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NRC Form 366 (9-83)	···· ··· ··· ··· ··· ··· ··· ··· ··· ·	•			<u> </u>	U.S. NUC	LEAR REGULATORY COMMISSION				
I.	LICE	ENSEE EVEN	IT RE	PORT	(LER)	ε	XPIRES: 8/31/66				
FACILITY NAME (1)					io	OCKET NUMBER (2) PAGE (3)				
Palo Verde Unit 1					C	5 10 10 1	0 5 2 8 1 OF 0 5				
Modifications to S	Steam to Turb	oine Drive	n Aux	cilia	ry Feedwat	er Pump	Isolation				
EVENT DATE (5)	UMBER (6)	REPORT DATE	(7)		OTHER F	ACILITIES INVOL	VED (\$)				
MONTH DAY YEAR YEAR	UMBER REVISION	MONTH DAY	YEAR	D-1.	FACILITY NAM	ES	DOCKET NUMBER(S)				
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OPERATING THIS REPORT IS S MODE (9) 1 20,402(b)	UBMITTED PURSUANT TO	20 405(c)	ITS OF 10	CFR §: /	Check one or more of	the following) (11)	73 71(b)				
POWER 20.405(s)(1)(i	•	50.38(c)(1)			50.73(a)(2)(v)	ŀ	73.71(c)				
		50.36(c)(2)		-	50.73(a)(2)(vii)	, F	OTHER (Specify in Abstract below and in Text, NRC Form				
20.405(a)(1)(i		50,73(a)(2)(ii)			50,73(a)(2)(viii)(A		J00A)				
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NAME		CENSEE CONTACT P	OR THIS	LEA (12)		т	ELEPHONE NUMBER				
John F. Malik (Act	ting) Complia	nco Load				AREA CODE	303 - 35 27				
	MPLETE ONE LINE FOR E	EACH COMPONENT	FAILURE	DESCRIBE	D IN THIS REPORT	(13)	3 3 3 - 3 3 2 1				
CAUSE SYSTEM COMPONENT MANU	IFAC- REPORTABLE		CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPRDS				
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S.	UPPLEMENTAL REPORT E	EXPECTED (14)				EXPECTED	MONTH DAY YEAR				
X YES (If yes, complete EXPECTED SUBMISSI	ON DATE!					DATE (15)	0 2 2 9 8 8				
ABSTRACT (Limit to 1400 spaces, I.e., approximate	aly fifteen single-space typew	ritten lines) (18)									
On November 27, 1	987, at appro	oximately	0220	MST,	with Pal	o Verde U	nit 2 in				
Mode I (POWER OPE Feedwater Pump (B	KAIIUN) at IG (A)(P) did no	JU percent t achieve	; powe rate	er, t d sne	ne lurbine ed during	e Driven . the mont	Auxiliary				
operability surve	illance test	•	Tutt	a she	ca aai mg		ii i y				
An investigation	found that th	ha llanaull	7	4 a	+ (CD)//		dute fou the				
"B" and "A" train	valves (SB)	ne "open" (ISV) were	adiu	c swi isted	on Octobe	er 14 and	15, 1987,				
respectively, to	prevent the	valve inte	ernal	s fro	m impacti	ng on the	backseat.				
This was done in	accordance w	ith an app	rove	d eng	ineering e	evaluatio	n which did				
(SR)(33), therefo	orresponding re preventing	adjustmer a the numn	its to from	o the m ach	"ramp up ieving it:	s rated s	need.				
Following the mod	lification to	the valve	e's 1	imit	switch, t	ne pump w	as returned				
to operable statu	s, contrary	to Technic	al Sp	becif	ication 3	.7.1.2.					
As immediate corr	ective actio	n the limi	it sw	itche	s were rea	adjusted.	operability				
tests conducted o	n November 2	7, 1987, a	nd tl	he in	vestigatio	on expand	ed to				
include Units I a	nd 3. Preli	minary eva ility por	luat	ions ancod	identifie	d the roo the engi	t Cause as				
evaluation did no	t address the	e full imp	bact o	of th	e approve	d modific	ation.				
Corrective action	to prevent 1	recurrence	e is u	under	evaluatio	on, and w	ill be				
addressed in a su	ipp lement to t	this repor	τ.				16				
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U.S. NUCLEAR REGULATORY COMMISSION (9-83)
LICENSEE EVENT REPORT (LER) TEXT CONTINUATION
APPROVED OMB NO 3150-0104
EXPIRES; 8/31/88
FACILITY NAME (1)
DOCKET NUMBER (2)
LER NUMBER (6)
PAGE (3)

				A NUMBER (6)	1			AGE (3)
		YEAR		SEQUENTIAL	SE NU	ISION	-		
Palo Verde Unit 1	0 5 0 0 5 2 8	8 8 7	_	01215	0	10.	012	OF	015
TEXT (If more space is required, use additional NRC Form 305A's) (17)					I	11	<u>_</u>	<u> </u>	بنانت

On November 27, 1987 at approximately 0220 MST, with Palo Verde Unit 2 in Mode 1 (POWER OPERATION) at 100 percent power, the Turbine Driven Auxiliary Feedwater Pump (BA)(P) did not achieve its rated speed during the performance of 42ST-2AF02, "Auxiliary Feedwater Pump AFA-P01 Operability Test." The Operations personnel (utility, licensed) notified the Shift Supervisor (utility, licensed), who declared the pump inoperable under the requirements of Technical Specification (T.S.) 3.7.1.2.

