VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

October 25, 2017

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

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VIRGINIA ELECTRIC AND POWER COMPANY (DOMINION ENERGY VIRGINIA)
NORTH ANNA POWER STATION UNIT 2
CORE OPERATING LIMITS REPORT
CYCLE 26 PATTERN KRB REVISION 1

Pursuant to North Anna Technical Specification 5.6.5.d, attached is a copy of the Dominion Core Operating Limits Report for North Anna Unit 2 Cycle 26, Pattern KRB, Revision 1, Addendum 0. The COLR was revised to include the cycle specific N(z) data, penalty factors, and operating space reductions. This information was not included in the original revision because it required shutdown data to verify.

If you have any questions or require additional information, please contact Ms. Diane Aitken at (804) 273-2694.

Sincerely,

B. L. Stanley, Director
Nuclear Regulatory Affairs
Dominion Energy Services, Inc. for
Virginia Electric and Power Company

Attachment:

COLR-N2C26, Revision 1, Addendum 0, Core Operating Limits Report, North Anna Unit 2 Cycle 26 Pattern KRB

Commitment Summary: There are no new commitments contained in this letter.

A001
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ATTACHMENT

COLR-N2C26, Revision 1, Addendum 0
CORE OPERATING LIMITS REPORT
North Anna Unit 2 Cycle 26 Pattern KRB

North Anna Power Station Units 1 and 2
Virginia Electric and Power Company

N2C26 CORE OPERATING LIMITS REPORT

INTRODUCTION

The Core Operating Limits Report (COLR) for North Anna Unit **2** Cycle **26** has been prepared in accordance with North Anna Technical Specification 5.6.5. The technical specifications affected by this report are listed below:

TS 2.1.1	Reactor Core Safety Limits
TS 3.1.1	Shutdown Margin (SDM)
TS 3.1.3	Moderator Temperature Coefficient (MTC)
TS 3.1.4	Rod Group Alignment Limits
TS 3.1.5	Shutdown Bank Insertion Limit
TS 3.1.6	Control Bank Insertion Limits
TS 3.1.9	PHYSICS TESTS Exceptions – Mode 2
TS 3.2.1	Heat Flux Hot Channel Factor
TS 3.2.2	Nuclear Enthalpy Rise Hot Channel Factor ($F^N_{\Delta H}$)
TS 3.2.3	Axial Flux Difference (AFD)
TS 3.3.1	Reactor Trip System (RTS) Instrumentation
TS 3.4.1	RCS Pressure, Temperature, and Flow DNB Limits
TS 3.5.6	Boron Injection Tank (BIT)
TS 3.9.1	Boron Concentration

In addition, a technical requirement (TR) in the NAPS Technical Requirements Manual (TRM) refers to the COLR:

TR 3.1.1	Boration Flow Paths – Operating

The analytical methods used to determine the core operating limits are those previously approved by the NRC and discussed in the documents listed in the References Section.

Cycle-specific values are presented **in bold.** Text in *italics* is provided for information only.

REFERENCES

1. VEP-FRD-42-A, Revision 2, Minor Revision 1, "Reload Nuclear Design Methodology," August 2003.

   Methodology for:
   TS 3.1.1 – Shutdown Margin
   TS 3.1.3 – Moderator Temperature Coefficient
   TS 3.1.4 – Rod Group Alignment Limits
   TS 3.1.5 – Shutdown Bank Insertion Limit
   TS 3.1.6 – Control Bank Insertion Limits
   TS 3.1.9 – Physics Tests Exceptions – Mode 2
   TS 3.2.1 – Heat Flux Hot Channel Factor
   TS 3.2.2 – Nuclear Enthalpy Rise Hot Channel Factor
   TS 3.5.6 – Boron Injection Tank (BIT) and
   TS 3.9.1 – Boron Concentration

2. Plant-specific adaptation of WCAP-16009-P-A, "Realistic Large Break LOCA Evaluation Methodology Using the Automated Statistical Treatment of Uncertainty Method (ASTRUM)," as approved by NRC Safety Evaluation Report dated February 29, 2012.

   Methodology for: TS 3.2.1 – Heat Flux Hot Channel Factor

3. WCAP-10054-P-A, "Westinghouse Small Break ECCS Evaluation Model Using the NOTRUMP Code," August 1985.

   Methodology for: TS 3.2.1 – Heat Flux Hot Channel Factor

4. WCAP-10079-P-A, "NOTRUMP, A Nodal Transient Small Break and General Network Code," August 1985.

   Methodology for: TS 3.2.1 – Heat Flux Hot Channel Factor

5. WCAP-12610-P-A, "VANTAGE+ FUEL ASSEMBLY – REFERENCE CORE REPORT," April 1995.

   Methodology for:
   TS 2.1.1 – Reactor Core Safety Limits
   TS 3.2.1 – Heat Flux Hot Channel Factor

6. VEP-NE-2-A, Revision 0, "Statistical DNBR Evaluation Methodology," June 1987.

   Methodology for:
   TS 3.2.2 – Nuclear Enthalpy Rise Hot Channel Factor and
   TS 3.4.1 – RCS Pressure, Temperature and Flow DNB Limits

7. VEP-NE-1-A, Revision 0, Minor Revision 2, "Relaxed Power Distribution Control Methodology and Associated FQ Surveillance Technical Specifications," April 2017.

   Methodology for:
   TS 3.2.1 – Heat Flux Hot Channel Factor and
   TS 3.2.3 – Axial Flux Difference

8. WCAP-8745-P-A, "Design Bases for the Thermal Overpower ΔT and Thermal Overtemperature ΔT Trip Functions," September 1986.

