and audible annunciations of alarms in both the central and secondary alarm stations.
		***		
648	2.6.09.07a	Not used per Amendment No. XX ~~7.a) Vital equipment is located only within a vital area.~~	~~Inspection will be performed to confirm that vital equipment is located within a vital area.~~	~~All vital equipment is located only within a vital area.~~
649	2.6.09.07b	Not used per Amendment No. XX ~~7.b) Access to vital equipment requires passage through the vital area barrier.~~	~~Inspection will be performed to confirm that access to vital equipment requires passage through the vital area barrier.~~	~~Vital equipment is located within a protected area such that access to vital equipment requires passage through the vital area barrier.~~
		***		
651	2.6.09.09	Not used per Amendment No. XX ~~9. Emergency exits through the vital area boundaries are locked, alarmed, and equipped with a crash bar to allow for emergency egress.~~	~~Test, inspection, or a combination of tests and inspections of the emergency exits through the vital area boundaries will be performed.~~	~~The emergency exits through the vital area boundaries are locked, alarmed, and equipped with a crash bar to allow for emergency egress.~~
		***		

Table 2.6.9-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
652	2.6.09.13a	13.a) The central and secondary alarm stations have conventional (landline) telephone service with the main control room and local law enforcement authorities. 13.b) The central and secondary alarm stations are capable of continuous communication with security personnel.	Tests, inspections, or a combination of tests and inspections of the central and secondary alarm stations' conventional telephone services will be performed. Tests, inspections, or a combination of tests and inspections of the central and secondary alarm stations' continuous communication capabilities will be performed.	The central and secondary alarm stations are equipped with conventional (landline) telephone service with the main control room and local law enforcement authorities. The central and secondary alarm stations are equipped with the capability to continuously communicate with security officers, watchmen, armed response individuals, or any security personnel that have responsibilities during a contingency event.
653	2.6.09.13b	Not used per Amendment No. XX ~~13.b) The central and secondary alarm stations are capable of continuous communication with security personnel.~~	~~Tests, inspections, or a combination of tests and inspections of the central and secondary alarm stations' continuous communication capabilities will be performed.~~	~~The central and secondary alarm stations are equipped with the capability to continuously communicate with security officers, watchmen, armed response individuals, or any security personnel that have responsibilities during a contingency event.~~
***				
655	2.6.09.15a	15.a) Security alarm devices, including transmission lines to annunciators, are tamper indicating and self-checking (e.g., an automatic indication is provided when failure of the alarm system or a component occurs, or when on standby power). Alarm annunciation shall indicate the type of alarm (e.g., intrusion alarms and emergency exit alarm) and location.	A test will be performed to verify that security alarms, including transmission lines to annunciators, are tamper indicating and self-checking (e.g., an automatic indication is provided when failure of the alarm system or a component occurs, or when on standby power) and that alarm annunciation indicates the type of alarm (e.g., intrusion alarms and emergency exit alarms) and location.	A report exists and concludes that security alarm devices, including transmission lines to annunciators, are tamper indicating and self-checking (e.g., an automatic indication is provided when failure of the alarm system or a component occurs, or when the system is on standby power) and that alarm annunciation indicates the type of alarm (e.g., intrusion alarms and emergency exit alarms) and location.

Table 2.6.9-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		16. Equipment exists to record onsite security alarm annunciation, including the location of the alarm, false alarm, alarm check, and tamper indication; and the type of alarm, location, alarm circuit, date, and time.	Test, analysis, or a combination of test and analysis will be performed to ensure that equipment is capable of recording each onsite security alarm annunciation, including the location of the alarm, false alarm, alarm check, and tamper indication; and the type of alarm, location, alarm circuit, date, and time.	A report exists and concludes that equipment is capable of recording each onsite security alarm annunciation, including the location of the alarm, false alarm, alarm check, and tamper indication; and the type of alarm, location, alarm circuit, date, and time.
656	2.6.09.15b	Not used per Amendment No. XX ~~15.b) Intrusion detection and assessment systems concurrently provide visual displays and audible annunciation of alarms in the central and secondary alarm stations.~~	~~Tests will be performed on intrusion detection and assessment equipment.~~	~~The intrusion detection system concurrently provides visual displays and audible annunciations of alarms in both the central and secondary alarm stations.~~
657	2.6.09.16	Not used per Amendment No. XX ~~16. Equipment exists to record onsite security alarm annunciation, including the location of the alarm, false alarm, alarm check, and tamper indication; and the type of alarm, location, alarm circuit, date, and time.~~	~~Test, analysis, or a combination of test and analysis will be performed to ensure that equipment is capable of recording each onsite security alarm annunciation, including the location of the alarm, false alarm, alarm check, and tamper indication; and the type of alarm, location, alarm circuit, date, and time.~~	~~A report exists and concludes that equipment is capable of recording each onsite security alarm annunciation, including the location of the alarm, false alarm, alarm check, and tamper indication; and the type of alarm, location, alarm circuit, date, and time.~~

Revise COL Appendix C Section C.2.6.9, Physical Security, Table C.2.6.9-2 as shown below:

Table C.2.6.9-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
661	C.2.6.09.03b	3.b) The isolation zones are monitored with intrusion detection equipment that provides the capability to detect and assess unauthorized persons.	Inspections will be performed of the intrusion detection equipment within the isolation zones.	The isolation zones are equipped with intrusion detection equipment that provides the capability to detect and assess unauthorized persons.
		4. The intrusion detection and assessment equipment at the protected area perimeter: a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in both the Central Alarm Station and Secondary Alarm Station;	Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.	The intrusion detection and assessment equipment at the protected area perimeter: a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in the Central Alarm Station and Secondary Alarm Station;
		b) remains operable from an uninterruptible power supply in the event of the loss of normal power.	Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.	b) remains operable from an uninterruptible power supply in the event of the loss of normal power.
662	C.2.6.09.04a	Not used per Amendment No. XX ~~4. The intrusion detection and assessment equipment at the protected area perimeter:~~ ~~a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in both the Central Alarm Station and Secondary Alarm Station;~~	~~Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.~~	~~The intrusion detection and assessment equipment at the protected area perimeter:~~ ~~a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in the Central Alarm Station and Secondary Alarm Station;~~
663	C.2.6.09.04b	Not used per Amendment No. XX ~~4. The intrusion detection and assessment equipment at the protected area perimeter:~~ ~~b) remains operable from an uninterruptible power supply in the event of the loss of normal power.~~	~~Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.~~	~~The intrusion detection and assessment equipment at the protected area perimeter:~~ ~~b) remains operable from an uninterruptible power supply in the event of the loss of normal power.~~

Table C.2.6.9-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
664	C.2.6.09.05a	5. Access control points are established to: a) control personnel and vehicle access into the protected area. b) detect firearms, explosives, and incendiary devices at the protected area personnel access points.	Tests, inspections, or combination of tests and inspections of installed systems and equipment at the access control points to the protected area will be performed. Tests, inspections, or combination of tests and inspections of installed systems and equipment at the access control points to the protected area will be performed.	The access control points for the protected area: a) are configured to control personnel and vehicle access. b) include detection equipment that is capable of detecting firearms, incendiary devices, and explosives at the protected area personnel access points.
665	C.2.6.09.05b	Not used per Amendment No. XX ~~5. Access control points are established to:~~ ~~b) detect firearms, explosives, and incendiary devices at the protected area personnel access points.~~	~~Tests, inspections, or combination of tests and inspections of installed systems and equipment at the access control points to the protected area will be performed.~~	~~The access control points for the protected area:~~ ~~b) include detection equipment that is capable of detecting firearms, incendiary devices, and explosives at the protected area personnel access points.~~
***				
667	C.2.6.09.07	7. Access to vital equipment physical barriers requires passage through the protected area perimeter barrier. 7.a) Vital equipment is located only within a vital area. 7.b) Access to vital equipment requires passage through the vital area barrier.	Inspection will be performed to confirm that access to vital equipment physical barriers requires passage through the protected area perimeter barrier. Inspection will be performed to confirm that vital equipment is located within a vital area. Inspection will be performed to confirm that access to vital equipment requires passage through the vital area barrier.	Vital equipment is located within a protected area such that access to vital equipment physical barriers requires passage through the protected area perimeter barrier. All vital equipment is located only within a vital area. Vital equipment is located within a protected area such that access to vital equipment requires passage through the vital area barrier.

Table C.2.6.9-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
668	C.2.6.09.08a	8.a) Penetrations through the protected area barrier are secured and monitored. 8.b) Unattended openings (such as underground pathways) that intersect the protected area boundary or vital area boundary will be protected by a physical barrier and monitored by intrusion detection equipment or provided surveillance at a frequency sufficient to detect exploitation.	Inspections will be performed of penetrations through the protected area barrier. Inspections will be performed of unattended openings that intersect the protected area boundary or vital area boundary.	Penetrations and openings through the protected area barrier are secured and monitored. Unattended openings (such as underground pathways) that intersect the protected area boundary or vital area boundary are protected by a physical barrier and monitored by intrusion detection equipment or provided surveillance at a frequency sufficient to detect exploitation.
669	C.2.6.09.08b	Not used per Amendment No. XX ~~8.b) Unattended openings (such as underground pathways) that intersect the protected area boundary or vital area boundary will be protected by a physical barrier and monitored by intrusion detection equipment or provided surveillance at a frequency sufficient to detect exploitation.~~	~~Inspections will be performed of unattended openings that intersect the protected area boundary or vital area boundary.~~	~~Unattended openings (such as underground pathways) that intersect the protected area boundary or vital area boundary are protected by a physical barrier and monitored by intrusion detection equipment or provided surveillance at a frequency sufficient to detect exploitation.~~
670	C.2.6.09.09	9. Emergency exits through the protected area perimeter are alarmed and secured with locking devices to allow for emergency egress. 9. Emergency exits through the vital area boundaries are locked, alarmed, and equipped with a crash bar to allow for emergency egress.	Tests, inspections, or a combination of tests and inspections of emergency exits through the protected area perimeter will be performed. Test, inspection, or a combination of tests and inspections of the emergency exits through the vital area boundaries will be performed.	Emergency exits through the protected area perimeter are alarmed and secured by locking devices that allow prompt egress during an emergency. The emergency exits through the vital area boundaries are locked, alarmed, and equipped with a crash bar to allow for emergency egress.

Revise COL Appendix C (and plant-specific Tier 1), Section 2.7.1, Nuclear Island Nonradioactive Ventilation System, Table 2.7.1-4 as shown below:

Table 2.7.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		***		
693	2.7.01.08d	Not used per Amendment No. XX ~~8.d) The VBS provides ventilation cooling via the ancillary equipment in Table 2.7.1-3 to the MCR and the division B&C Class 1E I&C rooms.~~	~~Testing will be performed on the components in Table 2.7.1-3.~~	~~The fans start and run.~~
694	2.7.01.09	Not used per Amendment No. XX ~~9. Safety-related displays identified in Table 2.7.1-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the safety-related displays in the MCR.~~	~~Safety-related displays identified in Table 2.7.1-1 can be retrieved in the MCR.~~
695	2.7.01.10a	Not used per Amendment No. XX ~~10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.7.1-1 to perform their active functions.~~	~~Stroke testing will be performed on the remotely operated valves identified in Table 2.7.1-1 using the controls in the MCR.~~	~~Controls in the MCR operate to cause the remotely operated valves identified in Table 2.7.1-1 to perform their active functions.~~
696	2.7.01.10b	Not used per Amendment No. XX ~~10.b) The valves identified in Table 2.7.1-1 as having PMS control perform their active safety function after receiving a signal from the PMS.~~	~~Testing will be performed using real or simulated signals into the PMS.~~	~~The valves identified in Table 2.7.1-1 as having PMS control perform their active safety function after receiving a signal from PMS.~~
697	2.7.01.11	Not used per Amendment No. XX ~~11. After loss of motive power, the remotely operated valves identified in Table 2.7.1-1 assume the indicated loss of motive power position.~~	~~Testing of the remotely operated valves will be performed under the conditions of loss of motive power.~~	~~Upon loss of motive power, each remotely operated valves identified in Table 2.7.1-1 assumes the indicated loss of motive power position.~~
698	2.7.01.12	Not used per Amendment No. XX ~~12. Controls exist in the MCR to cause the components identified in Table 2.7.1-3 to perform the listed function.~~	~~Testing will be performed on the components in Table 2.7.1-3 using controls in the MCR.~~	~~Controls in the MCR operate to cause the components listed in Table 2.7.1-3 to perform the listed functions.~~
699	2.7.01.13	Not used per Amendment No. XX ~~13. Displays of the parameters identified in Table 2.7.1-3 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the parameters in the MCR.~~	~~The displays identified in Table 2.7.1-3 can be retrieved in the MCR.~~

Table 2.7.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
700	2.7.01.14	8.d) The VBS provides ventilation cooling via the ancillary equipment in Table 2.7.1-3 to the MCR and the division B&C Class 1E I&C rooms.	Testing will be performed on the components in Table 2.7.1-3.	The fans start and run.
		9. Safety-related displays identified in Table 2.7.1-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.7.1-1 can be retrieved in the MCR.
		10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.7.1-1 to perform their active functions.	Stroke testing will be performed on the remotely operated valves identified in Table 2.7.1-1 using the controls in the MCR.	Controls in the MCR operate to cause the remotely operated valves identified in Table 2.7.1-1 to perform their active functions.
		10.b) The valves identified in Table 2.7.1-1 as having PMS control perform their active safety function after receiving a signal from the PMS.	Testing will be performed using real or simulated signals into the PMS.	The valves identified in Table 2.7.1-1 as having PMS control perform their active safety function after receiving a signal from PMS.
		11. After loss of motive power, the remotely operated valves identified in Table 2.7.1-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	Upon loss of motive power, each remotely operated valves identified in Table 2.7.1-1 assumes the indicated loss of motive power position.
		12. Controls exist in the MCR to cause the components identified in Table 2.7.1-3 to perform the listed function.	Testing will be performed on the components in Table 2.7.1-3 using controls in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.7.1-3 to perform the listed functions.
		13. Displays of the parameters identified in Table 2.7.1-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.7.1-3 can be retrieved in the MCR.
		14. The background noise level in the MCR and RSR does not exceed 65 dB(A) when the VBS is operating.	The as-built VBS will be operated, and background noise levels in the MCR and RSR will be measured.	The background noise level in the MCR and RSR does not exceed 65 dB(A) when the VBS is operating.

