

FLORIDA POWER AND LIGHT COMPANY

ORLANDO UTILITIES COMMISSION

OF THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT, UNIT 2

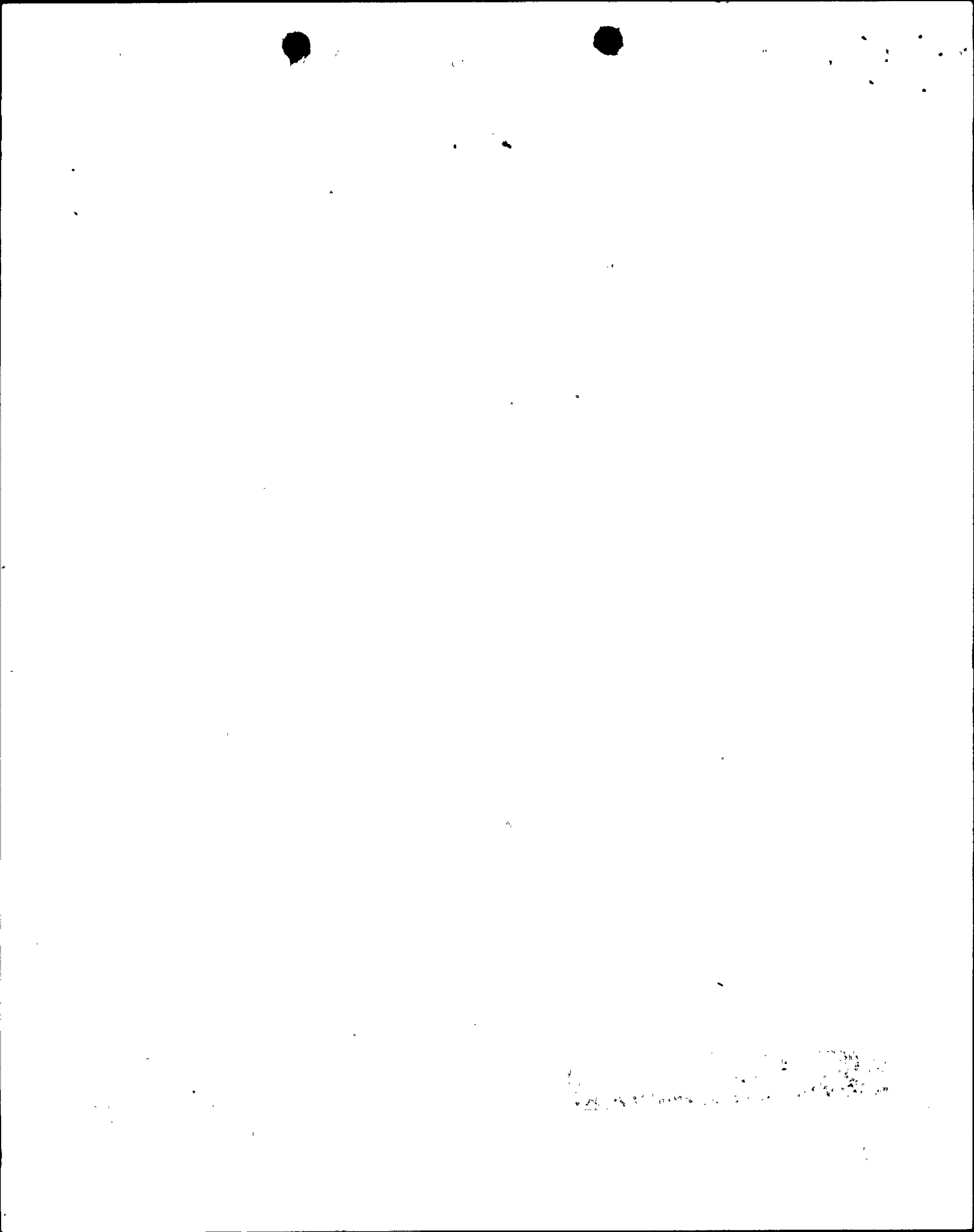
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 1
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Florida Power and Light Company, on behalf of itself and Orlando Utilities Commission of the City of Orlando, Florida and Florida Municipal Power Agency (licensees) dated April 20, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. NPF-16 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 1, are hereby incorporated in this license. The licensees shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance.