   Methodology for:
   TS 2.1.1 – Reactor Core Safety Limits and
   TS 3.3.1 – Reactor Trip System Instrumentation

9. WCAP-14483-A, "Generic Methodology for Expanded Core Operating Limits Report," January 1999.

   Methodology for:
   TS 2.1.1 – Reactor Core Safety Limits
   TS 3.1.1 – Shutdown Margin
   TS 3.1.4 – Rod Group Alignment Limits
   TS 3.1.9 – Physics Tests Exceptions – Mode 2
   TS 3.3.1 – Reactor Trip System Instrumentation
   TS 3.4.1 – RCS Pressure, Temperature, and Flow DNB Limits
   TS 3.5.6 – Boron Injection Tank (BIT) and
   TS 3.9.1 – Boron Concentration

10. DOM-NAF-2-P-A, Revision 0, Minor Revision 3, "Reactor Core Thermal-Hydraulics Using the VIPRE-D Computer Code," including Appendix C, "Qualification of the Westinghouse WRB-2M CHF Correlation in the Dominion VIPRE-D Computer Code," August 2010 and Appendix D, "Qualification of the ABB-NV and WLOP CHF Correlations in the Dominion VIPRE-D Computer Code," September 2014.

    Methodology for:
    TS 3.2.2 – Nuclear Enthalpy Rise Hot Channel Factor and
    TS 3.4.1 – RCS Pressure, Temperature and Flow DNB Limits

11. WCAP-12610-P-A and CENPD-404-P-A, Addendum 1-A, "Optimized ZIRLO™," July 2006.

    Methodology for:
    TS 2.1.1 – Reactor Core Safety Limits and
    TS 3.2.1 – Heat Flux Hot Channel Factor

2.0 SAFETY LIMITS (SLs)

2.1 SLs

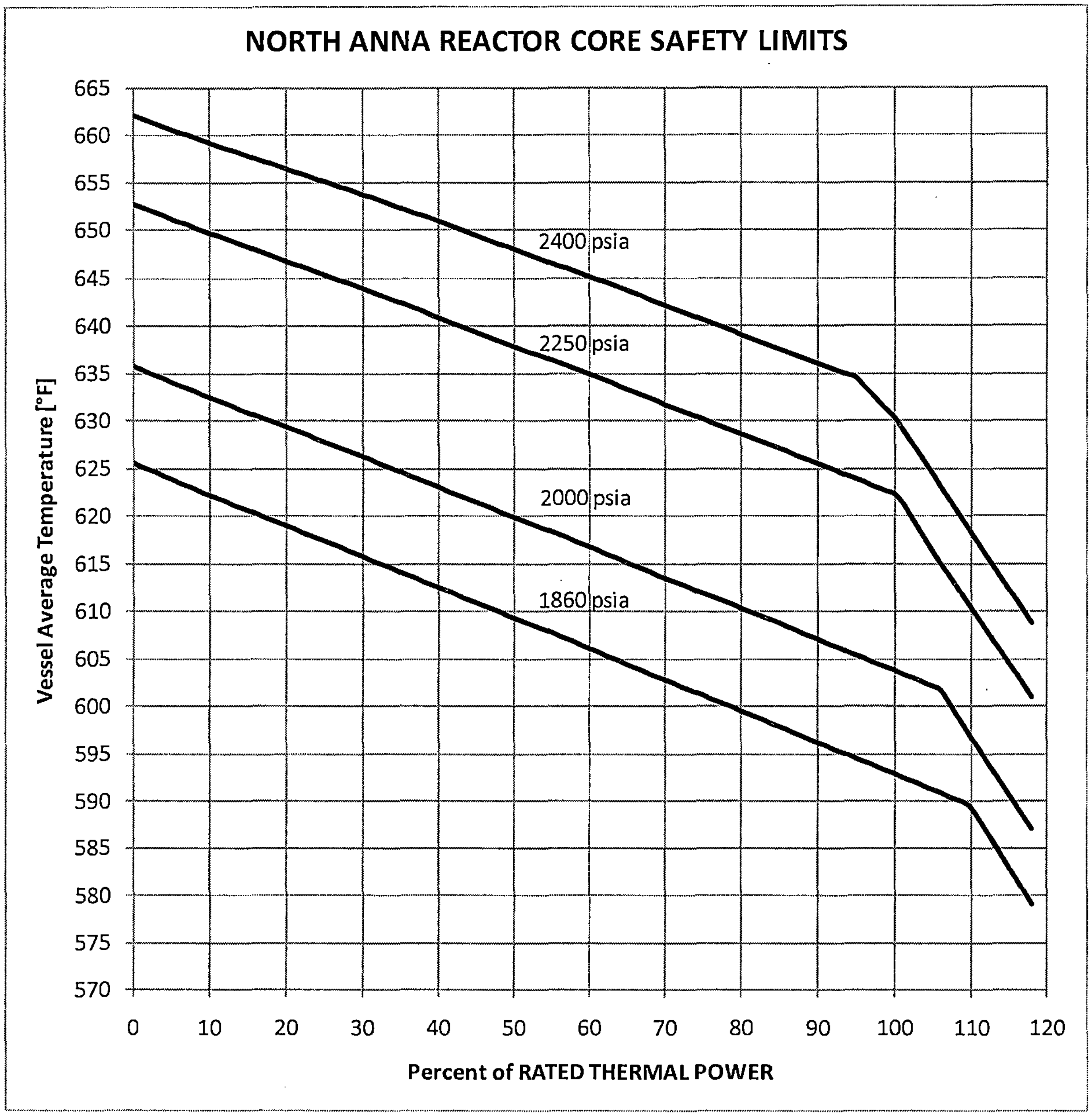
2.1.1 Reactor Core SLs

In MODES 1 and 2, the combination of THERMAL POWER, Reactor Coolant System (RCS) highest loop average temperature, and pressurizer pressure shall not exceed the limits specified in **COLR Figure 2.1-1**; and the following SLs shall not be exceeded.

2.1.1.1 The departure from nucleate boiling ratio (DNBR) shall be maintained greater than or equal to the 95/95 DNBR criterion for the DNB correlations and methodologies specified in the References Section.