Revise COL Appendix C (and plant-specific Tier 1) Section 2.7.2, Central Chilled Water System Table 2.7.2-2 as shown below:

Table 2.7.2-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
703	2.7.02.03a	3.a) The VWS provides chilled water to the supply air handling units serving the MCR, the Class 1E electrical rooms, and the unit coolers serving the RNS and CVS pump rooms. 4. Controls exist in the MCR to cause the components identified in Table 2.7.2-1 to perform the listed function. 5. Displays of the parameters identified in Table 2.7.2-1 can be retrieved in the MCR.	Testing will be performed by measuring the flow rates to the chilled water cooling coils. Testing will be performed on the components in Table 2.7.2-1 using controls in the MCR. Inspection will be performed for retrievability of parameters in the MCR.	The water flow to each cooling coil equals or exceeds the following: Coil / Flow (gpm) VBS MY C01A/B 96 VBS MY C02A/C 97 VBS MY C02B/D 52 VAS MY C07A/B 12.3 VAS MY C12A/B 8.2 VAS MY C06A/B 8.2 Controls in the MCR operate to cause the components listed in Table 2.7.2-1 to perform the listed functions. The displays identified in Table 2.7.2-1 can be retrieved in the MCR.
***				
705	2.7.02.04	Not used per Amendment No. XX~~4. Controls exist in the MCR to cause the components identified in Table 2.7.2-1 to perform the listed function.~~	~~Testing will be performed on the components in Table 2.7.2-1 using controls in the MCR.~~	~~Controls in the MCR operate to cause the components listed in Table 2.7.2-1 to perform the listed functions.~~
706	2.7.02.05	Not used per Amendment No. XX ~~5. Displays of the parameters identified in Table 2.7.2-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of parameters in the MCR.~~	~~The displays identified in Table 2.7.2-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.7.3, Annex/Auxiliary Building Nonradioactive Ventilation System, Table 2.7.3-2 as shown below:

Table 2.7.3-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
710	2.7.03.03	3. Controls exist in the MCR to cause the components identified in Table 2.7.3-1 to perform the listed function. 4. Displays of the parameters identified in Table 2.7.3-1 can be retrieved in the MCR.	Testing will be performed on the components in Table 2.7.3-1 using controls in the MCR. Inspection will be performed for retrievability of the parameters in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.7.3-1 to perform the listed functions. The displays identified in Table 2.7.3-1 can be retrieved in the MCR.
711	2.7.03.04	Not used per Amendment No. XX ~~4. Displays of the parameters identified in Table 2.7.3-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the parameters in the MCR.~~	~~The displays identified in Table 2.7.3-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.7.4, Diesel Generator Building Ventilation System, Table 2.7.4-2 as shown below:

Table 2.7.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
716	2.7.04.03	3. Controls exist in the MCR to cause the components identified in Table 2.7.4-1 to perform the listed function. 4. Displays of the parameters identified in Table 2.7.4-1 can be retrieved in the MCR.	Testing will be performed on the components in Table 2.7.4-1 using controls in the MCR. Inspection will be performed for retrievability of the parameters in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.7.4-1 to perform the listed functions. The displays identified in Table 2.7.4-1 can be retrieved in the MCR.
717	2.7.04.04	Not used per Amendment No. XX ~~4. Displays of the parameters identified in Table 2.7.4-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the parameters in the MCR.~~	~~The displays identified in Table 2.7.4-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.7.5, Radiologically Controlled Area Ventilation System Table 2.7.5-2 as shown below:

Table 2.7.5-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
719	2.7.05.02.i	2. The VAS maintains each building area at a slightly negative pressure relative to the atmosphere or adjacent clean plant areas.	i) Testing will be performed to confirm that the VAS maintains each building at a slightly negative pressure when operating all VAS supply AHUs and all VAS exhaust fans.	i) The time average pressure differential in the served areas of the annex, fuel handling and radiologically controlled auxiliary buildings as measured by each of the instruments identified in Table 2.7.5-1 is negative.
			ii) Testing will be performed to confirm the ventilation flow rate through the auxiliary building fuel handling area when operating all VAS supply AHUs and all VAS exhaust fans.	ii) A report exists and concludes that the calculated exhaust flow rate based on the measured flow rates is greater than or equal to 15,300 cfm.
			iii) Testing will be performed to confirm the auxiliary building radiologically controlled area ventilation flow rate when operating all VAS supply AHUs and all VAS exhaust fans.	iii) A report exists and concludes that the calculated exhaust flow rate based on the measured flow rates is greater than or equal to 22,500 cfm.
		3. Displays of the parameters identified in Table 2.7.5-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.7.5-1 can be retrieved in the MCR.
720	2.7.05.02.ii	Not used per Amendment No. XX ~~2. The VAS maintains each building area at a slightly negative pressure relative to the atmosphere or adjacent clean plant areas.~~	~~ii) Testing will be performed to confirm the ventilation flow rate through the auxiliary building fuel handling area when operating all VAS supply AHUs and all VAS exhaust fans.~~	~~ii) A report exists and concludes that the calculated exhaust flow rate based on the measured flow rates is greater than or equal to 15,300 cfm.~~
721	2.7.05.02.iii	Not used per Amendment No. XX ~~2. The VAS maintains each building area at a slightly negative pressure relative to the atmosphere or adjacent clean plant areas.~~	~~iii) Testing will be performed to confirm the auxiliary building radiologically controlled area ventilation flow rate when operating all VAS supply AHUs and all VAS exhaust fans.~~	~~iii) A report exists and concludes that the calculated exhaust flow rate based on the measured flow rates is greater than or equal to 22,500 cfm.~~

Table 2.7.5-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
722	2.7.05.03	Not used per Amendment No. XX ~~3. Displays of the parameters identified in Table 2.7.5-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the parameters in the MCR.~~	~~The displays identified in Table 2.7.5-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1), Section 2.7.6, Containment Air Filtration System Table 2.7.6-2 as shown below:

Table 2.7.6-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
726	2.7.06.03.i	3. The VFS provides the intermittent flow of outdoor air to purge the containment atmosphere during normal plant operation, and continuous flow during hot or cold plant shutdown conditions.	i) Testing will be performed to confirm that containment supply AHU fan A when operated with containment exhaust fan A provides a flow of outdoor air.	i) The flow rate measured at each fan is greater than or equal to 3,600 scfm.
			ii) Testing will be performed to confirm that containment supply AHU fan B when operated with containment exhaust fan B provides a flow of outdoor air.	ii) The flow rate measured at each fan is greater than or equal to 3,600 scfm.
			iii) Inspection will be conducted of the containment purge discharge line (VFS-L204) penetrating the containment.	iii) The nominal line size is ≥ 36 in.
		4. Controls exist in the MCR to cause the components identified in Table 2.7.6-1 to perform the listed function.	Testing will be performed on the components in Table 2.7.6-1 using controls in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.7.6-1 to perform the listed functions.
		5. Displays of the parameters identified in Table 2.7.6-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.7.6-1 can be retrieved in the MCR.
727	2.7.06.03.ii	Not used per Amendment No. XX ~~3. The VFS provides the intermittent flow of outdoor air to purge the containment atmosphere during normal plant operation, and continuous flow during hot or cold plant shutdown conditions.~~	~~ii) Testing will be performed to confirm that containment supply AHU fan B when operated with containment exhaust fan B provides a flow of outdoor air.~~	~~ii) The flow rate measured at each fan is greater than or equal to 3,600 scfm.~~
728	2.7.06.03.iii	Not used per Amendment No. XX ~~3. The VFS provides the intermittent flow of outdoor air to purge the containment atmosphere during normal plant operation, and continuous flow during hot or cold plant shutdown conditions.~~	~~iii) Inspection will be conducted of the containment purge discharge line (VFS-L204) penetrating the containment.~~	~~iii) The nominal line size is ≥ 36 in.~~
729	2.7.06.04	Not used per Amendment No. XX ~~4. Controls exist in the MCR to cause the components identified in Table 2.7.6-1 to perform the listed function.~~	~~Testing will be performed on the components in Table 2.7.6-1 using controls in the MCR.~~	~~Controls in the MCR operate to cause the components listed in Table 2.7.6-1 to perform the listed functions.~~

Table 2.7.6-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
730	2.7.06.05	Not used per Amendment No. XX ~~5. Displays of the parameters identified in Table 2.7.6-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the parameters in the MCR.~~	~~The displays identified in Table 2.7.6-1 can be retrieved in the MCR.~~

Revise COL Appendix C (and plant-specific Tier 1) Section 3.3, Buildings Table 3.3-6 as shown below:

*Note: Changes to ITAAC Nos. 3.3.00.10ii and 3.3.00.10.iii in Table 3.3-6, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 10.ii and 10.iii in Table 3.3-6 is needed.

Table 3.3-6
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
816	3.3.00.10.ii	10. The shield building roof and PCS storage tank support and retain the PCS water sources. The PCS storage tank has a stainless steel liner which provides a barrier on the inside surfaces of the tank. Leak chase channels are provided on the tank boundary liner welds.	ii) An inspection of the PCS storage tank exterior tank boundary and shield building tension ring will be performed before and after filling of the PCS storage tank to the overflow level. The vertical elevation of the shield building roof will be measured at a location at the outer radius of the roof (tension ring) and at a location on the same azimuth at the outer radius of the PCS storage tank before and after filling the PCS storage tank. iii) An inspection of the PCS storage tank exterior tank boundary and shield building tension ring will be performed before and after filling of the PCS storage tank to the overflow level. The boundaries of the PCS storage tank and the shield building roof above the tension ring will be inspected visually for excessive concrete cracking.	ii) A report exists and concludes that inspection and measurement of the PCS storage tank and the tension ring structure, before and after filling of the tank, shows structural behavior under normal loads to be acceptable. iii) A report exists and concludes that there is no visible water leakage from the PCS storage tank through the concrete and that there is no visible excessive cracking in the boundaries of the PCS storage tank and the shield building roof above the tension ring.

Table 3.3-6 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
817	3.3.00.10.iii	Not used per Amendment No. XX~~10. The shield building roof and PCS storage tank support and retain the PCS water sources. The PCS storage tank has a stainless steel liner which provides a barrier on the inside surfaces of the tank. Leak chase channels are provided on the tank boundary liner welds.~~	~~iii) An inspection of the PCS storage tank exterior tank boundary and shield building tension ring will be performed before and after filling of the PCS storage tank to the overflow level. The boundaries of the PCS storage tank and the shield building roof above the tension ring will be inspected visually for excessive concrete cracking.~~	~~iii) A report exists and concludes that there is no visible water leakage from the PCS storage tank through the concrete and that there is no visible excessive cracking in the boundaries of the PCS storage tank and the shield building roof above the tension ring.~~
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 3.5, Radiation Monitoring, Table 3.5-6 as shown below:

Table 3.5-6
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
829	3.5.00.04	Not used per Amendment No. XX ~~4. Safety-related displays identified in Table 3.5-1 can be retrieved in the MCR.~~	~~Inspection will be performed for retrievability of the displays in the MCR.~~	~~Safety-related displays identified in Table 3.5-1 can be retrieved in the MCR.~~
830	3.5.00.05	Not used per Amendment No. XX ~~5. The process radiation monitors listed in Table 3.5-2 are provided.~~	~~Inspection for the existence of the monitors will be performed.~~	~~Each of the monitors listed in Table 3.5-2 exists.~~
831	3.5.00.06	4. Safety-related displays identified in Table 3.5-1 can be retrieved in the MCR. 5. The process radiation monitors listed in Table 3.5-2 are provided. 6. The effluent radiation monitors listed in Table 3.5-3 are provided. 7. The airborne radiation monitors listed in Table 3.5-4 are provided. 8. The area radiation monitors listed in Table 3.5-5 are provided.	Inspection will be performed for retrievability of the displays in the MCR. Inspection for the existence of the monitors will be performed. Inspection for the existence of the monitors will be performed. Inspection for the existence of the monitors will be performed. Inspection for the existence of the monitors will be performed.	Safety-related displays identified in Table 3.5-1 can be retrieved in the MCR. Each of the monitors listed in Table 3.5-2 exists. Each of the monitors listed in Table 3.5-3 exists. Each of the monitors listed in Table 3.5-4 exists. Each of the monitors listed in Table 3.5-5 exists.
832	3.5.00.07	Not used per Amendment No. XX ~~7. The airborne radiation monitors listed in Table 3.5-4 are provided.~~	~~Inspection for the existence of the monitors will be performed.~~	~~Each of the monitors listed in Table 3.5-4 exists.~~
833	3.5.00.08	Not used per Amendment No. XX ~~8. The area radiation monitors listed in Table 3.5-5 are provided.~~	~~Inspection for the existence of the monitors will be performed.~~	~~Each of the monitors listed in Table 3.5-5 exists.~~

Southern Nuclear Operating Company

ND-17-0XXX

Enclosure 4

Vogtle Electric Generating Plant (VEGP) Units 3 and 4

Roadmap of the Proposed Changes to the Licensing Basis Documents

(LAR-17-038)

(This Enclosure consists of 8 pages, including this cover page.)

ITAAC Consolidation Roadmap

Index Number to be removed	ITAAC Number to be removed	Consolidate with ITAAC Number: Host Index Number	Consolidate with ITAAC Number: Host ITAAC Number	Category	Justification
2	2.1.01.02	4	2.1.01.04	2	Related Pre-Op tests with related inspections
5	2.1.01.05				
6	2.1.01.06.i				
9	2.1.01.07.ii				
37	2.1.02.08d.vi	36	2.1.02.08d.v	3	Related inspections
38	2.1.02.08d.vii				
39	2.1.02.08d.viii				
45	2.1.02.10	47	2.1.02.11a.ii	1	Related Pre-Op and component tests
49	2.1.02.11b.ii				
50	2.1.02.11b.iii				
62	2.1.02.12b				
70	2.1.03.02b	69	2.1.03.02a	3	Related inspections
71	2.1.03.02c				
79	2.1.03.07.ii	78	2.1.03.07.i	1	Related Pre-Op tests
85	2.1.03.10				
111	2.2.01.10a	110	2.2.01.09	1	Related component tests
112	2.2.01.10b				
146	2.2.02.07f.ii	137	2.2.02.07a.iii	3	Related inspections
147	2.2.02.08a				
135	2.2.02.07a.i	138	2.2.02.07b.i	2	Related Pre-Op and component tests, with related inspections
136	2.2.02.07a.ii				
139	2.2.02.07b.ii				
140	2.2.02.07b.iii				
141	2.2.02.07c				
142	2.2.02.07d				
144	2.2.02.07e.ii				
150	2.2.02.09				
151	2.2.02.10a				
152	2.2.02.10b				
156	2.2.02.11a.iii				
157	2.2.02.11b				
148	2.2.02.08b	145	2.2.02.07f.i	1	Related Pre-Op tests

Index Number to be removed	ITAAC Number to be removed	Consolidate with ITAAC Number		Category	Justification
		Host Index Number	Host ITAAC Number		
188	2.2.03.08c.v.02	183	2.2.03.08c.iv.01	3	Related inspections
208	2.2.03.11a.ii	206	2.2.03.10	1	Related component tests
210	2.2.03.11b.ii				
211	2.2.03.11b.iii				
217	2.2.03.12b				
218	2.2.03.13				
240	2.2.04.09a.i	250	2.2.04.12a.iii	1	Related Pre-Op and component tests
244	2.2.04.10				
245	2.2.04.11a				
246	2.2.04.11b.i				
247	2.2.04.11b.ii				
251	2.2.04.12b				
267	2.2.05.07a.iii	265	2.2.05.07a.i	1	Related Pre-Op and component tests
268	2.2.05.07b.i				
269	2.2.05.07b.ii				
271	2.2.05.07d				
272	2.2.05.08				
273	2.2.05.09a				
274	2.2.05.09b				
275	2.2.05.10				
276	2.2.05.11				
277	2.2.05.12				
282	2.3.01.04	281	2.3.01.03.ii	1	Related Pre-Op and component tests
283	2.3.01.05				

Index Number to be removed	ITAAC Number to be removed	Consolidate with ITAAC Number		Category	Justification
		Host Index Number	Host ITAAC Number		
304	2.3.02.08b	301	2.3.02.08a.i	1	Related Pre-Op and component tests
305	2.3.02.09				
306	2.3.02.10a				
307	2.3.02.10b.i				
308	2.3.02.10b.ii				
311	2.3.02.11a.iii				
312	2.3.02.11a.iv				
313	2.3.02.11b				
314	2.3.02.12a				
315	2.3.02.12b				
316	2.3.02.13				
325	2.3.03.05	324	2.3.03.04	1	Related component tests
333	2.3.04.06	330	2.3.04.04.i	3	Related inspections
334	2.3.04.07				
345	2.3.05.03a.iii	344	2.3.05.03a.ii	3	Related construction tests
347	2.3.05.03b.ii	348	2.3.05.03b.iii	1	Related component tests
353	2.3.05.04				
376	2.3.06.09b.iii	375	2.3.06.09b.ii	2	Related Pre-Op tests with related inspections
377	2.3.06.09b.iv				
378	2.3.06.09b.v				
379	2.3.06.09c				
380	2.3.06.09d				
386	2.3.06.12a.iii				
387	2.3.06.12a.iv				
381	2.3.06.10	382	2.3.06.11a	1	Related Pre-Op and component tests
383	2.3.06.11b				
388	2.3.06.12b				
389	2.3.06.13				
390	2.3.06.14				
410	2.3.07.08.ii	408	2.3.07.07c	1	Related Pre-Op and component tests
411	2.3.07.09				
412	2.3.07.10				
413	2.3.07.11				