Upon initiation of an Auxiliary Feedwater Actuation Signal (AFAS), the pump initially operates at approximately 400 rpm with steam supplied through a bypass valve (SB)(ISV) for either the "A" or "B" Train Steam Supply (2SGAUV0134A or 2SGAUV0138A, respectively). The pump will achieve rated speed (3560 rpm) after the corresponding steam supply isolation valve (2SGAUV0134 or 2SGAUV0138)(SB)(ISV) reaches a predetermined open position setting and generates a "ramp up" signal to the pump governor valve (BA)(FCV). Upon receipt of the "ramp up" signal, the governor valve opens to allow sufficient steam flow to bring the pump up to rated speed.

During the performance of the surveillance test, the pump did not achieve rated speed when steam was supplied from the "A" Train Steam Supply utilizing the "A" Train Steam Supply Isolation Valve (2SGAUV0134). Immediately following this test the pump was successfully operated with steam supplied by the "B" Train Steam Supply utilizing "B" Train Steam Supply Isolation Valve (2SGAUV0138), indicating a possible malfunction of isolation valve 2SGAUV0134.

Initial troubleshooting conducted in accordance with approved work authorizing documents revealed that the "ramp up" limit switch (SB)(33) for 2SGAUV0134 was set at approximately 90 percent of the valve's full open position, while the "open" limit switch (SB)(33) was set at approximately 65 percent. When operated, the valve coasted to an open position of approximately 80 percent of full travel, which was insufficient to actuate the "ramp up" limit switch and allow the pump to achieve rated speed. It was determined that the "open" limit switch settings for steam supply isolation valves 2SGAUV0138 and 2SGAUV0134 had been previously adjusted in accordance with an approved engineering evaluation (EER) intended to address the inherent coasting of the valve and prevent excessive impact when backseating. The engineering evaluation directed that the "open" limit switch setting for each valve be adjusted in 5 percent increments between 60 and 100 percent of full travel, such that the valve would open as far as possible without coasting into the backseat. The evaluation conducted to address the coasting of the valve into the backseat also recommended alternative resolutions other than the setting adjustments described above. These recommendations are documented in Plant Change Request 87-13-SG-019 and are being evaluated.