2.1.1.2 The peak fuel centerline temperature shall be maintained < 5080°F, decreasing by 58°F per 10,000 MWD/MTU of burnup, for Westinghouse fuel *and < 5173°F, decreasing by 65°F per 10,000 MWD/MTU of burnup, for AREVA fuel.*

COLR Figure 2.1-1



3.1 REACTIVITY CONTROL SYSTEMS

3.1.1 SHUTDOWN MARGIN (SDM)

LCO 3.1.1 SDM shall be **≥ 1.77 % Δk/k.**

3.1.3 Moderator Temperature Coefficient (MTC)

LCO 3.1.3 The MTC shall be maintained within the limits specified below. The upper limit of MTC is **$+0.6 \times 10^{-4}$ Δk/k/°F**, when < 70% RTP, and **0.0 Δk/k/°F** when ≥ 70% RTP.

The BOC/ARO-MTC shall be **≤ $+0.6 \times 10^{-4}$ Δk/k/°F** (upper limit), when < 70% RTP, and **≤ 0.0 Δk/k/°F** when ≥ 70% RTP.

The EOC/ARO/RTP-MTC shall be less negative than **-5.0×10^{-4} Δk/k/°F** (lower limit).

The MTC surveillance limits are:

The 300 ppm/ARO/RTP-MTC should be less negative than or equal to **-4.0×10^{-4} Δk/k/°F** [Note 1].

The 60 ppm/ARO/RTP-MTC should be less negative than or equal to **-4.7×10^{-4} Δk/k/°F** [Note 2].

SR 3.1.3.2 Verify MTC is within **-5.0×10^{-4} Δk/k/°F** (lower limit).

Note 1: If the MTC is more negative than **-4.0×10^{-4} Δk/k/°F**, SR 3.1.3.2 shall be repeated once per 14 EFPD during the remainder of the fuel cycle.

Note 2: SR 3.1.3.2 need not be repeated if the MTC measured at the equivalent of equilibrium RTP-ARO boron concentration of ≤ 60 ppm is less negative than **-4.7×10^{-4} Δk/k/°F.**

3.1.4 Rod Group Alignment Limits

Required Action A.1.1 Verify SDM to be ≥ **1.77 % Δk/k.**

Required Action B.1.1 Verify SDM to be ≥ **1.77 % Δk/k.**

Required Action D.1.1 Verify SDM to be ≥ **1.77 % Δk/k.**

3.1.5 Shutdown Bank Insertion Limits

LCO 3.1.5 Each shutdown bank shall be withdrawn to at least **229** steps.

Required Action A.1.1 Verify SDM to be ≥ **1.77 % Δk/k.**

Required Action B.1 Verify SDM to be ≥ **1.77 % Δk/k.**

SR 3.1.5.1 Verify each shutdown bank is withdrawn to at least **229** steps.

3.1.6 Control Bank Insertion Limits

LCO 3.1.6 Control banks shall be limited in physical insertion as shown in **COLR Figure 3.1-1.** Sequence of withdrawal shall be A, B, C and D, in that order; and the overlap limit during withdrawal shall be **101** steps.

Required Action A.1.1 Verify SDM to be ≥ **1.77 % Δk/k.**

Required Action B.1.1 Verify SDM to be ≥ **1.77 % Δk/k.**

Required Action C.1 Verify SDM to be ≥ **1.77 % Δk/k.**

SR 3.1.6.1 Verify estimated critical control bank position is within the insertion limits specified in **COLR Figure 3.1-1**.

SR 3.1.6.2 Verify each control bank is within the insertion limits specified in **COLR Figure 3.1-1**.

SR 3.1.6.3 Verify each control bank not fully withdrawn from the core is within the sequence and overlap limits specified in **LCO 3.1.6 above.**