Index Number to be removed	ITAAC Number to be removed	Consolidate with ITAAC Number		Category	Justification
		Host Index Number	Host ITAAC Number		
418	2.3.08.03	415	2.3.08.02.i	1	Related Pre-Op and Component tests
419	2.3.08.04				
423	2.3.09.03.i	424	2.3.09.03.ii	2	Related Pre-Op and component tests with related inspections
427	2.3.09.04a				
429	2.3.09.05				
443	2.3.10.07a.i	444	2.3.10.07a.ii	1	Related Pre-Op and component tests
445	2.3.10.07b				
446	2.3.10.08				
447	2.3.10.09				
448	2.3.10.10				
471	2.3.13.09	470	2.3.13.08	1	Related Pre-Op and component tests
472	2.3.13.10a				
473	2.3.13.10b				
475	2.3.13.11b				
476	2.3.13.12				
484	2.3.19.01a	486	2.3.19.02a	2	Related Pre-Op tests with related inspections
485	2.3.19.01b				
487	2.3.19.02b				
490	2.3.29.03	489	2.3.29.02	1	Related Pre-Op tests
494	2.4.01.03	493	2.4.01.02	1	Related Pre-Op and component tests
495	2.4.01.04				
499	2.4.02.02c	497	2.4.02.02a	1	Related Pre-Op tests
501	2.4.02.03.ii				
507	2.5.01.02b	506	2.5.01.02a	1	Related Pre-Op and component tests
508	2.5.01.02c.i				
509	2.5.01.02c.ii				
510	2.5.01.02d				
516	2.5.01.03f				
517	2.5.01.03g				

Index Number to be removed	ITAAC Number to be removed	Consolidate with ITAAC Number		Category	Justification
		Host Index Number	Host ITAAC Number		
531	2.5.02.06b	530	2.5.02.06a.ii	1	Related Pre-Op tests and component tests
533	2.5.02.06c.ii				
539	2.5.02.08a.i				
541	2.5.02.08a.iii				
544	2.5.02.08c				
545	2.5.02.09a				
546	2.5.02.09b				
547	2.5.02.09c				
558	2.5.04.02.ii	557	2.5.04.02.i	1	Related Pre-Op Tests
559	2.5.04.02.iii				
562	C.2.5.04.04b	561	C.2.5.04.04a	3	Related inspections and analyses
563	C.2.5.04.04c				
584	2.6.01.04a	588	2.6.01.04e	1	Related Pre-Op and component tests
589	2.6.01.04f				
590	2.6.01.05				
591	2.6.01.06				
604	2.6.03.04d	603	2.6.03.04c	1	Related Pre-Op and component tests
605	2.6.03.04e				
606	2.6.03.04f				
607	2.6.03.04g				
608	2.6.03.04h				
610	2.6.03.05a				
611	2.6.03.05b				
612	2.6.03.05c				
615	2.6.03.06				
620	2.6.03.11				
614	2.6.03.05d.ii	613	2.6.03.05d.i	3	Related inspections
617	2.6.03.08	616	2.6.03.07	3	Related analyses
618	2.6.03.09				
619	2.6.03.10				
623	2.6.04.02b	622	2.6.04.02a	1	Related Pre-Op and component tests
625	2.6.04.03				
626	2.6.04.04				

Index Number to be removed	ITAAC Number to be removed	Consolidate with ITAAC Number		Category	Justification
		Host Index Number	Host ITAAC Number		
628	2.6.05.02.i	629	2.6.05.02.ii	2	Related Pre-Op tests with a related inspection
633	2.6.05.05.i				
634	2.6.05.05.ii				
635	2.6.05.06.i				
636	2.6.05.06.ii				
638	2.6.06.01.ii	637	2.6.06.01.i	3	Related inspections
639	2.6.06.01.iii				
640	2.6.06.01.iv				
656	2.6.09.15b	644	2.6.09.05a	4	Related security tests
648	2.6.09.07a	667	C.2.6.09.07	4	Related security inspections
649	2.6.09.07b				
651	2.6.09.09	670	C.2.6.09.09	4	Related security tests and inspections
653	2.6.09.13b	652	2.6.09.13a	4	Related security tests and inspections
657	2.6.09.16	655	2.6.09.15a	4	Related security tests and analyses
662	C.2.6.09.04a	661	C.2.6.09.03b	4	Related security tests and inspections
663	C.2.6.09.04b				
665	C.2.6.09.05b	664	C.2.6.09.05a	4	Related security tests and inspections
669	C.2.6.09.08b	668	C.2.6.09.08a	4	Related security inspections
693	2.7.01.08d	700	2.7.01.14	1	Related Pre-Op and component tests
694	2.7.01.09				
695	2.7.01.10a				
696	2.7.01.10b				
697	2.7.01.11				
698	2.7.01.12				
699	2.7.01.13				

Index Number to be removed	**ITAAC Number to be removed**	**Consolidate with ITAAC Number**		**Category**	**Justification**
		Host Index Number	**Host ITAAC Number**		
705	2.7.02.04	703	2.7.02.03a	1	Related Pre-Op and component tests
706	2.7.02.05				
711	2.7.03.04	710	2.7.03.03	1	Related component tests
717	2.7.04.04	716	2.7.04.03	1	Related component tests
720	2.7.05.02.ii	719	2.7.05.02.i	1	Related Pre-Op and component tests
721	2.7.05.02.iii				
722	2.7.05.03				
727	2.7.06.03.ii	726	2.7.06.03.i	2	Related Pre-Op and component tests with a related inspection
728	2.7.06.03.iii				
729	2.7.06.04				
730	2.7.06.05				
817	3.3.00.10.iii	816	3.3.00.10.ii	3	Related inspections
829	3.5.00.04	831	3.5.00.06	2	Component test with related inspections
830	3.5.00.05				
832	3.5.00.07				
833	3.5.00.08				

Southern Nuclear Operating Company

ND-17-0XXX

Enclosure 5

Vogtle Electric Generating Plant (VEGP) Units 3 and 4

Reviewer's Aid: Clean Pages of the Proposed Changes to the Licensing Basis Documents

(LAR-17-038)

(This Enclosure consists of 69 pages, including this cover page.)

Revise COL Appendix C (and plant-specific Tier 1), Section 2.1.1, Fuel Handling and Refueling System, Table 2.1.1-1, as shown below:

Table 2.1.1-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
2	2.1.01.02	Not used per Amendment No. XX		
***				
4	2.1.01.04	2. The FHS has the refueling machine (RM), the fuel handling machine (FHM), and the new and spent fuel storage racks.	Inspection of the system will be performed.	The FHS has the RM, the FHM, and the new and spent fuel storage racks.
		4. The RM and FHM/spent fuel handling tool (SFHT) gripper assemblies are designed to prevent opening while the weight of the fuel assembly is suspended from the grippers.	The RM and FHM/SFHT gripper assemblies will be tested by operating the open controls of the gripper while suspending a dummy fuel assembly.	The RM and FHM/SFHT gripper assemblies will not open while suspending a dummy test assembly.
		5. The lift height of the RM mast and FHM hoist(s) is limited such that the minimum required depth of water shielding is maintained.	The RM and FHM will be tested by attempting to raise a dummy fuel assembly.	The bottom of the dummy fuel assembly cannot be raised to within 24 ft, 6 in. of the operating deck floor.
		6. The RM and FHM are designed to maintain their load carrying and structural integrity functions during a safe shutdown earthquake.	i) Inspection will be performed to verify that the RM and FHM are located on the nuclear island.	i) The RM and FHM are located on the nuclear island.
		7. The new and spent fuel storage racks maintain the effective neutron multiplication factor required by 10 CFR 50.68 limits during normal operation, design basis seismic events, and design basis dropped spent fuel assembly accidents over the spent fuel storage racks.	ii) Inspection will be performed to verify that the new and spent fuel storage racks are located on the nuclear island.	ii) The new and spent fuel storage racks are located on the nuclear island.
5	2.1.01.05	Not used per Amendment No. XX		
6	2.1.01.06.i	Not used per Amendment No. XX		
***				
9	2.1.01.07.ii	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.1.2, Reactor Coolant System, Table 2.1.2-4 as shown below:

*Note: Changes to ITAAC Nos. 2.1.02.08d.vi, 2.1.02.08d.vii and 2.1.2.02.08d.viii in Table 2.1.2-4, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 8d.vi, 8d.vii and 8d.viii in Table 2.1.2-4 is needed.

Table 2.1.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
36	2.1.02.08d.v	8.d) The RCS provides automatic depressurization during design basis events.	v) Inspections of the elevation of the ADS stage 4 valve discharge will be conducted. vi) Inspections of the ADS stage 4 valve discharge will be conducted. vii) Inspection of each ADS sparger will be conducted to determine the flow area through the sparger holes. viii) Inspection of the elevation of each ADS sparger will be conducted.	v) The minimum elevation of the bottom inside surface of the outlet of these valves is greater than plant elevation 110 feet. vi) The discharge of the ADS stage 4 valves is directed into the steam generator compartments. vii) The flow area through the holes in each ADS sparger is $\geq$ 274 in^2. viii) The centerline of the connection of the sparger arms to the sparger hub is $\leq$ 11.5 feet below the IRWST overflow level.
37	2.1.02.08d.vi	Not used per Amendment No. XX		
38	2.1.02.08d.vii	Not used per Amendment No. XX		
39	2.1.02.08d.viii	Not used per Amendment No. XX		
***				
45	2.1.02.10	Not used per Amendment No. XX		
***				
47	2.1.02.11a.ii	10. Safety-related displays identified in Table 2.1.2-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.1.2-1 can be retrieved in the MCR.

Table 2.1.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		11.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.1.2-1 to perform active functions.	ii) Stroke testing will be performed on the other remotely operated valves listed in Table 2.1.2-1 using controls in the MCR.	ii) Controls in the MCR operate to cause the remotely operated valves (other than squib valves) to perform active functions.
		11.b) The valves identified in Table 2.1.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	ii) Testing will be performed on the other remotely operated valves identified in Table 2.1.2-1 using real or simulated signals into the PMS.	ii) The other remotely operated valves identified in Table 2.1.2-1 as having PMS control perform the active function identified in the table after receiving a signal from PMS.
		11.b) The valves identified in Table 2.1.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	iii) Testing will be performed to demonstrate that remotely operated RCS valves RCS-V001A/B, V002A/B, V003A/B, V011A/B, V012A/B, V013A/B open within the required response times.	iii) These valves open within the following times after receipt of an actuation signal: V001A/B $\leq$ 40 sec V002A/B, V003A/B $\leq$ 100 sec V011A/B $\leq$ 30 sec V012A/B, V013A/B $\leq$ 60 sec
		12.b) After loss of motive power, the remotely operated valves identified in Table 2.1.2-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	Upon loss of motive power, each remotely operated valve identified in Table 2.1.2-1 assumes the indicated loss of motive power position.
***				
49	2.1.02.11b.ii	Not used per Amendment No. XX		
50	2.1.02.11b.iii	Not used per Amendment No. XX		
***				
62	2.1.02.12b	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1) Section 2.1.3, Reactor System, Table 2.1.3-2 as shown below:

Table 2.1.3-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
69	2.1.03.02a	2.a) The reactor upper internals rod guide arrangement is as shown in Figure 2.1.3-1.	Inspection of the as-built system will be performed.	The as-built RXS will accommodate the fuel assembly and control rod drive mechanism pattern shown in Figure 2.1.3-1.
		2.b) The control assemblies (rod cluster and gray rod) and drive rod arrangement is as shown in Figure 2.1.3-2.	Inspection of the as-built system will be performed.	The as-built RXS will accommodate the control assemblies (rod cluster and gray rod) and drive rod arrangement shown in Figure 2.1.3-2.
		2.c) The reactor vessel arrangement is as shown in Figure 2.1.3-3.	Inspection of the as-built system will be performed.	The as-built RXS will accommodate the reactor vessel arrangement shown in Figure 2.1.3-3.
70	2.1.03.02b	Not used per Amendment No. XX		
71	2.1.03.02c	Not used per Amendment No. XX		
***				

Table 2.1.3-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
78	2.1.03.07.i	7. The reactor internals will withstand the effects of flow induced vibration.	i) A vibration type test will be conducted on the (first unit) reactor internals representative of AP1000.	i) A report exists and concludes that the (first unit) reactor internals have no observable damage or loose parts as a result of the vibration type test.
			ii) A pre-test inspection, a flow test and a post-test inspection will be conducted on the as-built reactor internals.	ii) The as-built reactor internals have no observable damage or loose parts.
		10. The reactor lower internals assembly is equipped with holders for at least eight capsules for storing material surveillance specimens.	Inspection of the reactor lower internals assembly for the presence of capsules will be performed.	At least eight capsules are in the reactor lower internals assembly.
79	2.1.03.07.ii	Not used per Amendment No. XX		
***				
85	2.1.03.10	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.2.1, Containment System, Table 2.2.1-3 as shown below:

Table 2.2.1-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
110	2.2.01.09	9. Safety-related displays identified in Table 2.2.1-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.2.1-1 can be retrieved in the MCR.
		10.a) Controls exist in the MCR to cause those remotely operated valves identified in Table 2.2.1-1 to perform active functions.	Stroke testing will be performed on remotely operated valves identified in Table 2.2.1-1 using the controls in the MCR.	Controls in the MCR operate to cause remotely operated valves identified in Table 2.2.1-1 to perform active safety functions.
		10.b) The valves identified in Table 2.2.1-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	Testing will be performed on remotely operated valves listed in Table 2.2.1-1 using real or simulated signals into the PMS.	The remotely operated valves identified in Table 2.2.1-1 as having PMS control perform the active function identified in the table after receiving a signal from PMS.
111	2.2.01.10a	Not used per Amendment No. XX		
112	2.2.01.10b	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.2.2, Passive Containment Cooling System, Table 2.2.2-3 as shown below:

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
135	2.2.02.07a.i	Not used per Amendment No. XX		
136	2.2.02.07a.ii	Not used per Amendment No. XX		
137	2.2.02.07a.iii	7.a) The PCS delivers water from the PCCWST to the outside, top of the containment vessel.	iii) Inspection will be performed to determine the PCCWST standpipes elevations.	iii) The elevations of the standpipes above the tank floor are: – 16.8 ft ± 0.2 ft – 20.3 ft ± 0.2 ft – 24.1 ft ± 0.2 ft
		7.f) The PCS provides a flow path for long-term water makeup from the PCCWST to the spent fuel pool.	ii) Inspection of the PCCWST will be performed.	ii) The volume of the PCCWST is greater than 756,700 gallons.
		8.a) The PCCAWST contains an inventory of cooling water sufficient for PCS containment cooling from hour 72 through day 7.	Inspection of the PCCAWST will be performed.	The volume of the PCCAWST is greater than 780,000 gallons.
138	2.2.02.07b.i	7.a) The PCS delivers water from the PCCWST to the outside, top of the containment vessel.	i) Testing will be performed to measure the PCCWST delivery rate from each one of the three parallel flow paths.	i) When tested, each one of the three flow paths delivers water at greater than or equal to: – 469.1 gpm at a PCCWST water level of 27.4 ft + 0.2, - 0.0 ft above the tank floor – 226.6 gpm when the PCCWST water level uncovers the first (i.e. tallest) standpipe – 176.3 gpm when the PCCWST water level uncovers the second tallest standpipe – 144.2 gpm when the PCCWST water level uncovers the third tallest standpipe

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
			ii) Testing and or analysis will be performed to demonstrate the PCCWST inventory provides 72 hours of adequate water flow.	ii) When tested and/or analyzed with all flow paths delivering and an initial water level at 27.4 + 0.2, - 0.00 ft, the PCCWST water inventory provides greater than or equal to 72 hours of flow, and the flow rate at 72 hours is greater than or equal to 100.7 gpm.
		7.b) The PCS wets the outside surface of the containment vessel. The inside and the outside of the containment vessel above the operating deck are coated with an inorganic zinc material.	i) Testing will be performed to measure the outside wetted surface of the containment vessel with one of the three parallel flow paths delivering water to the top of the containment vessel.	i) A report exists and concludes that when the water in the PCCWST uncovers the standpipes at the following levels, the water delivered by one of the three parallel flow paths to the containment shell provides coverage measured at the spring line that is equal to or greater than the stated coverages. - 24.1 ± 0.2 ft above the tank floor; at least 90% of the perimeter is wetted. - 20.3 ± 0.2 ft above the tank floor; at least 72.9% of the perimeter is wetted. - 16.8 ± 0.2 ft above the tank floor; at least 59.6% of the perimeter is wetted.
			ii) Inspection of the containment vessel exterior coating will be conducted.	ii) A report exists and concludes that the containment vessel exterior surface is coated with an inorganic zinc coating above elevation 135'-3".
			iii) Inspection of the containment vessel interior coating will be conducted.	iii) A report exists and concludes that the containment vessel interior surface is coated with an inorganic zinc coating above 7' above the operating deck.
		7.c) The PCS provides air flow over the outside of the containment vessel by a natural circulation air flow path from the air inlets to the air discharge structure.	Inspections of the air flow path segments will be performed.	Flow paths exist at each of the following locations: – Air inlets – Base of the outer annulus – Base of the inner annulus – Discharge structure