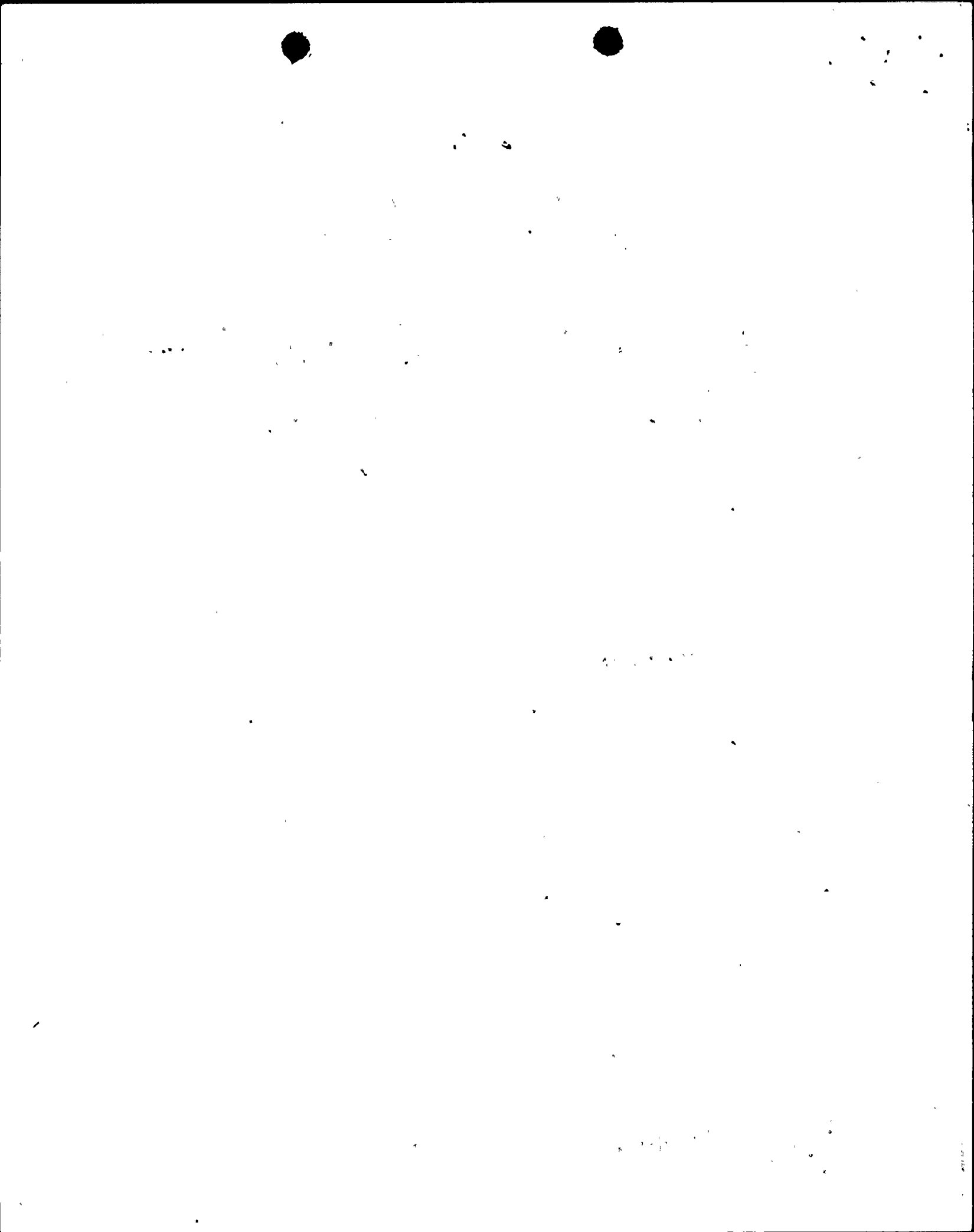
FOR THE NUCLEAR REGULATORY COMMISSION

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George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing

Date of Issuance: APR 22 1983

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SURNAME ▶	JLee:yt	JWilson	NA Wright	G.W. Knighton			
DATE ▶	4/21/83	4/21/83	4/22/83	4/21/83			



ATTACHMENT TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Also to be replaced is the following overleaf page to an amended page.