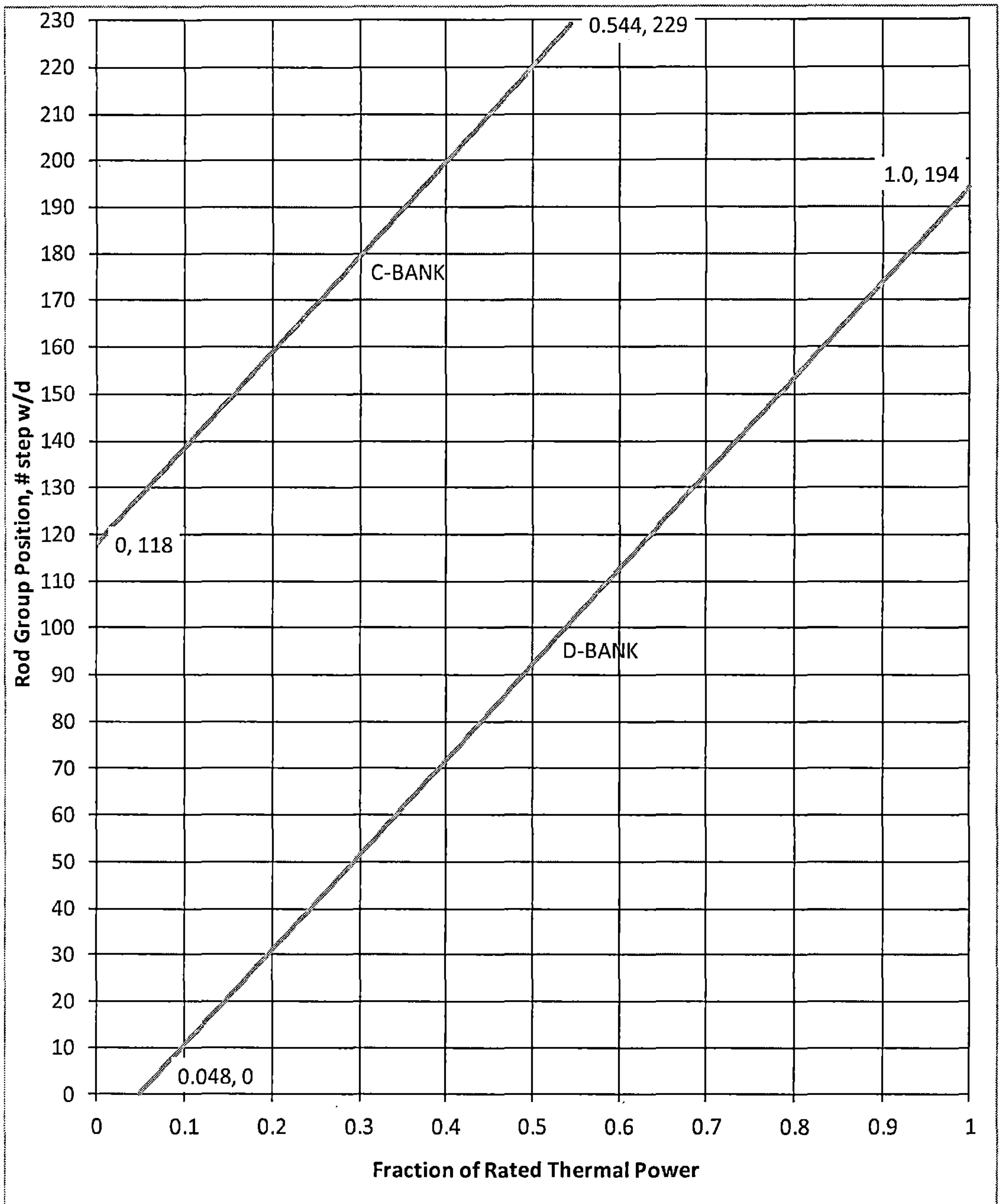
3.1.9 PHYSICS TESTS Exceptions – MODE 2

LCO 3.1.9.b SDM is ≥ **1.77 % Δk/k.**

SR 3.1.9.4 Verify SDM to be ≥ **1.77 % Δk/k.**

COLR Figure 3.1-1

North Anna 2 Cycle 26
Control Rod Bank Insertion Limits
Fully w/d position = 229 steps



3.2 POWER DISTRIBUTION LIMITS

3.2.1 Heat Flux Hot Channel Factor ($F_Q(Z)$)

LCO 3.2.1 $F_Q(Z)$, as approximated by $F_Q^E(Z)$ and $F_Q^T(Z)$, shall be within the limits specified below.

CFQ = 2.32

The Heat Flux Hot Channel Factor, $F_Q(Z)$, shall be limited by the following relationships:

$$F_Q(Z) \leq \frac{CFQ * K(Z)}{P} \quad for\ P > 0.5$$

$$F_Q(Z) \leq \frac{CFQ * K(Z)}{0.5} \quad for\ P \leq 0.5$$

where: $P = \frac{THERMAL\ POWER}{RATED\ THERMAL\ POWER}$; and

K(Z) is provided in **COLR Figure 3.2-1**

$F_Q^E(Z)$ is an excellent approximation for $F_Q(Z)$ when the reactor is at the steady-state power.

$F_Q(Z)$ from the incore flux map results is increased by 1.03 for fuel manufacturing tolerances and 1.05 for measurement uncertainty to obtain $F_Q^E(Z)$.

$$F_Q^E(Z) = F_Q(Z) * (1.03) * (1.05)$$

The expression for $F_Q^T(Z)$ is:

$$F_Q^T(Z) = F_Q^E(Z) * N(Z)$$

Where N(Z) is a cycle-specific non-equilibrium multiplier on $F_Q^E(Z)$ to account for power distribution transients during normal operation, provided in **COLR Table 3.2-1.**

The discussion in the Bases Section B 3.2.1 for this LCO requires the application of a cycle dependent non-equilibrium multiplier, N(Z), to the steady state $F_Q^E(Z)$. N(Z) accounts for power distribution transients encountered during normal operation. As function N(Z) is dependent on the predicted equilibrium $F_Q(Z)$ and is sensitive to the axial power distribution, it is typically generated from the actual EOC burnup distribution that can only be obtained after the shutdown of the previous cycle.

The cycle-specific penalty factors for flux map analysis are included in **COLR Table 3.2-2.**

Also discussed is the application of the appropriate factor to account for potential increases in $F_Q(Z)$ between surveillances. This factor is determined on a cycle specific basis and is dependent on the predicted increases in steady-state and transient $F_Q(Z)/K(Z)$ versus burnup. A minimum value of 2% is used should any increase in steady-state or transient measured or predicted peaking factor be determined unless frequent flux mapping is invoked (7 EFPD). These values are typically generated from the actual EOC burnup distribution that can only be obtained after the shutdown of the previous cycle.

The required operating space reductions are provided in **COLR Table 3.2-3.**

Should $F_Q^T(Z)$ exceed its limits the normal operating space should be reduced to gain peaking factor margins. The determination and verification of the margin improvements along with the corresponding required reductions in the Thermal Power Limit and AFD Bands are performed on a cycle-specific basis. These values are typically generated from the actual EOC burnup distribution that can only be obtained after the shutdown of the previous cycle.

COLR Table 3.2-1
N2C26 Normal Operation N(Z)