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		7.d) The PCS drains the excess water from the outside of the containment vessel through the two upper annulus drains.	Testing will be performed to verify the upper annulus drain flow performance.	With a water level within the upper annulus 10" ± 1" above the annulus drain inlet, the flow rate through each drain is greater than or equal to 525 gpm.
		7.e) The PCS provides a flow path for long-term water makeup to the PCCWST.	ii) Testing will be performed to measure the delivery rate from the long-term makeup connection to the PCCWST.	ii) With a water supply connected to the PCS long-term makeup connection, each PCS recirculation pump delivers greater than or equal to 100 gpm when tested separately.
		9. Safety-related displays identified in Table 2.2.2-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.2.2-1 can be retrieved in the MCR.
		10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.2.2-1 to perform active functions.	Stroke testing will be performed on the remotely operated valves identified in Table 2.2.2-1 using the controls in the MCR.	Controls in the MCR operate to cause remotely operated valves identified in Table 2.2.2-1 to perform active functions.
		10.b) The valves identified in Table 2.2.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	Testing will be performed on the remotely operated valves in Table 2.2.2-1 using real or simulated signals into the PMS.	The remotely operated valves identified in Table 2.2.2-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.
		11.a) The motor-operated valves identified in Table 2.2.2-1 perform an active safety-related function to change position as indicated in the table.	iii) Tests of the motor-operated valves will be performed under preoperational flow, differential pressure, and temperature conditions.	iii) Each motor-operated valve changes position as indicated in Table 2.2.2-1 under preoperational test conditions.
		11.b) After loss of motive power, the remotely operated valves identified in Table 2.2.2-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	After loss of motive power, each remotely operated valve identified in Table 2.2.2-1 assumes the indicated loss of motive power position.
139	2.2.02.07b.ii	Not used per Amendment No. XX		
140	2.2.02.07b.iii	Not used per Amendment No. XX		
141	2.2.02.07c	Not used per Amendment No. XX		
142	2.2.02.07d	Not used per Amendment No. XX		
		***		
144	2.2.02.07e.ii	Not used per Amendment No. XX		

Table 2.2.2-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
145	2.2.02.07f.i	7.f) The PCS provides a flow path for long-term water makeup from the PCCWST to the spent fuel pool. 8.b) The PCS delivers water from the PCCAWST to the PCCWST and spent fuel pool simultaneously.	i) Testing will be performed to measure the delivery rate from the PCCWST to the spent fuel pool. Testing will be performed to measure the delivery rate from the PCCAWST to the PCCWST and spent fuel pool simultaneously.	i) With the PCCWST water level at 27.4 ft + 0.2, - 0.0 ft above the bottom of the tank, the flow path from the PCCWST to the spent fuel pool delivers greater than or equal to 118 gpm. With PCCAWST aligned to the suction of the recirculation pumps, each pump delivers greater than or equal to 100 gpm to the PCCWST and 35 gpm to the spent fuel pool simultaneously when each pump is tested separately.
146	2.2.02.07f.ii	Not used per Amendment No. XX		
147	2.2.02.08a	Not used per Amendment No. XX		
148	2.2.02.08b	Not used per Amendment No. XX		
***				
150	2.2.02.09	Not used per Amendment No. XX		
151	2.2.02.10a	Not used per Amendment No. XX		
152	2.2.02.10b	Not used per Amendment No. XX		
***				
156	2.2.02.11a.iii	Not used per Amendment No. XX		
157	2.2.02.11b	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1) Section 2.2.3, Passive Core Cooling System, Table 2.2.3-4 as shown below:

Table 2.2.3-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
183	2.2.03.08c.iv.01	8.c) The PXS provides RCS makeup, boration, and safety injection during design basis events.	iv) Inspections of the elevation of the following pipe lines will be conducted: 1. IRWST injection lines; IRWST connection to DVI nozzles v) Inspections of the elevation of the following tanks will be conducted: 2. IRWST	iv) The maximum elevation of the top inside surface of these lines is less than the elevation of: 1. IRWST bottom inside surface v) The elevation of the bottom inside tank surface is higher than the direct vessel injection nozzle centerline by the following: 2. IRWST ≥ 3.4 ft
***				
188	2.2.03.08c.v.02	Not used per Amendment No. XX		
***				
206	2.2.03.10	10. Safety-related displays of the parameters identified in Table 2.2.3-1 can be retrieved in the MCR. 11.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.2.3-1 to perform their active function(s). 11.b) The valves identified in Table 2.2.3-1 as having PMS control perform their active function after receiving a signal from the PMS.	Inspection will be performed for the retrievability of the safety-related displays in the MCR. ii) Stroke testing will be performed on remotely operated valves other than squib valves identified in Table 2.2.3-1 using the controls in the MCR. ii) Testing will be performed on the remotely operated valves other than squib valves identified in Table 2.2.3-1 using real or simulated signals into the PMS. iii) Testing will be performed to demonstrate that remotely operated PXS isolation valves PXS-V014A/B, V015A/B, V108A/B open within the required response times.	Safety-related displays identified in Table 2.2.3-1 can be retrieved in the MCR. ii) Controls in the MCR operate to cause remotely operated valves other than squib valves to perform their active functions. ii) Remotely operated valves other than squib valves perform the active function identified in the table after a signal is input to the PMS. iii) These valves open within 20 seconds after receipt of an actuation signal.

Table 2.2.3-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		12.b) After loss of motive power, the remotely operated valves identified in Table 2.2.3-1 assume the indicated loss of motive power position. 13. Displays of the parameters identified in Table 2.2.3-3 can be retrieved in the MCR.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power. Inspection will be performed for retrievability of the displays identified in Table 2.2.3-3 in the MCR.	After loss of motive power, each remotely operated valve identified in Table 2.2.3-1 assumes the indicated loss of motive power position. Displays identified in Table 2.2.3-3 can be retrieved in the MCR.
***				
208	2.2.03.11a.ii	Not used per Amendment No. XX		
***				
210	2.2.03.11b.ii	Not used per Amendment No. XX		
211	2.2.03.11b.iii	Not used per Amendment No. XX		
***				
217	2.2.03.12b	Not used per Amendment No. XX		
218	2.2.03.13	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.2.4, Steam Generator System, Table 2.2.4-4 as shown below:

Table 2.2.4-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
240	2.2.04.09a.i	Not used per Amendment No. XX		
***				
244	2.2.04.10	Not used per Amendment No. XX		
245	2.2.04.11a	Not used per Amendment No. XX		
246	2.2.04.11b.i	Not used per Amendment No. XX		
247	2.2.04.11b.ii	Not used per Amendment No. XX		
***				
250	2.2.04.12a.iii	9.a) Components within the main steam system, main and startup feedwater system, and the main turbine system identified in Table 2.2.4-3 provide backup isolation of the SGS to limit steam generator blowdown and feedwater flow to the steam generator.	i) Testing will be performed to confirm closure of the valves identified in Table 2.2.4-3.	i) The valves identified in Table 2.2.4-3 close after a signal is generated by the PMS.
		10. Safety-related displays identified in Table 2.2.4-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.2.4-1 can be retrieved in the MCR.
		11.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.2.4-1 to perform active functions.	Stroke testing will be performed on the remotely operated valves listed in Table 2.2.4-1 using controls in the MCR.	Controls in the MCR operate to cause the remotely operated valves to perform active safety functions.
		11.b) The valves identified in Table 2.2.4-1 as having PMS control perform an active safety function after receiving a signal from PMS.	i) Testing will be performed on the remotely operated valves listed in Table 2.2.4-1 using real or simulated signals into the PMS.	i) The remotely-operated valves identified in Table 2.2.4-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.
			ii) Testing will be performed to demonstrate that remotely operated SGS isolation valves SGS-V027A/B, V040A/B, V057A/B, V250A/B close within the required response times.	ii) These valves close within the following times after receipt of an actuation signal: V027A/B < 44 sec V040A/B, V057A/B < 5 sec V250A/B < 5 sec

Table 2.2.4-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		12.a) The motor-operated valves identified in Table 2.2.4-1 perform an active safety-related function to change position as indicated in the table. 12.b) After loss of motive power, the remotely operated valves identified in Table 2.2.4-1 assume the indicated loss of motive power position.	iii) Tests of the motor-operated valves will be performed under pre-operational flow, differential pressure, and temperature conditions. Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	iii) Each motor-operated valve changes position as indicated in Table 2.2.4-1 under pre-operational test conditions. After loss of motive power, each remotely operated valve identified in Table 2.2.4-1 assumes the indicated loss of motive power position. Motive power to SGS-PL-V040A/B and SGS-PL-V057A/B is electric power to the actuator from plant services.
251	2.2.04.12b	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.2.5, Main Control Room Emergency Habitability System, Table 2.2.5-5 as shown below:

Table 2.2.5-5
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
265	2.2.05.07a.i	7.a) The VES provides a 72-hour supply of breathable quality air for the occupants of the MCR.	i) Testing will be performed to confirm that the required amount of air flow is delivered to the MCR.	i) The air flow rate from the VES is at least 60 scfm and not more than 70 scfm.
			iii) MCR air samples will be taken during VES testing and analyzed for quality.	iii) The MCR air is of breathable quality.
		7.b) The VES maintains the MCR pressure boundary at a positive pressure with respect to the surrounding areas.	i) Testing will be performed with VES flow rate between 60 and 70 scfm to confirm that the MCR is capable of maintaining the required pressurization of the pressure boundary.	i) The MCR pressure boundary is pressurized to greater than or equal to 1/8-in. water gauge with respect to the surrounding area.
			ii) Air leakage into the MCR will be measured during VES testing using a tracer gas.	ii) Air leakage into the MCR is less than or equal to 10 cfm.
		7.d) The system provides a passive recirculation flow of MCR air to maintain main control room dose rates below an acceptable level during VES operation.	Testing will be performed to confirm that the required amount of air flow circulates through the MCR passive filtration system.	The air flow rate at the outlet of the MCR passive filtration system is at least 600 cfm greater than the flow measured by VES-003A/B.
		8. Safety-related displays identified in Table 2.2.5-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.2.5-1 can be retrieved in the MCR.
		9.a) Controls exist in the MCR to cause remotely operated valves identified in Table 2.2.5-1 to perform their active functions.	Stroke testing will be performed on remotely operated valves identified in Table 2.2.5-1 using the controls in the MCR.	Controls in the MCR operate to cause remotely operated valves identified in Table 2.2.5-1 to perform their active safety functions.
		9.b) The valves identified in Table 2.2.5-1 as having PMS control perform their active safety function after receiving a signal from the PMS.	Testing will be performed on remotely operated valves listed in Table 2.2.5-1 using real or simulated signals into the PMS.	The remotely operated valves identified in Table 2.2.5-1 as having PMS control perform the active safety function identified in the table after receiving a signal from the PMS.

Table 2.2.5-5 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
		10. After loss of motive power, the remotely operated valves identified in Table 2.2.5-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	After loss of motive power, each remotely operated valve identified in Table 2.2.5-1 assumes the indicated loss of motive power position.
		11. Displays of the parameters identified in Table 2.2.5-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.2.5-3 can be retrieved in the MCR.
		12. The background noise level in the MCR does not exceed 65 dB(A) at the operator workstations when VES is operating.	The as-built VES will be operated, and background noise levels in the MCR will be measured at the operator work stations with the plant not operating.	The background noise level in the MCR does not exceed 65 dB(A) at the operator work stations when the VES is operating.
***				
267	2.2.05.07a.iii	Not used per Amendment No. XX		
268	2.2.05.07b.i	Not used per Amendment No. XX		
269	2.2.05.07b.ii	Not used per Amendment No. XX		
***				
271	2.2.05.07d	Not used per Amendment No. XX		
272	2.2.05.08	Not used per Amendment No. XX		
273	2.2.05.09a	Not used per Amendment No. XX		
274	2.2.05.09b	Not used per Amendment No. XX		
275	2.2.05.10	Not used per Amendment No. XX		
276	2.2.05.11	Not used per Amendment No. XX		
277	2.2.05.12	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.1, Component Cooling Water System, Table 2.3.1-2 as shown below:

Table 2.3.1-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
281	2.3.01.03.ii	3. The CCS provides the nonsafety-related functions of transferring heat from the RNS during shutdown and the spent fuel pool cooling system during all modes of operation to the SWS.	ii) Testing will be performed to confirm that the CCS can provide cooling water to the RNS HXs while providing cooling water to the SFS HXs.	ii) Each pump of the CCS can provide at least 2685 gpm of cooling water to one RNS HX and at least 1200 gpm of cooling water to one SFS HX while providing at least 4415 gpm to other users of cooling water.
		4. Controls exist in the MCR to cause the pumps identified in Table 2.3.1-1 to perform the listed functions.	Testing will be performed to actuate the pumps identified in Table 2.3.1-1 using controls in the MCR.	Controls in the MCR operate to cause pumps listed in Table 2.3.1-1 to perform the listed functions.
		5. Displays of the parameters identified in Table 2.3.1-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	Displays identified in Table 2.3.1-1 can be retrieved in the MCR.
282	2.3.01.04	Not used per Amendment No. XX		
283	2.3.01.05	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.2, Chemical and Volume Control System, Table 2.3.2-4 as shown below:

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
301	2.3.02.08a.i	8.a) The CVS provides makeup water to the RCS.	i) Testing will be performed by aligning a flow path from each CVS makeup pump, actuating makeup flow to the RCS at pressure greater than or equal to 2000 psia, and measuring the flow rate in the makeup pump discharge line with each pump suction aligned to the boric acid storage tank.	i) Each CVS makeup pump provides a flow rate of greater than or equal to 100 gpm.
		8.b) The CVS provides the pressurizer auxiliary spray.	Testing will be performed by aligning a flow path from each CVS makeup pump to the pressurizer auxiliary spray and measuring the flow rate in the makeup pump discharge line with each pump suction aligned to the boric acid storage tank and with RCS pressure greater than or equal to 2000 psia.	Each CVS makeup pump provides spray flow to the pressurizer.
		9. Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.3.2-1 can be retrieved in the MCR.
		10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.	Stroke testing will be performed on the remotely operated valves identified in Table 2.3.2-1 using the controls in the MCR.	Controls in the MCR operate to cause the remotely operated valves identified in Table 2.3.2-1 to perform active functions.
		10.b) The valves identified in Table 2.3.2-1 as having PMS control perform an active safety function after receiving a signal from the PMS.	i) Testing will be performed using real or simulated signals into the PMS.	i) The valves identified in Table 2.3.2-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.