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NRC Form 366A (9-83)					U.S. NUC	LEAR REG	ULATOR	Y CON	MISSION
	LICENSEE EVENT	REPORT (LER) TEXT CONTINU	ATION		APF EXP	ROVED O IRES: 8/31	M8 NO 3 /88	150-0	104
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		5.2.9	0.7	0.2	. 5	0.0	0.3		0.5
Palo Verd		0 5 0 0 0 5 2 0	0/-	-1 012	1-1-	00	015	01	<u> </u>
The los	won limit of 60 nonco	ant of full travel was ost	ablich	od to	oncu	, Maath			
steam f	low sufficient to on	erate the pump under full	load	would	be a	re in chiev	ed.		
The en	jineering evaluation	addressed the setting for	the "	open"	limi	t swi	tch		
, located	l on rotor #1, but di	d not address the effect	this c	hange	wou]	d hav	e on		
"ramp i	ictions of other limi	ted on rotor #3. A work	ne vai order v	ve, s was w	ucn a ritte	s tne n to			
implem	ent the engineering e	valuation, and the "open"	limit	swit	ch se	ttina	for		
valve	SGAUV0134 was adjust	ed to approximately 65 pe	rcent	of fu	11 op	en. Ö	The		
"open"	limit switch for iso	lation valve 2SGAUV0138 w	as adj	usted	to		m: +		
switch	left at a setting of	approximately 90 percent	Fol	lowin	amp up a the	5 11 5 42	111 L		
adjustn	ents, isolation valv	es 2SGAUV0138 and 2SGAUV0	134 we	re sa	tisfa	ctori	1y		
tested	for valve position a	nd stroke time in accorda	nce wi	th AS	ME Se	ction	XI		Ŧ
require	ments and returned to the en	O SERVICE ON UCTODER 14 a dimeering evaluation did	na 15, not in	1987 dicat	, o tha	t tho	vor		
perform	ned could affect operation	ation of the pump, additi	onal to	estin	g was	c une	1011		
determ	ined not to be necess	ary. During subsequent t	esting	on O	ctobe	r 31,	1983	7	
the pun	p was satisfactorily	tested with steam suppli	ed fro	m the	"B" -	Frain			
monthly	functional surveill	ance test. The next sche	duled i	month	lv sui	rveil	lance	2	
test wa	is conducted on Novem	ber 27, 1987, at which ti	me the	pump	was	iec la	red	-	
inopera	ble as described abov	ve. During subsequent tr	oubles	hooti	ng on	Nove	mber		
27, 190 valve 2	SGAUVO138. but in th	as made to operate the pu	not a	11Z1N enera	g 150 to th	latio s rem	n Uiro/	4	
"ramp u	p" signal, exhibitin	g the same characteristic	s as v	alve	2SGAU	/0134	•	A	
Because	of the close proxim	ity of the original switc	h_sett	ings	on va	lve			
	1138, it is assumed the	hat the expected variance	in the	e dis urvoi	tance	the the	valve ting	3	
on Octo	ber 31, 1987, but no	t achieve rated speed on I	Novemb	er 27	198	7.	cing		
					4				
	diate corrective act	ion, in accordance with a	n autho	orize	d worl	() 1 2 4 .			
adjuste	it, the "open" fimit:	O percent of full open	tion v The "ou	aive nen"	SGUAV limit	switi	was ch		
setting	for isolation valve	2SGAUV0138 was left at i	ts ori	ginal	sett	ing,	since	ž	
this se	tting prevented the	valve from striking the b	acksea	ť whi	le all	lowing	g it		
to oper	i to approximately 95 s for both valves we	percent. The correspond	ing "r	amp u	p" l'ii tostor	nit 1 to			
verify	generation of the re	quired "ramp up" signal.	with sa	atisf	actor	/			
results	. The pump was then	tested in accordance wit	h the a	appli	cable	T.S.			
require	ments and verified to	o operate utilizing both	the "A'	" and	"B" 7	rain			
initiat	ed to establish anv	final adjustments needed	to ensi	ure a	ctuat	ion o	f the	د	
"ramp u	p" limit switch and	proper operation of the p	ump. I	Based	upon	the		-	
results	of this evaluation,	additional adjustments w	ere ma	de to	the	limit			
SWITCh	settings, nowever the	ese additional actions had	a no at	rrect	on ti	ie pu	πp		
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UCENSEE EVEN	T REPORT (LER) TEXT CC	NTINUATION	U.S. NUCLEAR REGULATORY CO APPROVED OMB NO 3150- EXPIRES: 8/31/88	MMISSION
FACILITY NAME (1)	DOCKET NUMBER (2)	LER	R NUMBER (6) PAGE	(3)
		YEAR	SEQUENTIAL AN REVISION	<u> </u>
Palo Verde Unit 1	0 5 0 0 0 1	5 2 88 7 _	01215-010 014 0F	0 5
At approximately 1503 MST Driven Auxiliary Feedwater completion of testing. Ba duration was approximately Mode 1 throughout the ever had the potential to be in 14, 1987, when steam suppl service for the initial ad date, Palo Verde Unit 2 ex 17, 1987 and operated for T.S. 3.7.1.2.	on November 29, 1987 Pump was declared of ased on the time and 61 hours, allowing 1. The investigation 1 an inoperable condi 1 isolation valve 2S 1 justment of the open approximately 43 day	, the Palo Ve perable, follo date of disco Palo Verde Un n showed, how tion since O60 GAUVO138 was limit switch .S. limit at (s in a condit	rde Unit 2 Turbine owing satisfactory overy, the event it 2 to remain in vever, that the pump 05 MST on October taken out of . Based on this 0605 MST on October ion contrary to	
Upon identification of the licensed) notified control of the potential impact on	event in Unit 2 the room personnel (uti those units.	Shift Superv lity licensed	risor (utility,) in Units 1 and 3	
At approximately 1708 MST, (REFUELING) at 0 percent p were notified by the Unit the limit switches associa pump. Based upon the noti licensed) initiated an inv existed in Unit 1. As a r setting for the Unit 1 "B" found to have been adjuste 1987. The "A" Train Steam adjusted on November 9, 19 Control Room personnel (ut Auxiliary Feedwater Pump i Verde Unit 1 was in Mode 6 discovery, therefore neces restraints. As immediate implemented to adjust the valves, thereby ensuring p surveillance testing requi equipment operable.	on November 27, 198 ower the control roo 2 Shift Supervisor o ited with the turbine fication, the Unit 1 'estigation to determ esult of this invest Train Steam Supply d as described in the Supply Isolation Va 87. Upon reaching the ility, licensed) dec noperable at 0330 MS (REFUELING) at 0 per sary corrective action corrective action, we "ramp up" limit swite roper operation of p rements shall be met	7, with Unit m personnel (f a potential driven auxil Shift Superv- ine if a simi igation, the Isolation Val- e Unit 2 event lve (ISGAUVOIS his determinat lared the Turl T on November rcent power at ons were desig ork documents ch for each of ump. The app prior to decl	<pre>1 in Mode 6 utility, licensed) misadjustment of iary feedwater isor (utility, lar condition "open" limit switch ve (ISGUAV0138) was t on September 23, 34) had been tion, Unit 1 bine Driven 29, 1987. Palo t the time of gnated as Mode 4 have been f the affected licable laring the affected</pre>	
Unit 1 entered Mode 5 on 0 operated in a condition co September 23 until October	ctober 5, 1987, there ntrary to T.S. for ap 5, 1987.	efore, Palo Ve oproximately 1	erde Unit 1 12 days, from	
Following notification of Room Shift Supervisor (uti Unit 3 valves had been adj implemented on September 4 original engineering evalu "open" limit switch on rot rotor #3 such that it woul valves and pump were operat were verified and minor ad	the event in Palo Ver lity, licensed) deter usted in accordance w , 1987. These work of ation utilized in Un or #1, but included a d operate in tandem w ble. As a prudent ma justments made.	rde Unit 2, th rmined that th with approved locuments impl its 1 and 2 fo additional gui with rotor #1. easure, howeve	he Unit 3 Control he corresponding work documents lemented the or adjusting the idance for setting . Therefore the er, switch settings	