NODE	HEIGHT (FEET)	0 to 1000 MWD/MTU	1000 to 2000 MWD/MTU	2000 to 3000 MWD/MTU	3000 to 4000 MWD/MTU	4000 to 5000 MWD/MTU	5000 to 7000 MWD/MTU
5	11.2	1.087	1.086	1.121	1.159	1.180	1.180
6	11.0	1.085	1.085	1.125	1.157	1.178	1.178
7	10.8	1.082	1.084	1.128	1.155	1.173	1.173
8	10.6	1.083	1.100	1.134	1.151	1.167	1.167
9	10.4	1.088	1.126	1.142	1.147	1.161	1.161
10	10.2	1.100	1.142	1.145	1.145	1.156	1.156
11	10.0	1.114	1.143	1.144	1.145	1.150	1.150
12	9.8	1.125	1.144	1.143	1.145	1.146	1.146
13	9.6	1.131	1.144	1.143	1.144	1.144	1.144
14	9.4	1.135	1.146	1.145	1.142	1.141	1.142
15	9.2	1.135	1.147	1.147	1.142	1.142	1.144
16	9.0	1.138	1.147	1.147	1.142	1.147	1.148
17	8.8	1.140	1.147	1.147	1.145	1.155	1.156
18	8.6	1.139	1.145	1.146	1.150	1.160	1.160
19	8.4	1.140	1.145	1.147	1.156	1.163	1.168
20	8.2	1.146	1.148	1.151	1.162	1.167	1.179
21	8.0	1.152	1.152	1.155	1.165	1.169	1.186
22	7.8	1.154	1.154	1.156	1.166	1.169	1.187
23	7.6	1.156	1.156	1.156	1.165	1.167	1.186
24	7.4	1.158	1.158	1.157	1.164	1.164	1.186
25	7.2	1.159	1.159	1.157	1.161	1.161	1.185
26	7.0	1.159	1.158	1.156	1.159	1.158	1.183
27	6.8	1.159	1.159	1.157	1.159	1.158	1.184
28	6.6	1.158	1.158	1.156	1.157	1.157	1.183
29	6.4	1.153	1.153	1.150	1.149	1.149	1.176
30	6.2	1.146	1.146	1.143	1.137	1.137	1.165
31	6.0	1.144	1.144	1.140	1.134	1.134	1.163
32	5.8	1.140	1.140	1.137	1.129	1.129	1.157
33	5.6	1.128	1.128	1.125	1.114	1.113	1.137
34	5.4	1.120	1.120	1.117	1.103	1.103	1.120
35	5.2	1.119	1.119	1.116	1.102	1.105	1.117
36	5.0	1.123	1.123	1.120	1.107	1.111	1.118
37	4.8	1.127	1.127	1.123	1.113	1.114	1.117
38	4.6	1.133	1.133	1.129	1.122	1.116	1.117
39	4.4	1.139	1.138	1.136	1.129	1.117	1.116
40	4.2	1.144	1.144	1.144	1.136	1.115	1.115
41	4.0	1.150	1.151	1.151	1.142	1.116	1.116
42	3.8	1.159	1.159	1.160	1.151	1.124	1.123
43	3.6	1.169	1.167	1.167	1.159	1.137	1.135
44	3.4	1.175	1.172	1.172	1.164	1.143	1.143
45	3.2	1.178	1.175	1.175	1.167	1.147	1.150
46	3.0	1.181	1.177	1.176	1.171	1.153	1.158
47	2.8	1.185	1.181	1.181	1.178	1.162	1.166
48	2.6	1.189	1.186	1.190	1.189	1.173	1.173
49	2.4	1.196	1.196	1.203	1.203	1.185	1.184
50	2.2	1.206	1.207	1.214	1.214	1.196	1.201
51	2.0	1.219	1.219	1.224	1.225	1.205	1.212
52	1.8	1.229	1.229	1.234	1.235	1.215	1.214

COLR Table 3.2-1 (continued)
N2C26 Normal Operation N(Z)

NODE	HEIGHT (FEET)	0 to 1000 MWD/MTU	1000 to 2000 MWD/MTU	2000 to 3000 MWD/MTU	3000 to 4000 MWD/MTU	4000 to 5000 MWD/MTU	5000 to 7000 MWD/MTU
53	1.6	1.238	1.238	1.244	1.244	1.223	1.217
54	1.4	1.246	1.246	1.252	1.252	1.231	1.225
55	1.2	1.254	1.254	1.259	1.259	1.237	1.232
56	1.0	1.259	1.259	1.265	1.265	1.243	1.237
57	0.8	1.264	1.264	1.270	1.270	1.247	1.242

These decks are generated for normal operation flux maps that are typically taken at full power ARO. Additional N(Z) decks may be generated, if necessary, consistent with the methodology described in the RPDC topical (Reference 7). EOR is defined as Hot Full Power End of Reactivity.

COLR Table 3.2-1 (continued)
N2C26 Normal Operation N(Z)

NODE	HEIGHT (FEET)	7000 to 9000 MWD/MTU	9000 to 11000 MWD/MTU	11000 to 13000 MWD/MTU	13000 to 15000 MWD/MTU	15000 to 17000 MWD/MTU	17000 to EOR MWD/MTU
5	11.2	1.154	1.154	1.094	1.087	1.080	1.083
6	11.0	1.151	1.151	1.101	1.091	1.081	1.083
7	10.8	1.148	1.148	1.109	1.099	1.087	1.090
8	10.6	1.146	1.146	1.117	1.109	1.096	1.097
9	10.4	1.146	1.146	1.124	1.116	1.103	1.102
10	10.2	1.145	1.146	1.130	1.123	1.109	1.106
11	10.0	1.144	1.144	1.134	1.128	1.114	1.110
12	9.8	1.143	1.141	1.137	1.131	1.118	1.115
13	9.6	1.144	1.139	1.138	1.132	1.122	1.121
14	9.4	1.143	1.136	1.137	1.132	1.123	1.124
15	9.2	1.144	1.139	1.139	1.134	1.127	1.128
16	9.0	1.148	1.146	1.146	1.139	1.137	1.137
17	8.8	1.156	1.157	1.157	1.148	1.148	1.148
18	8.6	1.161	1.161	1.161	1.151	1.152	1.151
19	8.4	1.168	1.169	1.169	1.161	1.162	1.162
20	8.2	1.179	1.184	1.184	1.181	1.184	1.185
21	8.0	1.186	1.195	1.195	1.195	1.200	1.204
22	7.8	1.187	1.197	1.197	1.198	1.204	1.209
23	7.6	1.186	1.199	1.199	1.201	1.209	1.217
24	7.4	1.186	1.204	1.204	1.209	1.219	1.232
25	7.2	1.184	1.208	1.208	1.213	1.224	1.242
26	7.0	1.183	1.208	1.208	1.214	1.223	1.246
27	6.8	1.183	1.210	1.210	1.215	1.223	1.248
28	6.6	1.182	1.209	1.210	1.213	1.222	1.248
29	6.4	1.176	1.204	1.208	1.210	1.217	1.250
30	6.2	1.168	1.195	1.204	1.204	1.210	1.249
31	6.0	1.167	1.194	1.205	1.204	1.211	1.251
32	5.8	1.164	1.187	1.199	1.199	1.209	1.246
33	5.6	1.150	1.165	1.183	1.183	1.199	1.232
34	5.4	1.137	1.146	1.168	1.168	1.189	1.217
35	5.2	1.130	1.141	1.160	1.160	1.183	1.211
36	5.0	1.130	1.141	1.156	1.156	1.178	1.202
37	4.8	1.135	1.138	1.153	1.153	1.172	1.187
38	4.6	1.140	1.139	1.152	1.152	1.171	1.179
39	4.4	1.141	1.141	1.151	1.151	1.175	1.178
40	4.2	1.143	1.142	1.151	1.151	1.178	1.178
41	4.0	1.142	1.142	1.150	1.150	1.179	1.178
42	3.8	1.138	1.139	1.145	1.147	1.178	1.177
43	3.6	1.135	1.135	1.140	1.144	1.175	1.175
44	3.4	1.139	1.134	1.141	1.144	1.168	1.168
45	3.2	1.150	1.140	1.149	1.149	1.162	1.162
46	3.0	1.157	1.149	1.156	1.156	1.159	1.160
47	2.8	1.166	1.161	1.165	1.165	1.163	1.165
48	2.6	1.171	1.164	1.167	1.169	1.166	1.167
49	2.4	1.181	1.171	1.174	1.178	1.178	1.176
50	2.2	1.200	1.188	1.189	1.193	1.193	1.192
51	2.0	1.212	1.200	1.200	1.206	1.208	1.209
52	1.8	1.213	1.201	1.201	1.213	1.218	1.219