Table 2.3.2-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
			ii) Testing will be performed to demonstrate that the remotely operated CVS isolation valves CVS-V090, V091, V136A/B close within the required response time.	ii) These valves close within the following times after receipt of an actuation signal: V090, V091 < 30 sec V136A/B < 20 sec
		11.a) The motor-operated and check valves identified in Table 2.3.2-1 perform an active safety-related function to change position as indicated in the table.	iii) Tests of the motor-operated valves will be performed under pre-operational flow, differential pressure, and temperature conditions. iv) Exercise testing of the check valves with active safety functions identified in Table 2.3.2-1 will be performed under pre-operational test pressure, temperature and fluid flow conditions.	iii) Each motor-operated valve changes position as indicated in Table 2.3.2-1 under pre-operational test conditions. iv) Each check valve changes position as indicated in Table 2.3.2-1.
		11.b) After loss of motive power, the remotely operated valves identified in Table 2.3.2-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	Upon loss of motive power, each remotely operated valve identified in Table 2.3.2-1 assumes the indicated loss of motive power position.
		12.a) Controls exist in the MCR to cause the pumps identified in Table 2.3.2-3 to perform the listed function.	Testing will be performed to actuate the pumps identified in Table 2.3.2-3 using controls in the MCR.	Controls in the MCR cause pumps identified in Table 2.3.2-3 to perform the listed function.
		12.b) The pumps identified in Table 2.3.2-3 start after receiving a signal from the PLS.	Testing will be performed to confirm starting of the pumps identified in Table 2.3.2-3.	The pumps identified in Table 2.3.2-3 start after a signal is generated by the PLS.
		13. Displays of the parameters identified in Table 2.3.2-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.3.2-3 in the MCR.	Displays identified in Table 2.3.2-3 can be retrieved in the MCR.
***				
304	2.3.02.08b	Not used per Amendment No. XX		
305	2.3.02.09	Not used per Amendment No. XX		
306	2.3.02.10a	Not used per Amendment No. XX		
307	2.3.02.10b.i	Not used per Amendment No. XX		

Table 2.3.2-4 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
308	2.3.02.10b.ii	Not used per Amendment No. XX		
***				
311	2.3.02.11a.iii	Not used per Amendment No. XX		
312	2.3.02.11a.iv	Not used per Amendment No. XX		
313	2.3.02.11b	Not used per Amendment No. XX		
314	2.3.02.12a	Not used per Amendment No. XX		
315	2.3.02.12b	Not used per Amendment No. XX		
316	2.3.02.13	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.3, Standby Diesel Fuel Oil System, Table 2.3.3-2 as shown below:

Table 2.3.3-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
324	2.3.03.04	4. Controls exist in the MCR to cause the components identified in Table 2.3.3-1 to perform the listed function. 5. Displays of the parameters identified in Table 2.3.3-1 can be retrieved in the MCR.	Testing will be performed on the components in Table 2.3.3-1 using controls in the MCR. Inspection will be performed for retrievability of parameters in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.3.3-1 to perform the listed functions. The displays identified in Table 2.3.3-1 can be retrieved in the MCR.
325	2.3.03.05	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.4, Fire Protection System, Table 2.3.4-2 as shown below:

Table 2.3.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
330	2.3.04.04.i	4. The FPS provides for manual fire fighting capability in plant areas containing safety-related equipment.	i) Inspection of the passive containment cooling system (PCS) storage tank will be performed.	i) The volume of the PCS tank above the standpipe feeding the FPS and below the overflow is at least 18,000 gal.
		6. The FPS provides nonsafety-related containment spray for severe accident management.	Inspection of the containment spray headers will be performed.	The FPS has spray headers and nozzles as follows: At least 44 nozzles at plant elevation of at least 260 feet, and 24 nozzles at plant elevation of at least 275 feet.
		7. The FPS provides two fire water storage tanks, each capable of holding at least 300,000 gallons of water.	Inspection of each fire water storage tank will be performed.	The volume of each fire water storage tank supplying the FPS is at least 300,000 gallons.
***				
333	2.3.04.06	Not used per Amendment No. XX		
334	2.3.04.07	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.5, Mechanical Handling System, Table 2.3.5-2 as shown below:

*Note: Changes to ITAAC Nos. 2.3.05.03a.ii and 2.3.05.03a.iii in Table 2.3.5-2, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 03a.ii and 03a.iii in Table 2.3.5-2 is needed.

Table 2.3.5-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
344	2.3.05.03a.ii	3.a) The polar crane is single failure proof.	ii) Testing of the polar crane is performed. iii) Testing of the polar crane is performed.	ii) The polar crane shall be static-load tested to 125% of the rated load. iii) The polar crane shall lift a test load that is 100% of the rated load. Then it shall lower, stop, and hold the test load.
345	2.3.05.03a.iii	Not used per Amendment No. XX		
***				
347	2.3.05.03b.ii	Not used per Amendment No. XX		
348	2.3.05.03b.iii	3.b) The cask handling crane is single failure proof. 4. The cask handling crane cannot move over the spent fuel pool.	ii) Testing of the cask handling crane is performed. iii) Testing of the cask handling crane is performed. Testing of the cask handling crane is performed.	ii) The cask handling crane shall be static load tested to 125% of the rated load. iii) The cask handling crane shall lift a test load that is 100% of the rated load. Then it shall lower, stop, and hold the test load. The cask handling crane does not move over the spent fuel pool.
***				
353	2.3.05.04	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.6 Normal Residual Heat Removal System, Table 2.3.6-4 as shown below:

Table 2.3.6-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
375	2.3.06.09b.ii	9.b) The RNS provides heat removal from the reactor coolant during shutdown operations.	ii) Testing will be performed to confirm that the RNS can provide flow through the RNS heat exchangers when the pump suction is aligned to the RCS hot leg and the discharge is aligned to both PXS DVI lines with the RCS at atmospheric pressure.	ii) Each RNS pump provides at least 1400 gpm net flow to the RCS when the hot leg water level is at an elevation 15.5 inches ± 2 inches above the bottom of the hot leg.
			iii) Inspection will be performed of the reactor coolant loop piping.	iii) The RCS cold legs piping centerline is 17.5 inches ± 2 inches above the hot legs piping centerline.
			iv) Inspection will be performed of the RNS pump suction piping.	iv) The RNS pump suction piping from the hot leg to the pump suction piping low point does not form a local high point (defined as an upward slope with a vertical rise greater than 3 inches).
			v) Inspection will be performed of the RNS pump suction nozzle connection to the RCS hot leg.	v) The RNS suction line connection to the RCS is constructed from 20-inch Schedule 140 pipe.
		9.c) The RNS provides low pressure makeup flow from the cask loading pit to the RCS for scenarios following actuation of the ADS.	Testing will be performed to confirm that the RNS can provide low pressure makeup flow from the cask loading pit to the RCS when the pump suction is aligned to the cask loading pit and the discharge is aligned to both PXS DVI lines with RCS at atmospheric pressure.	Each RNS pump provides at least 1100 gpm net flow to the RCS when the water level above the bottom of the cask loading pit is 1 foot ± 6 inches.
		9.d) The RNS provides heat removal from the in-containment refueling water storage tank (IRWST).	Testing will be performed to confirm that the RNS can provide flow through the RNS heat exchangers when the pump suction is aligned to the IRWST and the discharge is aligned to the IRWST.	Two operating RNS pumps provide at least 2000 gpm to the IRWST.

Table 2.3.6-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		12.a) The motor-operated and check valves identified in Table 2.3.6-1 perform an active safety-related function to change position as indicated in the table.	iii) Tests of the motor-operated valves will be performed under preoperational flow, differential pressure and temperature conditions. iv) Exercise testing of the check valves active safety functions identified in Table 2.3.6-1 will be performed under preoperational test pressure, temperature and fluid flow conditions.	iii) Each motor-operated valve changes position as indicated in Table 2.3.6-1 under preoperational test conditions. iv) Each check valve changes position as indicated in Table 2.3.6-1.
376	2.3.06.09b.iii	Not used per Amendment No. XX		
377	2.3.06.09b.iv	Not used per Amendment No. XX		
378	2.3.06.09b.v	Not used per Amendment No. XX		
379	2.3.06.09c	Not used per Amendment No. XX		
380	2.3.06.09d	Not used per Amendment No. XX		
381	2.3.06.10	Not used per Amendment No. XX		
382	2.3.06.11a	10. Safety-related displays identified in Table 2.3.6-1 can be retrieved in the MCR. 11.a) Controls exist in the MCR to cause those remotely operated valves identified in Table 2.3.6-1 to perform active functions. 11.b) The valves identified in Table 2.3.6-1 as having PMS control perform active safety functions after receiving a signal from the PMS. 12.b) After loss of motive power, the remotely operated valves identified in Table 2.3.6-1 assume the indicated loss of motive power position.	Inspection will be performed for retrievability of the safety-related displays in the MCR. Stroke testing will be performed on the remotely operated valves identified in Table 2.3.6-1 using the controls in the MCR. Testing will be performed using real or simulated signals into the PMS. Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	Safety-related displays identified in Table 2.3.6-1 can be retrieved in the MCR. Controls in the MCR operate to cause those remotely operated valves identified in Table 2.3.6-1 to perform active functions. The valves identified in Table 2.3.6-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS. Upon loss of motive power, each remotely operated valve identified in Table 2.3.6-1 assumes the indicated loss of motive power position.

Table 2.3.6-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		13. Controls exist in the MCR to cause the pumps identified in Table 2.3.6-3 to perform the listed function.	Testing will be performed to actuate the pumps identified in Table 2.3.6-3 using controls in the MCR.	Controls in the MCR cause pumps identified in Table 2.3.6-3 to perform the listed action.
		14. Displays of the RNS parameters identified in Table 2.3.6-3 can be retrieved in the MCR.	Inspection will be performed for retrievability in the MCR of the displays identified in Table 2.3.6-3.	Displays of the RNS parameters identified in Table 2.3.6-3 are retrieved in the MCR.
383	2.3.06.11b	Not used per Amendment No. XX		
***				
386	2.3.06.12a.iii	Not used per Amendment No. XX		
387	2.3.06.12a.iv	Not used per Amendment No. XX		
388	2.3.06.12b	Not used per Amendment No. XX		
389	2.3.06.13	Not used per Amendment No. XX		
390	2.3.06.14	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.7, Spent Fuel Pool Cooling System, Table 2.3.7-4 as shown below:

Table 2.3.7-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
408	2.3.07.07c	7c) The SFS provides check valves in the drain line from the refueling cavity to prevent flooding of the refueling cavity during containment flooding.	Exercise testing of the check valves with active safety-functions identified in Table 2.3.7-1 will be performed under pre-operational test pressure, temperature and flow conditions.	Each check valve changes position as indicated on Table 2.3.7-1.
		8. The SFS provides the nonsafety-related function of removing spent fuel decay heat using pumped flow through a heat exchanger.	ii) Testing will be performed to confirm that each SFS pump provides flow through its heat exchanger when taking suction from the SFP and returning flow to the SFP.	ii) Each SFS pump produces at least 900 gpm through its heat exchanger.
		9. Safety-related displays identified in Table 2.3.7-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.3.7-1 can be retrieved in the MCR.
		10. Controls exist in the MCR to cause the pumps identified in Table 2.3.7-3 to perform their listed functions.	Testing will be performed to actuate the pumps identified in Table 2.3.7-3 using controls in the MCR.	Controls in the MCR cause pumps identified in Table 2.3.7-3 to perform the listed functions.
		11. Displays of the SFS parameters identified in Table 2.3.7-3 can be retrieved in the MCR.	Inspection will be performed for retrievability in the MCR of the displays identified in Table 2.3.7-3.	Displays of the SFS parameters identified in Table 2.3.7-3 are retrieved in the MCR.
***				
410	2.3.07.08.ii	Not used per Amendment No. XX		
411	2.3.07.09	Not used per Amendment No. XX		
412	2.3.07.10	Not used per Amendment No. XX		
413	2.3.07.11	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.8, Service Water System, Table 2.3.8-2 as shown below:

Table 2.3.8-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
415	2.3.08.02.i	2. The SWS provides the nonsafety-related function of transferring heat from the component cooling water system to the surrounding atmosphere to support plant shutdown and spent fuel pool cooling.	i) Testing will be performed to confirm that the SWS can provide cooling water to the CCS heat exchangers.	i) Each SWS pump can provide at least 10,000 gpm of cooling water through its CCS heat exchanger.
		3. Controls exist in the MCR to cause the components identified in Table 2.3.8-1 to perform the listed function.	Testing will be performed on the components in Table 2.3.8-1 using controls in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.3.8-1 to perform the listed functions.
		4. Displays of the parameters identified in Table 2.3.8-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of parameters in the MCR.	The displays identified in Table 2.3.8-1 can be retrieved in the MCR.
***				
418	2.3.08.03	Not used per Amendment No. XX		
419	2.3.08.04	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1) Section 2.3.9, Containment Hydrogen Control System Table 2.3.9-3 as shown below:

Table 2.3.9-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
423	2.3.09.03.i	Not used per Amendment No. XX		
424	2.3.09.03.ii	3. The VLS provides the nonsafety-related function to control the containment hydrogen concentration for beyond design basis accidents.	i) Inspection for the number of igniters will be performed.	i) At least 66 hydrogen igniters are provided inside containment at the locations specified in Table 2.3.9-2.
			ii) Operability testing will be performed on the igniters.	ii) The surface temperature of the igniter meets or exceeds 1700°F.
		4.a) Controls exist in the MCR to cause the components identified in Table 2.3.9-2 to perform the listed function.	Testing will be performed on the igniters using the controls in the MCR.	Controls in the MCR operate to energize the igniters.
		5. Displays of the parameters identified in Table 2.3.9-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.3.9-1 in the MCR.	Displays identified in Table 2.3.9-1 can be retrieved in the MCR.
***				
427	2.3.09.04a	Not used per Amendment No. XX		
***				
429	2.3.09.05	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.10, Liquid Radwaste System, Table 2.3.10-4 as shown below:

Table 2.3.10-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
443	2.3.10.07a.i	Not used per Amendment No. XX		
444	2.3.10.07a.ii	7.a) The WLS provides the nonsafety-related function of detecting leaks within containment to the containment sump.	i) Inspection will be performed for retrievability of the displays of containment sump level channels WLS-034, WLS-035, and WLS-036 in the MCR.	i) Nonsafety-related displays of WLS containment sump level channels WLS-034, WLS-035, and WLS-036 can be retrieved in the MCR.
			ii) Testing will be performed by adding water to the sump and observing display of sump level.	ii) A report exists and concludes that sump level channels WLS-034, WLS-035, and WLS-036 can detect a change of 1.75 ± 0.1 inches.
		7.b) The WLS provides the nonsafety-related function of controlling releases of radioactive materials in liquid effluents.	Tests will be performed to confirm that a simulated high radiation signal from the discharge radiation monitor, WLS-RE-229, causes the discharge isolation valve WLS-PL-V223 to close.	A simulated high radiation signal causes the discharge control isolation valve WLS-PL-V223 to close.
		8. Controls exist in the MCR to cause the remotely operated valve identified in Table 2.3.10-3 to perform its active function.	Stroke testing will be performed on the remotely operated valve listed in Table 2.3.10-3 using controls in the MCR.	Controls in the MCR operate to cause the remotely operated valve to perform its active function.
		9. The check valves identified in Table 2.3.10-1 perform an active safety-related function to change position as indicated in the table.	Exercise testing of the check valves with active safety functions identified in Table 2.3.10-1 will be performed under pre-operational test pressure, temperature and flow conditions.	Each check valve changes position as indicated on Table 2.3.10-1.
		10. Displays of the parameters identified in Table 2.3.10-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.3.10-3 in the MCR.	Displays identified in Table 2.3.10-3 can be retrieved in the MCR.
445	2.3.10.07b	Not used per Amendment No. XX		
446	2.3.10.08	Not used per Amendment No. XX		
447	2.3.10.09	Not used per Amendment No. XX		

Table 2.3.10-4 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
448	2.3.10.10	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.13, Primary Sampling System, Table 2.3.13-3 as shown below:

Table 2.3.13-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
470	2.3.13.08	8. The PSS provides the nonsafety-related function of providing the capability of obtaining reactor coolant and containment atmosphere samples.	Testing will be performed to obtain samples of the reactor coolant and containment atmosphere.	A sample is drawn from the reactor coolant and the containment atmosphere.
		9. Safety-related displays identified in Table 2.3.13-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	The safety-related displays identified in Table 2.3.13-1 can be retrieved in the MCR.
		10.a) Controls exist in the MCR to cause those remotely operated valves identified in Table 2.3.13-1 to perform active functions.	Stroke testing will be performed on the remotely operated valves identified in Table 2.3.13-1 using the controls in the MCR.	Controls in the MCR operate to cause those remotely operated valves identified in Table 2.3.13-1 to perform active functions.
		10.b) The valves identified in Table 2.3.13-1 as having PMS control perform an active function after receiving a signal from the PMS.	Testing will be performed on remotely operated valves listed in Table 2.3.13-1 using real or simulated signals into the PMS.	The remotely operated valves identified in Table 2.3.13-1 as having PMS control perform the active function identified in the table after receiving a signal from the PMS.
		11.b) After loss of motive power, the remotely operated valves identified in Table 2.3.13-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	After loss of motive power, each remotely operated valve identified in Table 2.3.13-1 assumes the indicated loss of motive power position.
		12. Controls exist in the MCR to cause the valves identified in Table 2.3.13-2 to perform the listed function.	Testing will be performed on the components in Table 2.3.13-2 using controls in the MCR.	Controls in the MCR cause valves identified in Table 2.3.13-2 to perform the listed functions.
471	2.3.13.09	Not used per Amendment No. XX		
472	2.3.13.10a	Not used per Amendment No. XX		
473	2.3.13.10b	Not used per Amendment No. XX		
***				
475	2.3.13.11b	Not used per Amendment No. XX		
476	2.3.13.12	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.19, Communication System, Table 2.3.19-2 as shown below:

