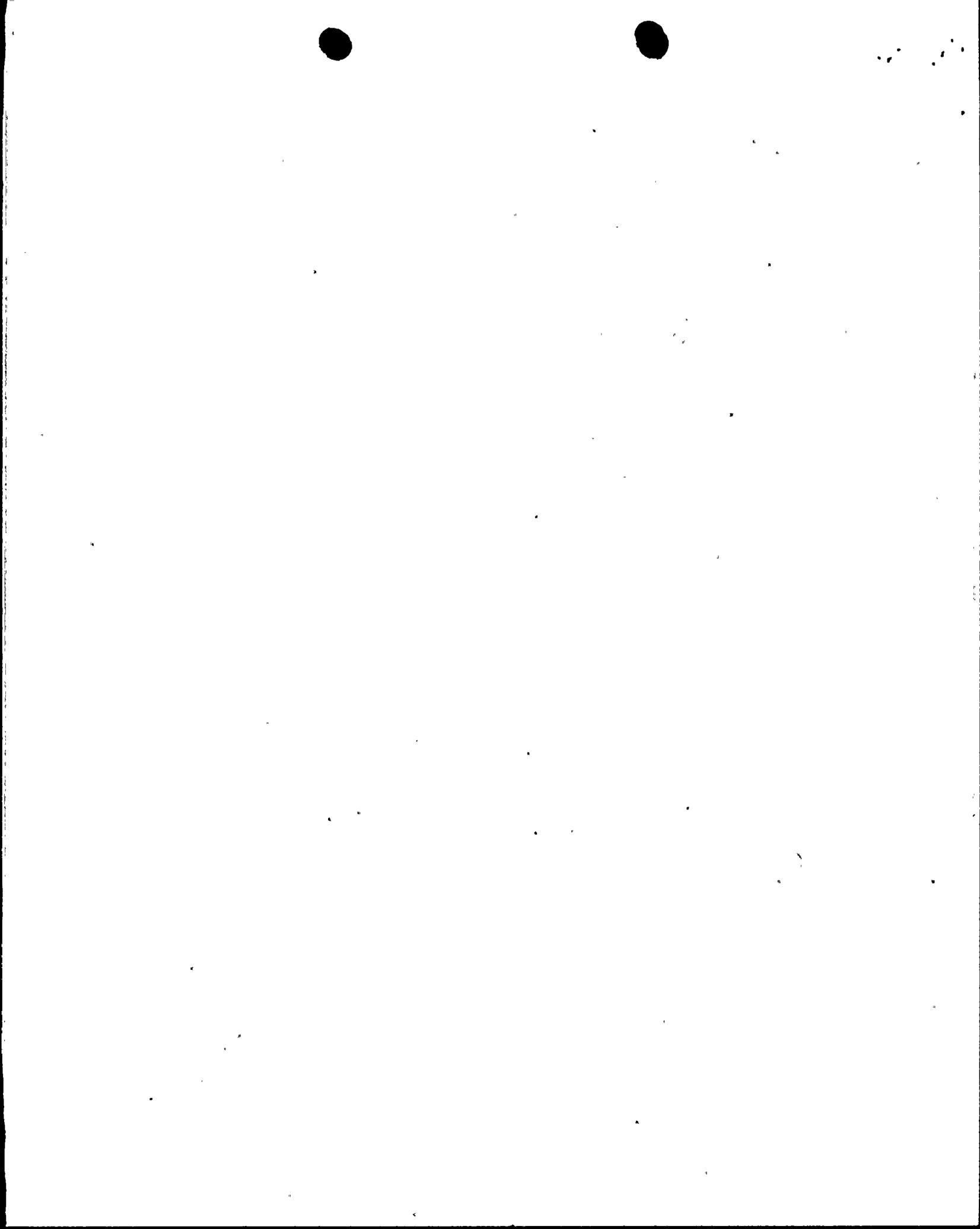


UNITED STATES NUCLEAR REGULATORY COMMISSIONARIZONA PUBLIC SERVICE COMPANY, ET AL.DOCKET NO. 50-528NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION

The U. S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NFP-41 issued to Arizona Public Service Company (the licensee), et al., for operation of the Palo Verde Nuclear Generating Station, Unit 1, located in Maricopa County, Arizona.

The amendment would change Technical Specification 3/4.11.1, "Secondary System Liquid Waste Discharges to the Onsite Evaporation Pond," to allow the concentrations of Antimony-124 (Sb-124) discharged from the secondary system liquid waste to the onsite evaporation pond to exceed 5×10^{-7} $\mu\text{Ci/ml}$ for a period not to exceed 292 days. This discharge will be within the limits of 10 CFR 20, Appendix B, Table II, Column 2, concentrations. These revisions to the technical specifications would be made in response to the licensee's application for amendment dated May 10, 1987.