COLR Table 3.2-1 (continued)
N2C26 Normal Operation N(Z)

NODE	HEIGHT (FEET)	7000 to 9000 MWD/MTU	9000 to 11000 MWD/MTU	11000 to 13000 MWD/MTU	13000 to 15000 MWD/MTU	15000 to 17000 MWD/MTU	17000 to EOR MWD/MTU
53	1.6	1.215	1.204	1.204	1.217	1.222	1.224
54	1.4	1.223	1.212	1.212	1.220	1.224	1.227
55	1.2	1.230	1.220	1.220	1.225	1.228	1.231
56	1.0	1.236	1.226	1.226	1.233	1.236	1.239
57	0.8	1.241	1.232	1.232	1.240	1.244	1.246

These decks are generated for normal operation flux maps that are typically taken at full power ARO. Additional N(Z) decks may be generated, if necessary, consistent with the methodology described in the RPDC topical (Reference 7). EOR is defined as Hot Full Power End of Reactivity.

COLR Table 3.2-2
N2C26 Penalty Factors for Flux Map Analysis

Burnup (MWD/MTU)	Penalty Factor %
0-999	7.50
1000-1999	5.50
2000-2999	2.00
3000-3999	2.00
4000-4999	2.00
5000-6999	2.00
7000-8999	2.00
9000-10999	2.00
11000-12999	2.00
13000-14999	2.00
15000-16999	2.00
17000-18999	2.00
19000-EOC	2.00

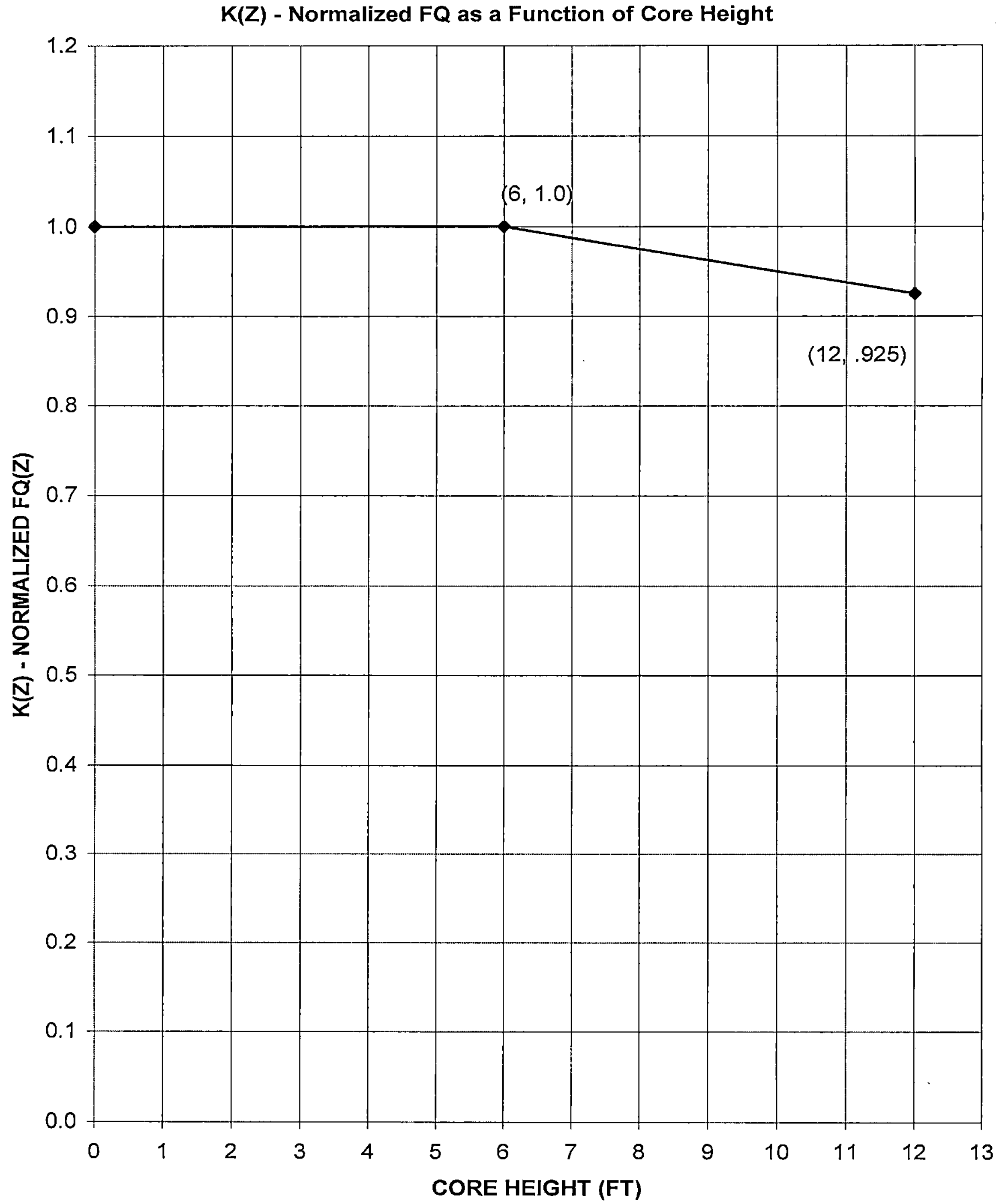
Notes:
1. *Penalty Factors are not required for initial power ascension flux maps.*
2. *All full power maps shall apply a Penalty Factor unless frequent flux mapping is invoked (≤ 7 EFPD).*