Table 2.3.19-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
484	2.3.19.01a	Not used per Amendment No. XX		
485	2.3.19.01b	Not used per Amendment No. XX		
486	2.3.19.02a	1.a) The EFS has handsets, amplifiers, loudspeakers, and siren tone generators connected as a telephone/page system.	Inspection of the as-built system will be performed.	The as-built EFS has handsets, amplifiers, loudspeakers, and siren tone generators connected as a telephone/page system.
		1.b) The EFS has sound-powered equipment connected as a system.	Inspection of the as-built system will be performed.	The as-built EFS has sound-powered equipment connected as a system.
		2.a) The EFS telephone/page system provides intraplant, station-to-station communications and area broadcasting between the MCR and the locations listed in Table 2.3.19-1.	An inspection and test will be performed on the telephone/page communication equipment.	Telephone/page equipment is installed and voice transmission and reception from the MCR are accomplished.
		2.b) EFS provides sound-powered communications between the MCR, the RSW, the Division A, B, C, D dc equipment rooms (Rooms 12201/12203/12205/ 12207), the Division A, B, C, D I&C rooms (Rooms 12301/12302/ 12304/12305), and the diesel generator building (Rooms 60310/60320) without external power.	An inspection and test will be performed of the sound-powered communication equipment.	Sound-powered equipment is installed and voice transmission and reception are accomplished.
487	2.3.19.02b	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.3.29, Radioactive Waste Drain System, Table 2.3.29-1 as shown below:

Table 2.3.29-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
489	2.3.29.02	2. The WRS collects liquid wastes from the equipment and floor drainage of the radioactive portions of the auxiliary building, annex building, and radwaste building and directs these wastes to a WRS sump or WLS waste holdup tanks located in the auxiliary building.	A test is performed by pouring water into the equipment and floor drains in the radioactive portions of the auxiliary building, annex building, and radwaste building.	The water poured into these drains is collected either in the auxiliary building radioactive drains sump or the WLS waste holdup tanks.
		3. The WRS collects chemical wastes from the auxiliary building chemical laboratory drains and the decontamination solution drains in the annex building and directs these wastes to the chemical waste tank of the liquid radwaste system.	A test is performed by pouring water into the auxiliary building chemical laboratory and the decontamination solution drains in the annex building.	The water poured into these drains is collected in the chemical waste tank of the liquid radwaste system.
490	2.3.29.03	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.4.1, Main and Startup Feedwater System, Table 2.4.1-2 as shown below:

Table 2.4.1-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
493	2.4.01.02	2. The FWS provides startup feedwater flow from the CST to the SGS for heat removal from the RCS.	Testing will be performed to confirm that each of the startup feedwater pumps can provide water from the CST to both steam generators.	Each FWS startup feedwater pump provides a flow rate greater than or equal to 260 gpm to each steam generator system at a steam generator secondary side pressure of at least 1106 psia.
		3. Controls exist in the MCR to cause the components identified in Table 2.4.1-1 to perform the listed function.	Testing will be performed on the components in Table 2.4.1-1 using controls in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.4.1-1 to perform the listed functions.
		4. Displays of the parameters identified in Table 2.4.1-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of parameters in the MCR.	The displays identified in Table 2.4.1-1 can be retrieved in the MCR.
494	2.4.01.03	Not used per Amendment No. XX		
495	2.4.01.04	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.4.2, Main Turbine System, Table 2.4.2-1 as shown below:

Table 2.4.2-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
497	2.4.02.02a	2.a) Controls exist in the MCR to trip the main turbine-generator. 2.c) The main turbine-generator trips after receiving a signal from the DAS. 3) The trip signals from the two turbine electrical overspeed protection trip systems are isolated from, and independent of, each other.	Testing will be performed on the main turbine-generator using controls in the MCR. Testing will be performed using real or simulated signals into the DAS. ii) Testing of the as-built system will be performed using simulated signals from the turbine speed sensors.	Controls in the MCR operate to trip the main turbine-generator. The main turbine-generator trips after receiving a signal from the DAS. ii) The main turbine-generator trips after overspeed signals are received from the speed sensors of the 110% emergency electrical overspeed trip system, and the main turbine-generator trips after overspeed signals are received from the speed sensors of the 111% backup electrical overspeed trip system.
***				
499	2.4.02.02c	Not used per Amendment No. XX		
***				
501	2.4.02.03.ii	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.5.1, Diverse Actuation System, Table 2.5.1-4 as shown below:

Table 2.5.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
506	2.5.01.02a	2.a) The DAS provides an automatic reactor trip on low wide-range steam generator water level, or on low pressurizer water level, or on high hot leg temperature, separate from the PMS.	Electrical power to the PMS equipment will be disconnected and an operational test of the as-built DAS will be performed using real or simulated test signals.	The generator field control relays (contained in the control cabinets for the rod drive motor-generator sets) open after the test signal reaches the specified limit.
		2.b) The DAS provides automatic actuation of selected functions, as identified in Table 2.5.1-1, separate from the PMS.	Electrical power to the PMS equipment will be disconnected and an operational test of the as-built DAS will be performed using real or simulated test signals.	Appropriate DAS output signals are generated after the test signal reaches the specified limit.
		2.c) The DAS provides manual initiation of reactor trip, and selected functions, as identified in Table 2.5.1-2, separate from the PMS. These manual initiation functions are implemented in a manner that bypasses the control room multiplexers, if any; the PMS cabinets; and the signal processing equipment of the DAS.	Electrical power to the control room multiplexers, if any, and PMS equipment will be disconnected and the outputs from the DAS signal processing equipment will be disabled. While in this configuration, an operational test of the as-built system will be performed using the DAS manual actuation controls.	i) The generator field control relays (contained in the control cabinets for the rod drive motor-generator sets) open after reactor and turbine trip manual initiation controls are actuated.
			Electrical power to the control room multiplexers, if any, and PMS equipment will be disconnected and the outputs from the DAS signal processing equipment will be disabled. While in this configuration, an operational test of the as-built system will be performed using the DAS manual actuation controls.	ii) DAS output signals are generated for the selected functions, as identified in Table 2.5.1-2, after manual initiation controls are actuated.
		2.d) The DAS provides MCR displays of selected plant parameters, as identified in Table 2.5.1-3, separate from the PMS.	Electrical power to the PMS equipment will be disconnected and inspection will be performed for retrievability of the selected plant parameters in the MCR.	The selected plant parameters can be retrieved in the MCR.

Table 2.5.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		3.f) The DAS is powered by non-Class 1E uninterruptible power supplies that are independent and separate from the power supplies which power the PMS. 3.g) The DAS signal processing cabinets are provided with the capability for channel testing without actuating the controlled components.	Electrical power to the PMS equipment will be disconnected. While in this configuration, a test will be performed by providing simulated test signals in the non-Class 1E uninterruptible power supplies. Channel tests will be performed on the as built system.	A simulated test signal exists at the DAS equipment when the assigned non-Class 1E uninterruptible power supply is provided the test signal. The capability exists for testing individual DAS channels without propagating an actuation signal to a DAS controlled component.
507	2.5.01.02b	Not used per Amendment No. XX		
508	2.5.01.02c.i	Not used per Amendment No. XX		
509	2.5.01.02c.ii	Not used per Amendment No. XX		
510	2.5.01.02d	Not used per Amendment No. XX		
***				
516	2.5.01.03f	Not used per Amendment No. XX		
517	2.5.01.03g	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.5.2, Protection and Safety Monitoring System, Table 2.5.2-8 as shown below:

Table 2.5.2-8
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
530	2.5.02.06a.ii	6.a) The PMS initiates an automatic reactor trip, as identified in Table 2.5.2-2, when plant process signals reach specified limits.	An operational test of the as-built PMS will be performed using real or simulated test signals.	ii) PMS output signals to the reactor trip switchgear are generated after the test signal reaches the specified limit. This needs to be verified for each automatic reactor trip function.
		6.b) The PMS initiates automatic actuation of engineered safety features, as identified in Table 2.5.2-3, when plant process signals reach specified limits.	An operational test of the as-built PMS will be performed using real or simulated test signals.	Appropriate PMS output signals are generated after the test signal reaches the specified limit. These output signals remain following removal of the test signal. Tests from the actuation signal to the actuated device(s) are performed as part of the system-related inspection, test, analysis, and acceptance criteria.
		6.c) The PMS provides manual initiation of reactor trip and selected engineered safety features as identified in Table 2.5.2-4.	An operational test of the as-built PMS will be performed using the PMS manual actuation controls.	ii) PMS output signals are generated for reactor trip and selected engineered safety features as identified in Table 2.5.2-4 after the manual initiation controls are actuated.
		8.a) The PMS provides for the minimum inventory of displays, visual alerts, and fixed position controls, as identified in Table 2.5.2-5. The plant parameters listed with a "Yes" in the "Display" column and visual alerts listed with a "Yes" in the "Alert" column can be retrieved in the MCR. The fixed position controls listed with a "Yes" in the "Control" column are provided in the MCR.	i) An inspection will be performed for retrievability of plant parameters in the MCR.	i) The plant parameters listed in Table 2.5.2-5 with a "Yes" in the "Display" column, can be retrieved in the MCR.

Table 2.5.2-8
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
			iii) An operational test of the as-built system will be performed using each MCR fixed position control.	iii) For each test of an as-built fixed position control listed in Table 2.5.2-5 with a "Yes" in the "Control" column, an actuation signal is generated. Tests from the actuation signal to the actuated device(s) are performed as part of the system-related inspection, test, analysis and acceptance criteria.
		8.c) Displays of the open/closed status of the reactor trip breakers can be retrieved in the MCR.	Inspection will be performed for retrievability of displays of the open/closed status of the reactor trip breakers in the MCR.	Displays of the open/closed status of the reactor trip breakers can be retrieved in the MCR.
		9.a) The PMS automatically removes blocks of reactor trip and engineered safety features actuation when the plant approaches conditions for which the associated function is designed to provide protection. These blocks are identified in Table 2.5.2-6.	An operational test of the as-built PMS will be performed using real or simulated test signals.	The PMS blocks are automatically removed when the test signal reaches the specified limit.
		9.b) The PMS two-out-of-four initiation logic reverts to a two-out-of-three coincidence logic if one of the four channels is bypassed. All bypassed channels are alarmed in the MCR.	An operational test of the as-built PMS will be performed.	The PMS two-out-of-four initiation logic reverts to a two-out-of-three coincidence logic if one of the four channels is bypassed. All bypassed channels are alarmed in the MCR.
		9.c) The PMS does not allow simultaneous bypass of two redundant channels.	An operational test of the as-built PMS will be performed. With one channel in bypass, an attempt will be made to place a redundant channel in bypass.	The redundant channel cannot be placed in bypass.
531	2.5.02.06b	Not used per Amendment No. XX		
***				
533	2.5.02.06c.ii	Not used per Amendment No. XX		
***				
539	2.5.02.08a.i	Not used per Amendment No. XX		
***				

Table 2.5.2-8
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
541	2.5.02.08a.iii	Not used per Amendment No. XX		
***				
544	2.5.02.08c	Not used per Amendment No. XX		
545	2.5.02.09a	Not used per Amendment No. XX		
546	2.5.02.09b	Not used per Amendment No. XX		
547	2.5.02.09c	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 2.5.4, Data Display and Processing System, Table 2.5.4-2 as shown below:

*Note: Changes to ITAAC Nos. 2.5.04.02.i, 2.5.04.02.ii and 2.5.04.02.iii in Table 2.5.4-2, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 2.i, 2ii and 2.iii in Table 2.5.4-2 is needed.

Table 2.5.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
557	2.5.04.02.i	2. The DDS provides for the minimum inventory of displays, visual alerts, and fixed position controls, as identified in Table 2.5.4-1. The plant parameters listed with a "Yes" in the "Display" column and visual alerts listed with a "Yes" in the "Alert" column can be retrieved at the RSW. The controls listed with a "Yes" in the "Control" column are provided at the RSW.	i) An inspection will be performed for retrievability of plant parameters at the RSW. ii) An inspection and test will be performed to verify that the plant parameters are used to generate visual alerts that identify challenges to critical safety functions. iii) An operational test of the as-built system will be performed using each RSW control.	i) The plant parameters listed in Table 2.5.4-1 with a "Yes" in the "Display" column can be retrieved at the RSW. ii) The plant parameters listed in Table 2.5.4-1 with a "Yes" in the "Alert" column are used to generate visual alerts that identify challenges to critical safety functions. The visual alerts actuate in accordance with their logic and values. iii) For each test of a control listed in Table 2.5.4-1 with a "Yes" in the "Control" column, an actuation signal is generated. Tests from the actuation signal to the actuated device(s) are performed as part of the system-related inspection, test, analysis and acceptance criteria.
558	2.5.04.02.ii	Not used per Amendment No. XX		
559	2.5.04.02.iii	Not used per Amendment No. XX		
***				

Table 2.5.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
561	C.2.5.04.04a	4. The plant calorimetric uncertainty and plant instrumentation performance is bounded by the 1% calorimetric uncertainty value assumed for the initial reactor power in the safety analysis.	Inspection will be performed of the plant operating instrumentation installed for feedwater flow measurement, its associated power calorimetric uncertainty calculation, and the calculated calorimetric values.	a) The as-built system takes input for feedwater flow measurement from a Caldon [Cameron] LEFM CheckPlus™ System;
			Inspection will be performed of the plant operating instrumentation installed for feedwater flow measurement, its associated power calorimetric uncertainty calculation, and the calculated calorimetric values.	b) the power calorimetric uncertainty calculation documented for that instrumentation is based on an accepted Westinghouse methodology and the uncertainty values for that instrumentation are not lower than those for the actual installed instrumentation; and
			Inspection will be performed of the plant operating instrumentation installed for feedwater flow measurement, its associated power calorimetric uncertainty calculation, and the calculated calorimetric values.	c) the calculated calorimetric power uncertainty measurement values are bounded by the 1% uncertainty value assumed for the initial reactor power in the safety analysis.
562	C.2.5.04.04b	Not used per Amendment No. XX		
563	C.2.5.04.04c	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.1, Main ac Power System, Table 2.6.1-4 as shown below:

Table 2.6.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
584	2.6.01.04a	Not used per Amendment No. XX		
***				
588	2.6.01.04e	4.a) The ECS provides the capability for distributing non-Class 1E ac power from onsite sources (ZOS) to nonsafety-related loads listed in Table 2.6.1-2.	Tests will be performed using a test signal to confirm that an electrical path exists for each selected load listed in Table 2.6.1-2 from an ECS-ES-1 or ECS-ES-2 bus. Each test may be a single test or a series of over-lapping tests.	A test signal exists at the terminals of each selected load.
		4.e) The ECS provides two loss-of-voltage signals to the onsite standby power system (ZOS), one for each diesel-backed 6900 Vac switchgear bus.	Tests on the as-built ECS system will be conducted by simulating a loss-of-voltage condition on each diesel-backed 6900 Vac switchgear bus.	A loss-of-voltage signal is generated when the loss-of-voltage condition is simulated.
		4.f) The ECS provides a reverse-power trip of the generator circuit breaker which is blocked for at least 15 seconds following a turbine trip.	Tests on the as-built ECS system will be conducted by simulating a turbine trip signal followed by a simulated reverse-power condition. The generator circuit breaker trip signal will be monitored.	The generator circuit breaker trip signal does not occur until at least 15 seconds after the simulated turbine trip.
		5. Controls exist in the MCR to cause the circuit breakers identified in Table 2.6.1-3 to perform the listed functions.	Tests will be performed to verify that controls in the MCR can operate the circuit breakers identified in Table 2.6.1-3.	Controls in the MCR cause the circuit breakers identified in Table 2.6.1-3 to operate.
		6. Displays of the parameters identified in Table 2.6.1-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.6.1-3 in the MCR.	Displays identified in Table 2.6.1-3 can be retrieved in the MCR.
589	2.6.01.04f	Not used per Amendment No. XX		
590	2.6.01.05	Not used per Amendment No. XX		
591	2.6.01.06	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.3, Class 1E dc and Uninterruptible Power Supply System, Table 2.6.3-3 as shown below:

*Note: Changes to ITAAC Nos. 2.6.03.05d.i and 2.6.03.05d.ii in Table 2.6.3-3, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 5d.i and 5d.ii in Table 2.6.3-3 is needed.