Amendment Pages

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3/4 3-9
3/4 3-10

Overleaf Page

3/4 3-7

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TABLE 3.3-2 (Continued)

REACTOR PROTECTIVE INSTRUMENTATION RESPONSE TIMES

<u>FUNCTIONAL UNIT</u>	<u>RESPONSE TIME</u>
10. Loss of Component Cooling Water to Reactor Coolant Pumps	Not Applicable
11. Reactor Protection System Logic	Not Applicable
12. Reactor Trip Breakers	Not Applicable
13. Wide Range Logarithmic Neutron Flux Monitor	Not Applicable
14. Reactor Coolant Flow - Low	0.65 second
15. Loss of Load (Turbine Hydraulic Fluid Pressure - Low)	Not Applicable

^x Neutron detectors are exempt from response time testing. Response time of the neutron flux signal portion of the channel shall be measured from detector output or input of first electronic component in channel.

^{**} Based on a resistance temperature detector (RTD) response time of less than or equal to 8.0 seconds where the RTD response time is equivalent to the time interval required for the RTD output to achieve 63.2% of its total change when subjected to a step change in RTD temperature.

TABLE 4.3-1

REACTOR PROTECTIVE INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
1. Manual Reactor Trip	N.A.	N.A.	S/U(1)	1, 2, 3*, 4*, 5*
2. Variable Power Level - High				
a. Nuclear Power	S	D(2),M(3),Q(4)	M	1, 2
b. ΔT Power	S	D(5),Q(4)		1
3. Pressurizer Pressure - High	S	R	M	1, 2
4. Thermal Margin/Low Pressure	S	R	M	1, 2
5. Containment Pressure - High	S	R	M	1, 2
6. Steam Generator Pressure - Low	S	R	M	1, 2
7. Steam Generator Pressure Difference - High	S	R	M	1, 2
8. Steam Generator Level - Low	S	R	M	1, 2
9. Local Power Density - High	S	R	M	1
10. Loss of Component Cooling Water to Reactor Coolant Pumps	N.A.	N.A.	M	N.A.
11. Reactor Protection System Logic	N.A.	N.A.	M(7)	1, 2, 3*, 4*, 5*