COLR Table 3.2-3
N2C26 Required Operating Space Reductions for $F_Q^T(Z)$ Exceeding its Limits

Required $F_Q^T(Z)$ Margin Improvement	Required THERMAL POWER Limit (% RTP)	Negative AFD Band Reduction from AFD Limits* (% AFD)	Positive AFD Band Reduction from AFD Limits* (% AFD)
≤ 1%	≤ 97.0	≥ 1.5	≥ 1.5
> 1% and ≤ 2%	≤ 95.0	≥ 3.0	≥ 3.0
> 2% and ≤ 3%	≤ 93.0	≥ 3.5	≥ 3.5
> 3%	≤ 50.0	N/A	N/A

* Axial Flux Difference Limits are provided in COLR Figure 3.2-2

COLR Figure 3.2-1



3.2.2 Nuclear Enthalpy Rise Hot Channel Factor ($F^N_{\Delta H}$)

LCO 3.2.2 $F^N_{\Delta H}$ shall be within the limits specified below.

$$\mathbf{F^N_{\Delta H} \leq 1.587\{1 + 0.3(1 - P)\}}$$

where: $$P = \frac{THERMAL\ POWER}{RATED\ THERMAL\ POWER}$$

SR 3.2.2.1 Verify $F^N_{\Delta H}$ is within limits specified above.

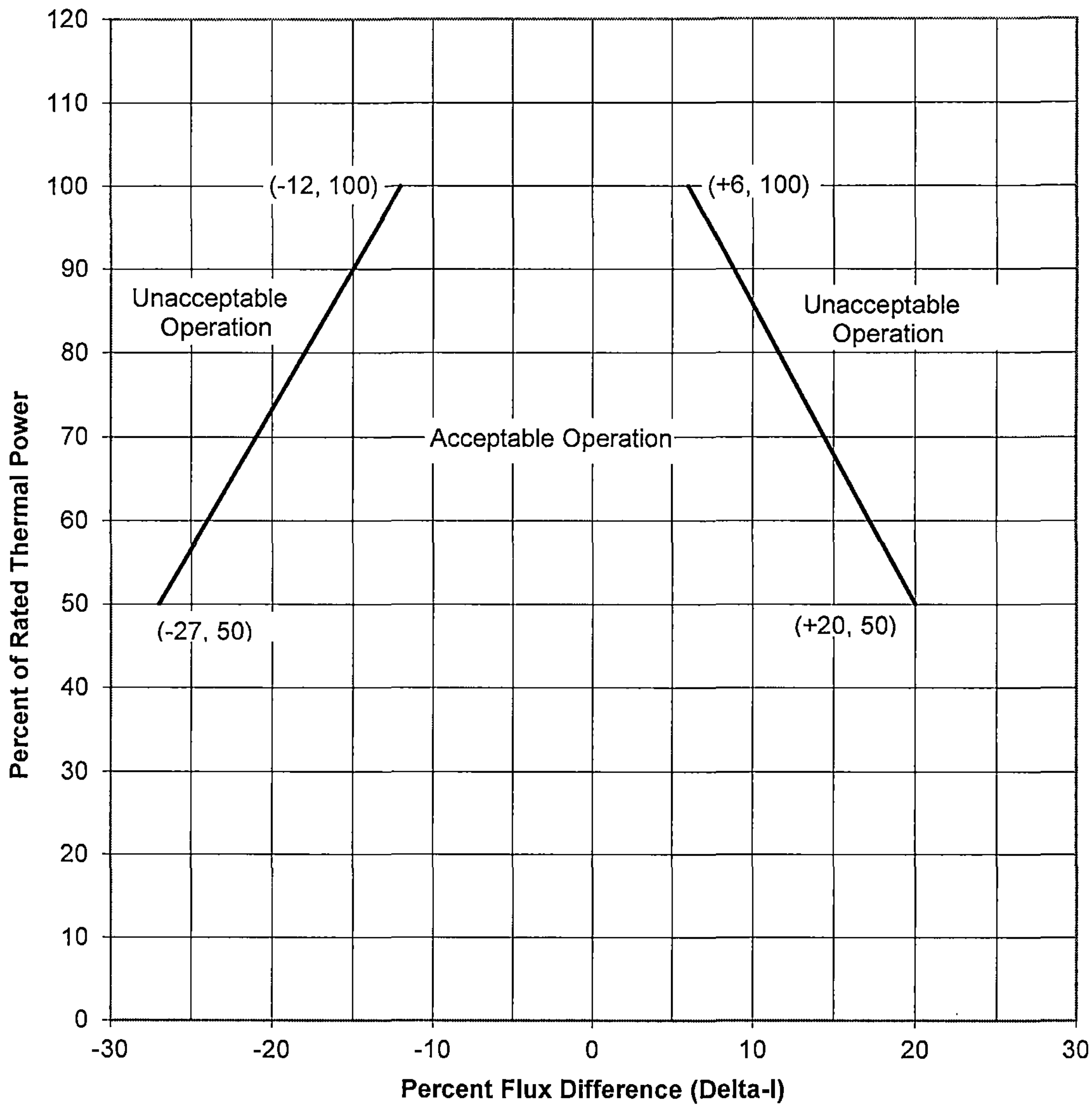
3.2.3 AXIAL FLUX DIFFERENCE (AFD)