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NRC Form 346A 19 831 LICENSEE EVEN	IT REPORT (LER) TEXT CONTINU	U.S. NUCLEAR REGULATORY COMMISS JATION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/88
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
-		YEAR SEQUENTIAL SEQUENTIAL SERVISION NUMBER
Palo Verde Unit 1	0 15 10 10 10 1 5 12 1 8	817 - 01215 - 010 015 OF 0
TEXT IN more space is required, use additional NRC Form 306A's) (17)		
The root cause of the sub error, in that the personn engineering evaluation in for the steam supply valve have on the ability of the for proper operation of the	ject event is attributed to nel (utility, licensed) resp itially used to adjust the o as did not address the impac e valves to generate the "ra ne pumps. The current proce	a cognitive personnel ponsible for the open limit switch settings act that this action would amp up" signal required edural controls governing

knowledge and training of the specific engineer assigned. The controls are considered adequate and there were no procedural deficiencies or violations. Because of the nature of this event and the importance of ensuring a comprehensive corrective action plan is implemented, an investigation has been conducted by the Shift Technical Advisor (STA) Group. The results of this evaluation and recommendations for corrective actions are currently being reviewed. The results of the review and subsequent implementation of the applicable corrective actions will be provided in a supplement to this report.

The only other system affected by this event was an actuation for input to the Emergency Response Facility Data Aquisition Display System (ERFDADS), which is located on the same rotor as the "ramp up" limit switch. The limitations imposed by this event on ERFDADS had no safety impact because the system is not required for safe shutdown of the unit. The corrective actions taken to adjust the valve operation also corrected this input.

There were no structures, systems or components other than those described above that were inoperable at the start of the event that contributed to the event. There were no unusual characteristics of the work location that directly contributed to the event. There were no manually or automatically initiated safety system responses associated with this event. Had a situation arisen that required the use of the turbine driven auxiliary feedwater pump, the motor driven essential and non-essential auxiliary feedwater pumps were available. As a result, this event had no adverse impact on the health and safety of the public.

No similar events involving the conditions and actions described above have been reported.

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Contraction of the local data



Arizona Nuclear Power Project P.O. BOX 52034 • PHOENIX, ARIZONA 85072-2034

> 192-00328-JGH/JEM/KCP December 23, 1987

NRC Document Control Desk Nuclear Regulatory Commission Washington, D.C. 20555

Dear Sirs:

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Subject: Palo Verde Nuclear Generating Station (PVNGS) Unit 1 Docket No. STN 50-528 Licensee Event Report 87-025-00 File: 87-020-404

Attached please find Licensee Event Report (LER) No. 87-025-00 prepared and submitted pursuant to 10CFR 50.73. In accordance with 10CFR 50.73(d), we are herewith forwarding a copy of the LER to the Regional Administrator of the Region V office.

If you have any questions, please contact J. E. Malik, (Acting) Compliance Lead at (602) 393-3527.

Very truly yours,

J. G. Haynes Vice President Nuclear Production

JGH/JEM/KCP/kj

Attachment

cc: 0. M. DeMichele (all w/a)
E. E. Van Brunt, Jr.
J. B. Martin
J. R. Ball
R. C. Sorenson
E. A. Licitra
A. C. Gehr
INPO Records Center



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