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
603	2.6.03.04c	4.c) Each IDS 24-hour battery bank supplies a dc switchboard bus load for a period of 24 hours without recharging.	Testing of each 24-hour as-built battery bank will be performed by applying a simulated or real load, or a combination of simulated or real loads which envelope the battery bank design duty cycle. The test will be conducted on a battery bank that has been fully charged and has been connected to a battery charger maintained at 270±2 V for a period of no less than 24 hours prior to the test.	The battery terminal voltage is greater than or equal to 210 V after a period of no less than 24 hours with an equivalent load that equals or exceeds the battery bank design duty cycle capacity.
		4.d) Each IDS 72-hour battery bank supplies a dc switchboard bus load for a period of 72 hours without recharging.	Testing of each 72-hour as-built battery bank will be performed by applying a simulated or real load, or a combination of simulated or real loads which envelope the battery bank design duty cycle. The test will be conducted on a battery bank that has been fully charged and has been connected to a battery charger maintained at 270±2 V for a period of no less than 24 hours prior to the test.	The battery terminal voltage is greater than or equal to 210 V after a period of no less than 72 hours with an equivalent load that equals or exceeds the battery bank design duty cycle capacity.

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		4.e) The IDS spare battery bank supplies a dc load equal to or greater than the most severe switchboard bus load for the required period without recharging.	Testing of the as-built spare battery bank will be performed by applying a simulated or real load, or a combination of simulated or real loads which envelope the most severe of the division batteries design duty cycle. The test will be conducted on a battery bank that has been fully charged and has been connected to a battery charger maintained at 270±2 V for a period of no less than 24 hours prior to the test.	The battery terminal voltage is greater than or equal to 210 V after a period with a load and duration that equals or exceeds the most severe battery bank design duty cycle capacity.
		4.f) Each IDS 24-hour inverter supplies its ac load.	Testing of each 24-hour as-built inverter will be performed by applying a simulated or real load, or a combination of simulated or real loads, equivalent to a resistive load greater than 12 kW. The inverter input voltage will be no more than 210 Vdc during the test.	Each 24-hour inverter supplies a line-to-line output voltage of 208 ± 2% V at a frequency of 60 ± 0.5% Hz.
		4.g) Each IDS 72-hour inverter supplies its ac load.	Testing of each 72-hour as-built inverter will be performed by applying a simulated or real load, or a combination of simulated or real loads, equivalent to a resistive load greater than 7 kW. The inverter input voltage will be no more than 210 Vdc during the test.	Each 72-hour inverter supplies a line-to-line output voltage of 208 ± 2% V at a frequency of 60 ± 0.5% Hz.
		4.h) Each IDS 24-hour battery charger provides the PMS with two loss-of-ac input voltage signals.	Testing will be performed by simulating a loss of input voltage to each 24-hour battery charger.	Two PMS input signals exist from each 24-hour battery charger indicating loss of ac input voltage when the loss-of-input voltage condition is simulated.
		5.a) Each IDS 24-hour battery charger supplies a dc switchboard bus load while maintaining the corresponding battery charged.	Testing of each as-built 24-hour battery charger will be performed by applying a simulated or real load, or a combination of simulated or real loads.	Each 24-hour battery charger provides an output current of at least 150 A with an output voltage in the range 210 to 280 V.

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		5.b) Each IDS 72-hour battery charger supplies a dc switchboard bus load while maintaining the corresponding battery charged.	Testing of each 72-hour as-built battery charger will be performed by applying a simulated or real load, or a combination of simulated or real loads.	Each 72-hour battery charger provides an output current of at least 125 A with an output voltage in the range 210 to 280 V.
		5.c) Each IDS regulating transformer supplies an ac load when powered from the 480 V MCC.	Testing of each as-built regulating transformer will be performed by applying a simulated or real load, or a combination of simulated or real loads, equivalent to a resistive load greater than 30 kW when powered from the 480 V MCC.	Each regulating transformer supplies a line-to-line output voltage of 208 ± 2% V.
		6. Safety-related displays identified in Table 2.6.3-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.6.3-1 can be retrieved in the MCR.
		11. Displays of the parameters identified in Table 2.6.3-2 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays identified in Table 2.6.3-2 in the MCR.	Displays identified in Table 2.6.3-2 can be retrieved in the MCR.
604	2.6.03.04d	Not used per Amendment No. XX		
605	2.6.03.04e	Not used per Amendment No. XX		
606	2.6.03.04f	Not used per Amendment No. XX		
607	2.6.03.04g	Not used per Amendment No. XX		
608	2.6.03.04h	Not used per Amendment No. XX		
***				
610	2.6.03.05a	Not used per Amendment No. XX		
611	2.6.03.05b	Not used per Amendment No. XX		
612	2.6.03.05c	Not used per Amendment No. XX		
613	2.6.03.05d.i	5.d) The IDS Divisions B and C regulating transformers supply their post-72-hour ac loads when powered from an ancillary diesel generator.	Inspection of the as-built system will be performed.	i) Ancillary diesel generator 1 is electrically connected to regulating transformer IDSC-DT-1
			Inspection of the as-built system will be performed.	ii) Ancillary diesel generator 2 is electrically connected to regulating transformer IDSB-DT-1.

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
614	2.6.03.05d.ii	Not used per Amendment No. XX		
615	2.6.03.06	Not used per Amendment No. XX		
616	2.6.03.07	7. The IDS dc battery fuses and battery charger circuit breakers, and dc distribution panels, MCCs, and their circuit breakers and fuses, are sized to supply their load requirements.	Analyses for the as-built IDS dc electrical distribution system to determine the capacities of the battery fuses and battery charger circuit breakers, and dc distribution panels, MCCs, and their circuit breakers and fuses, will be performed.	Analyses for the as-built IDS dc electrical distribution system exist and conclude that the capacities of as-built IDS battery fuses and battery charger circuit breakers, and dc distribution panels, MCCs, and their circuit breakers and fuses, as determined by their nameplate ratings, exceed their analyzed load requirements.
		8. Circuit breakers and fuses in IDS battery, battery charger, dc distribution panel, and MCC circuits are rated to interrupt fault currents.	Analyses for the as-built IDS dc electrical distribution system to determine fault currents will be performed.	Analyses for the as-built IDS dc electrical distribution system exist and conclude that the analyzed fault currents do not exceed the interrupt capacity of circuit breakers and fuses in the battery, battery charger, dc distribution panel, and MCC circuits, as determined by their nameplate ratings.
		9. The IDS batteries, battery chargers, dc distribution panels, and MCCs are rated to withstand fault currents for the time required to clear the fault from its power source.	Analyses for the as-built IDS dc electrical distribution system to determine fault currents will be performed.	Analyses for the as-built IDS dc electrical distribution system exist and conclude that the fault current capacities of as-built IDS batteries, battery chargers, dc distribution panels, and MCCs, as determined by manufacturer's ratings, exceed their analyzed fault currents for the time required to clear the fault from its power source as determined by the circuit interrupting device coordination analyses.

Table 2.6.3-3
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		10. The IDS electrical distribution system cables are rated to withstand fault currents for the time required to clear the fault from its power source.	Analyses for the as-built IDS dc electrical distribution system to determine fault currents will be performed.	Analyses for the as-built IDS dc electrical distribution system exist and conclude that the IDS dc electrical distribution system cables will withstand the analyzed fault currents, as determined by manufacturer's ratings, for the time required to clear the fault from its power source as determined by the circuit interrupting device coordination analyses.
617	2.6.03.08	Not used per Amendment No. XX		
618	2.6.03.09	Not used per Amendment No. XX		
619	2.6.03.10	Not used per Amendment No. XX		
620	2.6.03.11	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), 2.6.4, Onsite Standby Power System, Table 2.6.4-1 as shown below:

Table 2.6.4-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
622	2.6.04.02a	2.a) On loss of power to a 6900 volt diesel-backed bus, the associated diesel generator automatically starts and produces ac power at rated voltage and frequency. The source circuit breakers and bus load circuit breakers are opened, and the generator is connected to the bus.	Tests on the as-built ZOS system will be conducted by providing a simulated loss-of-voltage signal. The starting air supply receiver will not be replenished during the test.	Each as-built diesel generator automatically starts on receiving a simulated loss-of-voltage signal and attains a voltage of 6900 ± 10% V and frequency 60 ± 5% Hz after the start signal is initiated and opens ac power system breakers on the associated 6900 V bus.
		2.b) Each diesel generator unit is sized to supply power to the selected nonsafety-related electrical components.	Each diesel generator will be operated with a load of 4000 kW or greater and a power factor between 0.9 and 1.0 for a time period required to reach engine temperature equilibrium plus 2.5 hours.	Each diesel generator provides power to the load with a generator terminal voltage of 6900 ± 10% V and a frequency of 60 ± 5% Hz.
		3. Displays of diesel generator status (running/not running) and electrical output power (watts) can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays in the MCR.	Displays of diesel generator status and electrical output power can be retrieved in the MCR.
		4. Controls exist in the MCR to start and stop each diesel generator.	A test will be performed to verify that controls in the MCR can start and stop each diesel generator.	Controls in the MCR operate to start and stop each diesel generator.
623	2.6.04.02b	Not used per Amendment No. XX		
***				
625	2.6.04.03	Not used per Amendment No. XX		
626	2.6.04.04	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.5, Lighting System, Table 2.6.5-1 as shown below:

Table 2.6.5-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
628	2.6.05.02.i	Not used per Amendment No. XX		
629	2.6.05.02.ii	2. The ELS has six groups of emergency lighting fixtures located in the MCR and at the RSW. Each group is powered by one of the Class 1E inverters. The ELS has four groups of panel lighting fixtures located on or near safety panels in the MCR. Each group is powered by one of the Class 1E inverters in Divisions B and C (one 24-hour and one 72-hour inverter in each Division).	i) Inspection of the as-built system will be performed.	i) The as-built ELS has six groups of emergency lighting fixtures located in the MCR and at the RSW. The ELS has four groups of panel lighting fixtures located on or near safety panels in the MCR.
			ii) Testing of the as-built system will be performed using one Class 1E inverter at a time.	ii) Each of the six as-built emergency lighting groups is supplied power from its respective Class 1E inverter and each of the four as-built panel lighting groups is supplied power from its respective Class 1E inverter.
		5. The normal lighting can provide 50 foot candles at the safety panel and at the workstations in the MCR and at the RSW.	i) Testing of the as-built normal lighting in the MCR will be performed.	i) When adjusted for maximum illumination and powered by the main ac power system, the normal lighting in the MCR provides at least 50 foot candles at the safety panel and at the workstations.
			ii) Testing of the as-built normal lighting at the RSW will be performed.	ii) When adjusted for maximum illumination and powered by the main ac power system, the normal lighting in the RSW provides at least 50 foot candles at the safety panel and at the workstations.

Table 2.6.5-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		6. The emergency lighting can provide 10 foot candles at the safety panel and at the workstations in the MCR and at the RSW.	i) Testing of the as-built emergency lighting in the MCR will be performed. ii) Testing of the as-built emergency lighting at the RSW will be performed.	i) When adjusted for maximum illumination and powered by the six Class 1E inverters, the emergency lighting in the MCR provides at least 10 foot candles at the safety panel and at the workstations. ii) When adjusted for maximum illumination and powered by the six Class 1E inverters, the emergency lighting provides at least 10 foot candles at the RSW.
***				
633	2.6.05.05.i	Not used per Amendment No. XX		
634	2.6.05.05.ii	Not used per Amendment No. XX		
635	2.6.05.06.i	Not used per Amendment No. XX		
636	2.6.05.06.ii	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.6, Grounding and Lightning Protection System, Table 2.6.6-1 as shown below:

*Note: Changes to ITAAC Nos. 2.6.06.01.i, 2.6.06.01.ii, 2.6.06.01.iii and 2.6.06.01.iv in Table 2.6.6-1, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 1.i, 1ii, 1.iii and 1.iv in Table 2.6.6-1 is needed.

Table 2.6.6-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
637	2.6.06.01.i	1. The EGS provides an electrical grounding system for: (1) instrument/computer grounding; (2) electrical system grounding of the neutral points of the main generator, main step-up transformers, auxiliary transformers, load center transformers, auxiliary and onsite standby diesel generators; and (3) equipment grounding of equipment enclosures, metal structures, metallic tanks, ground bus of switchgear assemblies, load centers, motor control centers, and control cabinets. Lightning protection is provided for exposed structures and buildings housing safety-related and fire protection equipment. Each grounding system and lightning protection system is grounded to the station grounding grid.	i) An inspection for the instrument/computer grounding system connection to the station grounding grid will be performed.	i) A connection exists between the instrument/computer grounding system and the station grounding grid.
			ii) An inspection for the electrical system grounding connection to the station grounding grid will be performed.	ii) A connection exists between the electrical system grounding and the station grounding grid.
			iii) An inspection for the equipment grounding system connection to the station grounding grid will be performed.	iii) A connection exists between the equipment grounding system and the station grounding grid.
			iv) An inspection for the lightning protection system connection to the station grounding grid will be performed.	iv) A connection exists between the lightning protection system and the station grounding grid.
638	2.6.06.01.ii	Not used per Amendment No. XX		

Table 2.6.6-1 Inspections, Tests, Analyses, and Acceptance Criteria				
No.	**ITAAC No.**	**Design Commitment**	**Inspections, Tests, Analyses**	**Acceptance Criteria**
639	2.6.06.01.iii	Not used per Amendment No. XX		
640	2.6.06.01.iv	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.6.9, Plant Security System, Table 2.6.9-1 as shown below:

Table 2.6.9-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
644	2.6.09.05a	5.a) Security alarm annunciation and video assessment information is displayed concurrently in the central alarm station and the secondary alarm station, and the video image recording with real time playback capability can provide assessment of activities before and after each alarm annunciation within the perimeter area barrier.	Test, inspection, or a combination of test and inspections of the installed systems will be performed.	Security alarm annunciation and video assessment information is displayed concurrently in the central alarm station and the secondary alarm station, and the video image recording with real time playback capability provides assessment of activities before and after alarm annunciation within the perimeter barrier.
		15.b) Intrusion detection and assessment systems concurrently provide visual displays and audible annunciation of alarms in the central and secondary alarm stations.	Tests will be performed on intrusion detection and assessment equipment.	The intrusion detection system concurrently provides visual displays and audible annunciations of alarms in both the central and secondary alarm stations.
***				
648	2.6.09.07a	Not used per Amendment No. XX		
649	2.6.09.07b	Not used per Amendment No. XX		
***				
651	2.6.09.09	Not used per Amendment No. XX		
***				