LCO 3.2.3 The AFD in % flux difference units shall be maintained within the limits specified in **COLR Figure 3.2-2.**

COLR Figure 3.2-2

North Anna 2 Cycle 26

Axial Flux Difference Limits



3.3 INSTRUMENTATION

3.3.1 Reactor Trip System (RTS) Instrumentation

TS Table 3.3.1-1 Note 1: Overtemperature ΔT

The Overtemperature ΔT Function Allowable Value shall not exceed the following nominal trip setpoint by more than 2% of ΔT span, with the numerical values of the parameters as specified below.

$$\Delta T \leq \Delta T_0 \left\{ K_1 - K_2 \frac{(1+\tau_1 s)}{(1+\tau_2 s)} [T - T'] + K_3 (P - P') - f_1(\Delta I) \right\}$$

where:
- ΔT is measured RCS ΔT, °F
- ΔT_0 is the indicated ΔT at RTP, °F
- s is the Laplace transform operator, sec^{-1}
- T is the measured RCS average temperature, °F
- T′ is the nominal T_{avg} at RTP, **≤ 586.8 °F**
- P is the measured pressurizer pressure, psig
- P′ is the nominal RCS operating pressure, **≥ 2235 psig**

$K_1 \leq$ **1.2715** $K_2 \geq$ **0.02174 /°F** $K_3 \geq$ **0.001145 /psig**

τ_1, τ_2 = *time constants utilized in the lead-lag controller for T_{avg}*

$\tau_1 \geq$ **23.75 sec** $\tau_2 \leq$ **4.4 sec**

$(1+\tau_1 s)/(1+\tau_2 s)$ = *function generated by the lead-lag controller for T_{avg} dynamic compensation*

$f_1(\Delta I) \geq$ **0.0291{-13.0** $- (q_t - q_b)$**}** when $(q_t - q_b) <$ **−13.0%** RTP
0 when **−13.0%** RTP $\leq (q_t - q_b) \leq$ **+7.0%** RTP
0.0251{$(q_t - q_b)$ **− 7.0}** when $(q_t - q_b) >$ **+7.0%** RTP

Where q_t and q_b are percent RTP in the upper and lower halves of the core, respectively, and $q_t + q_b$ is the total THERMAL POWER in percent RTP.

TS Table 3.3.1-1 Note 2: Overpower ΔT

The Overpower ΔT Function Allowable Value shall not exceed the following nominal trip setpoint by more than 2% of ΔT span, with the numerical values of the parameters as specified below.

$$\Delta T \leq \Delta T_0 \left\{ K_4 - K_5 \left[\frac{\tau_3 s}{1+\tau_3 s} \right] T - K_6 [T - T'] - f_2(\Delta I) \right\}$$

where: ΔT is measured RCS ΔT, °F.
ΔT_0 is the indicated ΔT at RTP, °F.
s is the Laplace transform operator, sec^{-1}.
T is the measured RCS average temperature, °F.
T′ is the nominal T_{avg} at RTP, **≤ 586.8 °F.**

K_4 ≤ **1.0865**

K_5 ≥	**0.0198 /°F**	for increasing T_{avg}	K_6 ≥	**0.00162 /°F**	when T > T′
	0 /°F	for decreasing T_{avg}		**0 /°F**	when T ≤ T′

τ_3 = time constant utilized in the rate lag controller for T_{avg}
τ_3 ≥ **9.5 sec**

$\tau_3 s/(1+\tau_3 s)$ = function generated by the rate lag controller for T_{avg} dynamic compensation

$f_2(\Delta I)$ = **0, for all ΔI.**

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.1 RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits

LCO 3.4.1 RCS DNB parameters for pressurizer pressure, RCS average temperature, and RCS total flow rate shall be within the limits specified below:

a. Pressurizer pressure is greater than or equal to **2205 psig**;

b. RCS average temperature is less than or equal to **591 °F**; and

c. RCS total flow rate is greater than or equal to **295,000 gpm**.

SR 3.4.1.1 Verify pressurizer pressure is greater than or equal to **2205 psig**.

SR 3.4.1.2 Verify RCS average temperature is less than or equal to **591 °F**.

SR 3.4.1.3 Verify RCS total flow rate is greater than or equal to **295,000 gpm**.

SR 3.4.1.4 ------------------------------NOTE--
Not required to be performed until 30 days after ≥ 90% RTP.

Verify by precision heat balance that RCS total flow rate is **≥ 295,000 gpm**.

3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)

3.5.6 Boron Injection Tank (BIT)

Required Action B.2 Borate to a SDM **≥ 1.77 % Δk/k at 200 °F**.

3.9 REFUELING OPERATIONS

3.9.1 Boron Concentration

LCO 3.9.1 Boron concentrations of the Reactor Coolant System (RCS), the refueling canal, and the refueling cavity shall be maintained ≥ **2600 ppm**.

SR 3.9.1.1 Verify boron concentration is within the limit specified above.

NAPS TECHNICAL REQUIREMENTS MANUAL

TRM 3.1 REACTIVITY CONTROL SYSTEMS

TR 3.1.1 Boration Flow Paths – Operating

Required Action D.2 Borate to a SHUTDOWN MARGIN ≥ **1.77 % Δk/k at 200 °F**, after xenon decay.