ST. LUCIE - UNIT 2

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AMENDMENT NO. 1

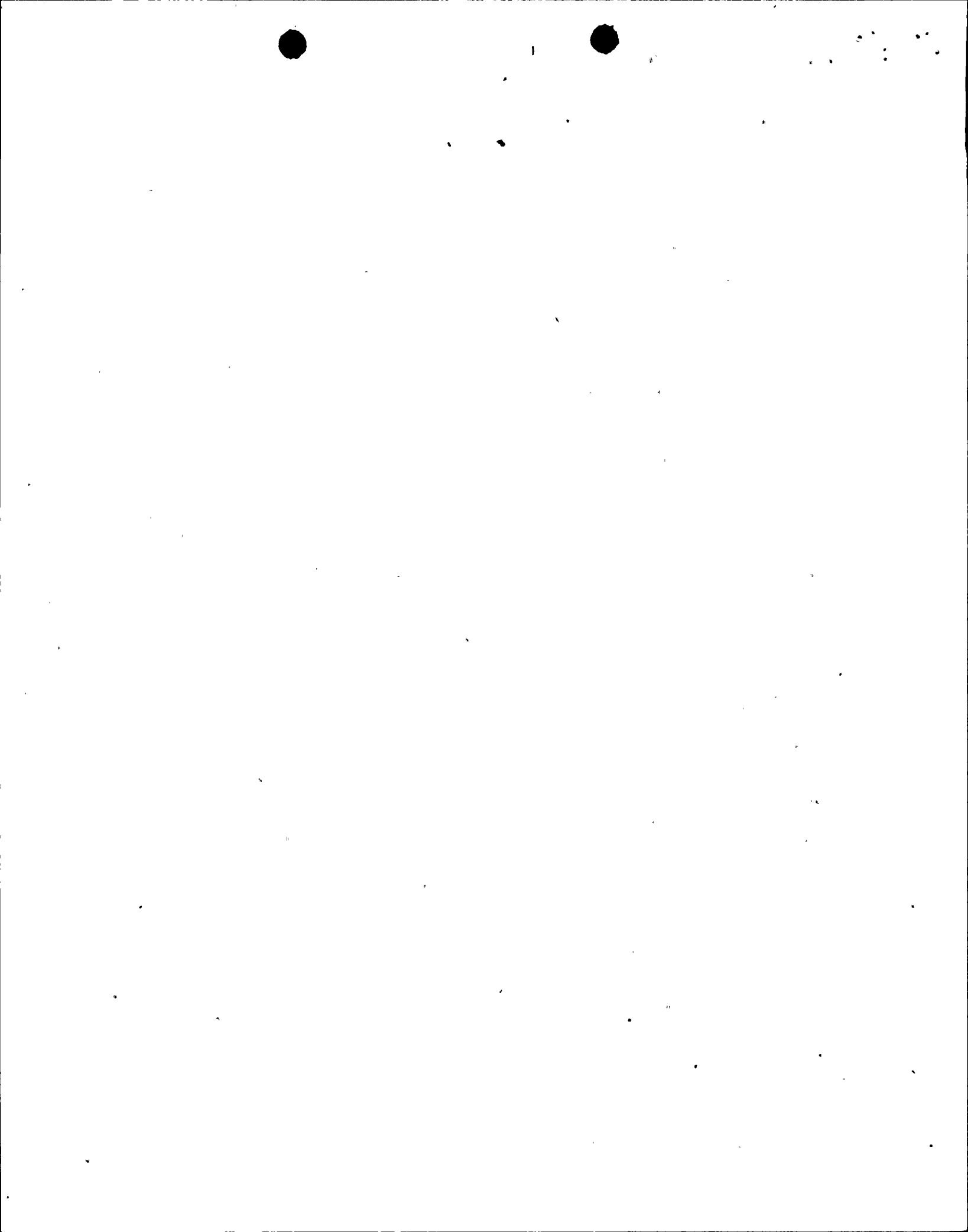


TABLE 4.3-1 (Continued)

REACTOR PROTECTIVE INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
12. Reactor Trip Breakers	N.A.	N.A.	S/U(1),M,R(6)	1, 2, 3*, 4*, 5*
13. Wide Range Logarithmic Neutron Flux Monitor	S	R	S/U(1),R	1, 2, 3, 4, 5
14. Reactor Coolant Flow-Low	S	R	M	1, 2
15. Loss of Load (Turbine Hydraulic Fluid Pressure - Low)	S	N.A.	M	1

TABLE 4.3-1 (Continued)

TABLE NOTATION

- * - Only if the reactor trip breakers are in the closed position and the CEA drive system is capable of CEA withdrawal.
- (1) - Each startup or when required with the reactor trip breakers closed and the CEA drive system capable of rod withdrawal, if not performed in the previous 7 days.
- (2) - Heat balance only (CHANNEL FUNCTIONAL TEST not included), above 15% of RATED THERMAL POWER; adjust "Nuclear Power Calibrate" potentiometer to null "Nuclear Power - ΔT Power". During PHYSICS TESTS, these daily calibrations may be suspended provided these calibrations are performed upon reaching each major test power plateau and prior to proceeding to the next major test power plateau.
- (3) - Above 15% of RATED THERMAL POWER, recalibrate the excore detectors which monitor the AXIAL SHAPE INDEX by using the incore detectors or restrict THERMAL POWER during subsequent operations to $\leq 90\%$ of the maximum allowed THERMAL POWER level with the existing reactor coolant pump combination.
- (4) - Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (5) - Adjust " ΔT Pwr Calibrate" potentiometers to make ΔT power signals agree with calorimetric calculation.
- (6) - At least once per 18 months and following maintenance or adjustment of the reactor trip breakers, the CHANNEL FUNCTIONAL TEST shall include verification of the independent OPERABILITY of the undervoltage and shunt trips.
- (7) - The fuse circuitry in the matrix fault protection circuitry shall be determined to be OPERABLE by testing with the installed test circuitry.