Table 2.6.9-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
652	2.6.09.13a	13.a) The central and secondary alarm stations have conventional (landline) telephone service with the main control room and local law enforcement authorities. 13.b) The central and secondary alarm stations are capable of continuous communication with security personnel.	Tests, inspections, or a combination of tests and inspections of the central and secondary alarm stations' conventional telephone services will be performed. Tests, inspections, or a combination of tests and inspections of the central and secondary alarm stations' continuous communication capabilities will be performed.	The central and secondary alarm stations are equipped with conventional (landline) telephone service with the main control room and local law enforcement authorities. The central and secondary alarm stations are equipped with the capability to continuously communicate with security officers, watchmen, armed response individuals, or any security personnel that have responsibilities during a contingency event.
653	2.6.09.13b	Not used per Amendment No. XX		
***				
655	2.6.09.15a	15.a) Security alarm devices, including transmission lines to annunciators, are tamper indicating and self-checking (e.g., an automatic indication is provided when failure of the alarm system or a component occurs, or when on standby power). Alarm annunciation shall indicate the type of alarm (e.g., intrusion alarms and emergency exit alarm) and location. 16. Equipment exists to record onsite security alarm annunciation, including the location of the alarm, false alarm, alarm check, and tamper indication; and the type of alarm, location, alarm circuit, date, and time.	A test will be performed to verify that security alarms, including transmission lines to annunciators, are tamper indicating and self-checking (e.g., an automatic indication is provided when failure of the alarm system or a component occurs, or when on standby power) and that alarm annunciation indicates the type of alarm (e.g., intrusion alarms and emergency exit alarms) and location. Test, analysis, or a combination of test and analysis will be performed to ensure that equipment is capable of recording each onsite security alarm annunciation, including the location of the alarm, false alarm, alarm check, and tamper indication; and the type of alarm, location, alarm circuit, date, and time.	A report exists and concludes that security alarm devices, including transmission lines to annunciators, are tamper indicating and self-checking (e.g., an automatic indication is provided when failure of the alarm system or a component occurs, or when the system is on standby power) and that alarm annunciation indicates the type of alarm (e.g., intrusion alarms and emergency exit alarms) and location. A report exists and concludes that equipment is capable of recording each onsite security alarm annunciation, including the location of the alarm, false alarm, alarm check, and tamper indication; and the type of alarm, location, alarm circuit, date, and time.
656	2.6.09.15b	Not used per Amendment No. XX		

Table 2.6.9-1
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
657	2.6.09.16	Not used per Amendment No. XX		

Revise COL Appendix C Section C.2.6.9, Physical Security, Table C.2.6.9-2 as shown below:

Table C.2.6.9-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
661	C.2.6.09.03b	3.b) The isolation zones are monitored with intrusion detection equipment that provides the capability to detect and assess unauthorized persons.	Inspections will be performed of the intrusion detection equipment within the isolation zones.	The isolation zones are equipped with intrusion detection equipment that provides the capability to detect and assess unauthorized persons.
		4. The intrusion detection and assessment equipment at the protected area perimeter: a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in both the Central Alarm Station and Secondary Alarm Station;	Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.	The intrusion detection and assessment equipment at the protected area perimeter: a) detects penetration or attempted penetration of the protected area barrier and concurrently alarms in the Central Alarm Station and Secondary Alarm Station;
		b) remains operable from an uninterruptible power supply in the event of the loss of normal power.	Tests, inspections or a combination of tests and inspections of the intrusion detection and assessment equipment at the protected area perimeter and its uninterruptible power supply will be performed.	b) remains operable from an uninterruptible power supply in the event of the loss of normal power.
662	C.2.6.09.04a	Not used per Amendment No. XX		
663	C.2.6.09.04b	Not used per Amendment No. XX		
664	C.2.6.09.05a	5. Access control points are established to: a) control personnel and vehicle access into the protected area.	Tests, inspections, or combination of tests and inspections of installed systems and equipment at the access control points to the protected area will be performed.	The access control points for the protected area: a) are configured to control personnel and vehicle access.
		b) detect firearms, explosives, and incendiary devices at the protected area personnel access points.	Tests, inspections, or combination of tests and inspections of installed systems and equipment at the access control points to the protected area will be performed.	b) include detection equipment that is capable of detecting firearms, incendiary devices, and explosives at the protected area personnel access points.
665	C.2.6.09.05b	Not used per Amendment No. XX		

Table C.2.6.9-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
667	C.2.6.09.07	7. Access to vital equipment physical barriers requires passage through the protected area perimeter barrier.	Inspection will be performed to confirm that access to vital equipment physical barriers requires passage through the protected area perimeter barrier.	Vital equipment is located within a protected area such that access to vital equipment physical barriers requires passage through the protected area perimeter barrier.
		7.a) Vital equipment is located only within a vital area.	Inspection will be performed to confirm that vital equipment is located within a vital area.	All vital equipment is located only within a vital area.
		7.b) Access to vital equipment requires passage through the vital area barrier.	Inspection will be performed to confirm that access to vital equipment requires passage through the vital area barrier.	Vital equipment is located within a protected area such that access to vital equipment requires passage through the vital area barrier.
668	C.2.6.09.08a	8.a) Penetrations through the protected area barrier are secured and monitored.	Inspections will be performed of penetrations through the protected area barrier.	Penetrations and openings through the protected area barrier are secured and monitored.
		8.b) Unattended openings (such as underground pathways) that intersect the protected area boundary or vital area boundary will be protected by a physical barrier and monitored by intrusion detection equipment or provided surveillance at a frequency sufficient to detect exploitation.	Inspections will be performed of unattended openings that intersect the protected area boundary or vital area boundary.	Unattended openings (such as underground pathways) that intersect the protected area boundary or vital area boundary are protected by a physical barrier and monitored by intrusion detection equipment or provided surveillance at a frequency sufficient to detect exploitation.
669	C.2.6.09.08b	Not used per Amendment No. XX		
670	C.2.6.09.09	9. Emergency exits through the protected area perimeter are alarmed and secured with locking devices to allow for emergency egress.	Tests, inspections, or a combination of tests and inspections of emergency exits through the protected area perimeter will be performed.	Emergency exits through the protected area perimeter are alarmed and secured by locking devices that allow prompt egress during an emergency.
		9. Emergency exits through the vital area boundaries are locked, alarmed, and equipped with a crash bar to allow for emergency egress.	Test, inspection, or a combination of tests and inspections of the emergency exits through the vital area boundaries will be performed.	The emergency exits through the vital area boundaries are locked, alarmed, and equipped with a crash bar to allow for emergency egress.

Revise COL Appendix C (and plant-specific Tier 1), Section 2.7.1, Nuclear Island Nonradioactive Ventilation System, Table 2.7.1-4 as shown below:

Table 2.7.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
693	2.7.01.08d	Not used per Amendment No. XX		
694	2.7.01.09	Not used per Amendment No. XX		
695	2.7.01.10a	Not used per Amendment No. XX		
696	2.7.01.10b	Not used per Amendment No. XX		
697	2.7.01.11	Not used per Amendment No. XX		
698	2.7.01.12	Not used per Amendment No. XX		
699	2.7.01.13	Not used per Amendment No. XX		
700	2.7.01.14	8.d) The VBS provides ventilation cooling via the ancillary equipment in Table 2.7.1-3 to the MCR and the division B&C Class 1E I&C rooms.	Testing will be performed on the components in Table 2.7.1-3.	The fans start and run.
		9. Safety-related displays identified in Table 2.7.1-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.7.1-1 can be retrieved in the MCR.
		10.a) Controls exist in the MCR to cause the remotely operated valves identified in Table 2.7.1-1 to perform their active functions.	Stroke testing will be performed on the remotely operated valves identified in Table 2.7.1-1 using the controls in the MCR.	Controls in the MCR operate to cause the remotely operated valves identified in Table 2.7.1-1 to perform their active functions.
		10.b) The valves identified in Table 2.7.1-1 as having PMS control perform their active safety function after receiving a signal from the PMS.	Testing will be performed using real or simulated signals into the PMS.	The valves identified in Table 2.7.1-1 as having PMS control perform their active safety function after receiving a signal from PMS.
		11. After loss of motive power, the remotely operated valves identified in Table 2.7.1-1 assume the indicated loss of motive power position.	Testing of the remotely operated valves will be performed under the conditions of loss of motive power.	Upon loss of motive power, each remotely operated valves identified in Table 2.7.1-1 assumes the indicated loss of motive power position.
		12. Controls exist in the MCR to cause the components identified in Table 2.7.1-3 to perform the listed function.	Testing will be performed on the components in Table 2.7.1-3 using controls in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.7.1-3 to perform the listed functions.

Table 2.7.1-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
		13. Displays of the parameters identified in Table 2.7.1-3 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.7.1-3 can be retrieved in the MCR.
		14. The background noise level in the MCR and RSR does not exceed 65 dB(A) when the VBS is operating.	The as-built VBS will be operated, and background noise levels in the MCR and RSR will be measured.	The background noise level in the MCR and RSR does not exceed 65 dB(A) when the VBS is operating.

Revise COL Appendix C (and plant-specific Tier 1) Section 2.7.2, Central Chilled Water System Table 2.7.2-2 as shown below:

Table 2.7.2-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
703	2.7.02.03a	3.a) The VWS provides chilled water to the supply air handling units serving the MCR, the Class 1E electrical rooms, and the unit coolers serving the RNS and CVS pump rooms.	Testing will be performed by measuring the flow rates to the chilled water cooling coils.	The water flow to each cooling coil equals or exceeds the following: Coil — Flow (gpm) VBS MY C01A/B 96 VBS MY C02A/C 97 VBS MY C02B/D 52 VAS MY C07A/B 12.3 VAS MY C12A/B 8.2 VAS MY C06A/B 8.2
		4. Controls exist in the MCR to cause the components identified in Table 2.7.2-1 to perform the listed function.	Testing will be performed on the components in Table 2.7.2-1 using controls in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.7.2-1 to perform the listed functions.
		5. Displays of the parameters identified in Table 2.7.2-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of parameters in the MCR.	The displays identified in Table 2.7.2-1 can be retrieved in the MCR.
***				
705	2.7.02.04	Not used per Amendment No. XX		
706	2.7.02.05	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.7.3, Annex/Auxiliary Building Nonradioactive Ventilation System, Table 2.7.3-2 as shown below:

Table 2.7.3-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
710	2.7.03.03	3. Controls exist in the MCR to cause the components identified in Table 2.7.3-1 to perform the listed function.	Testing will be performed on the components in Table 2.7.3-1 using controls in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.7.3-1 to perform the listed functions.
		4. Displays of the parameters identified in Table 2.7.3-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.7.3-1 can be retrieved in the MCR.
711	2.7.03.04	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.7.4, Diesel Generator Building Ventilation System, Table 2.7.4-2 as shown below:

Table 2.7.4-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
716	2.7.04.03	3. Controls exist in the MCR to cause the components identified in Table 2.7.4-1 to perform the listed function.	Testing will be performed on the components in Table 2.7.4-1 using controls in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.7.4-1 to perform the listed functions.
		4. Displays of the parameters identified in Table 2.7.4-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.7.4-1 can be retrieved in the MCR.
717	2.7.04.04	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1), Section 2.7.5, Radiologically Controlled Area Ventilation System Table 2.7.5-2 as shown below:

Table 2.7.5-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
719	2.7.05.02.i	2. The VAS maintains each building area at a slightly negative pressure relative to the atmosphere or adjacent clean plant areas.	i) Testing will be performed to confirm that the VAS maintains each building at a slightly negative pressure when operating all VAS supply AHUs and all VAS exhaust fans.	i) The time average pressure differential in the served areas of the annex, fuel handling and radiologically controlled auxiliary buildings as measured by each of the instruments identified in Table 2.7.5-1 is negative.
			ii) Testing will be performed to confirm the ventilation flow rate through the auxiliary building fuel handling area when operating all VAS supply AHUs and all VAS exhaust fans.	ii) A report exists and concludes that the calculated exhaust flow rate based on the measured flow rates is greater than or equal to 15,300 cfm.
			iii) Testing will be performed to confirm the auxiliary building radiologically controlled area ventilation flow rate when operating all VAS supply AHUs and all VAS exhaust fans.	iii) A report exists and concludes that the calculated exhaust flow rate based on the measured flow rates is greater than or equal to 22,500 cfm.
		3. Displays of the parameters identified in Table 2.7.5-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.7.5-1 can be retrieved in the MCR.
720	2.7.05.02.ii	Not used per Amendment No. XX		
721	2.7.05.02.iii	Not used per Amendment No. XX		
722	2.7.05.03	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1) Section 2.7.6, Containment Air Filtration System Table 2.7.6-2 as shown below:

Table 2.7.6-2
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
726	2.7.06.03.i	3. The VFS provides the intermittent flow of outdoor air to purge the containment atmosphere during normal plant operation, and continuous flow during hot or cold plant shutdown conditions.	i) Testing will be performed to confirm that containment supply AHU fan A when operated with containment exhaust fan A provides a flow of outdoor air.	i) The flow rate measured at each fan is greater than or equal to 3,600 scfm.
			ii) Testing will be performed to confirm that containment supply AHU fan B when operated with containment exhaust fan B provides a flow of outdoor air.	ii) The flow rate measured at each fan is greater than or equal to 3,600 scfm.
			iii) Inspection will be conducted of the containment purge discharge line (VFS-L204) penetrating the containment.	iii) The nominal line size is ≥ 36 in.
		4. Controls exist in the MCR to cause the components identified in Table 2.7.6-1 to perform the listed function.	Testing will be performed on the components in Table 2.7.6-1 using controls in the MCR.	Controls in the MCR operate to cause the components listed in Table 2.7.6-1 to perform the listed functions.
		5. Displays of the parameters identified in Table 2.7.6-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the parameters in the MCR.	The displays identified in Table 2.7.6-1 can be retrieved in the MCR.
727	2.7.06.03.ii	Not used per Amendment No. XX		
728	2.7.06.03.iii	Not used per Amendment No. XX		
729	2.7.06.04	Not used per Amendment No. XX		
730	2.7.06.05	Not used per Amendment No. XX		

Revise COL Appendix C (and plant-specific Tier 1) Section 3.3, Buildings Table 3.3-6 as shown below:

*Note: Changes to ITAAC Nos. 3.3.00.10ii and 3.3.00.10.iii in Table 3.3-6, shown below, only impact COL Appendix C. No change to corresponding plant-specific Tier 1 items 10.ii and 10.iii in Table 3.3-6 is needed.

Table 3.3-6
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
816	3.3.00.10.ii	10. The shield building roof and PCS storage tank support and retain the PCS water sources. The PCS storage tank has a stainless steel liner which provides a barrier on the inside surfaces of the tank. Leak chase channels are provided on the tank boundary liner welds.	ii) An inspection of the PCS storage tank exterior tank boundary and shield building tension ring will be performed before and after filling of the PCS storage tank to the overflow level. The vertical elevation of the shield building roof will be measured at a location at the outer radius of the roof (tension ring) and at a location on the same azimuth at the outer radius of the PCS storage tank before and after filling the PCS storage tank. iii) An inspection of the PCS storage tank exterior tank boundary and shield building tension ring will be performed before and after filling of the PCS storage tank to the overflow level. The boundaries of the PCS storage tank and the shield building roof above the tension ring will be inspected visually for excessive concrete cracking.	ii) A report exists and concludes that inspection and measurement of the PCS storage tank and the tension ring structure, before and after filling of the tank, shows structural behavior under normal loads to be acceptable. iii) A report exists and concludes that there is no visible water leakage from the PCS storage tank through the concrete and that there is no visible excessive cracking in the boundaries of the PCS storage tank and the shield building roof above the tension ring.
817	3.3.00.10.iii	Not used per Amendment No. XX		
***				

Revise COL Appendix C (and plant-specific Tier 1), Section 3.5, Radiation Monitoring, Table 3.5-6 as shown below:

Table 3.5-6
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
***				
829	3.5.00.04	Not used per Amendment No. XX		
830	3.5.00.05	Not used per Amendment No. XX		
831	3.5.00.06	4. Safety-related displays identified in Table 3.5-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the displays in the MCR.	Safety-related displays identified in Table 3.5-1 can be retrieved in the MCR.
		5. The process radiation monitors listed in Table 3.5-2 are provided.	Inspection for the existence of the monitors will be performed.	Each of the monitors listed in Table 3.5-2 exists.
		6. The effluent radiation monitors listed in Table 3.5-3 are provided.	Inspection for the existence of the monitors will be performed.	Each of the monitors listed in Table 3.5-3 exists.
		7. The airborne radiation monitors listed in Table 3.5-4 are provided.	Inspection for the existence of the monitors will be performed.	Each of the monitors listed in Table 3.5-4 exists.
		8. The area radiation monitors listed in Table 3.5-5 are provided.	Inspection for the existence of the monitors will be performed.	Each of the monitors listed in Table 3.5-5 exists.
832	3.5.00.07	Not used per Amendment No. XX		
833	3.5.00.08	Not used per Amendment No. XX		