On March 24, 1987, the Commission granted an emergency Technical Specification change (documented in Amendment 16 to Facility Operating License NPF-41) in response to the licensee's application for amendment dated March 23, 1987, to allow for a period of 60 days the release of secondary system liquid waste, to the onsite evaporation pond, while the concentration of principal



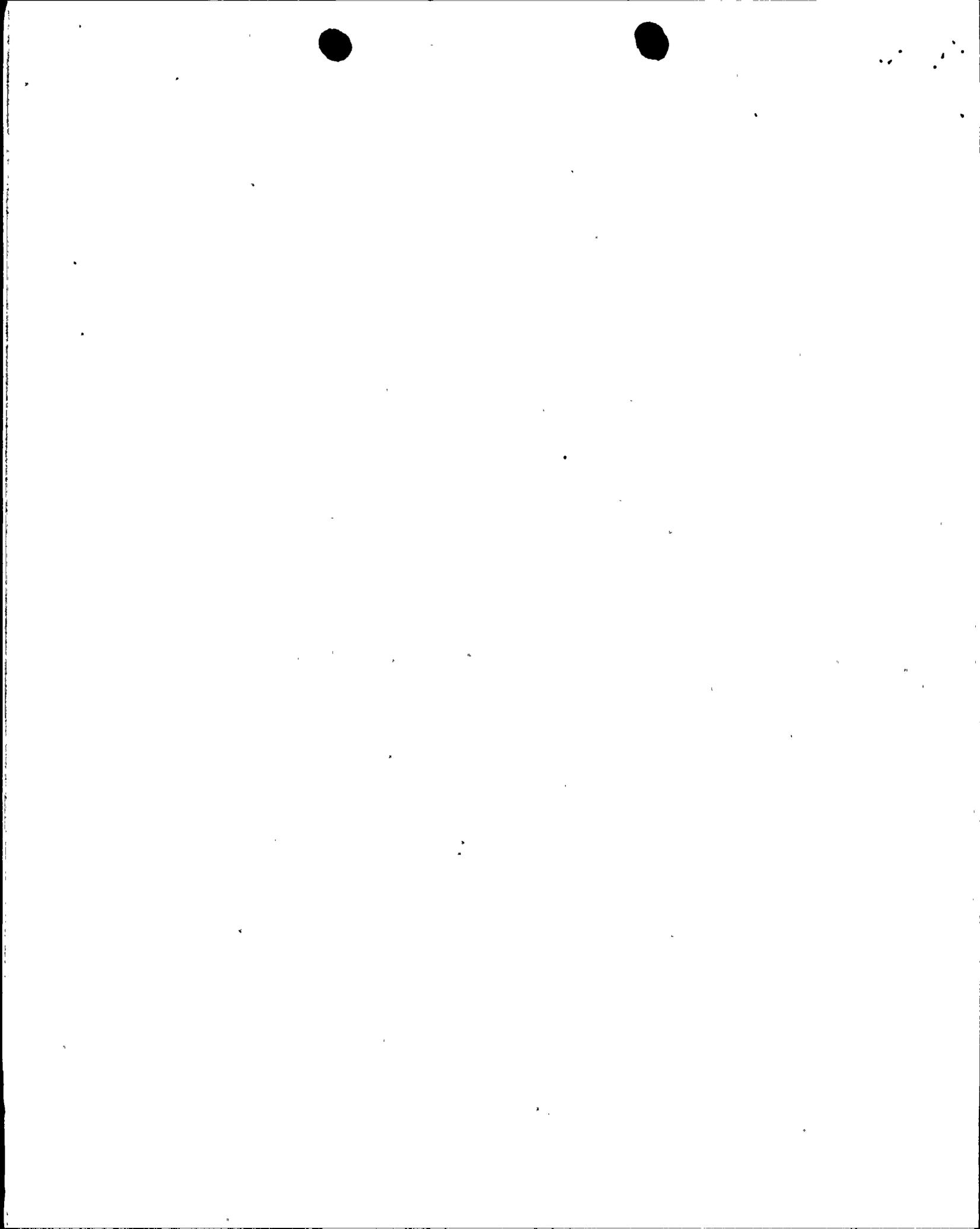
gamma emitters with half-lives less than 75 days is in excess of 5×10^{-7} $\mu\text{Ci/ml}$, provided that 10 CFR Part 20 limits are not exceeded. The required cleanup activities of the secondary system are due to the primary to secondary leakage which occurred in January 1987. In the present request, the licensee has determined that 60 days authorized in March 1987 would not afford adequate time for removal of the isotope Antimony-124 (Sb-124), with a half life of 60 days, and has requested an additional 292 days.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration.

The Commission has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR 50.92. A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with a proposed amendment would not: (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; (3) Involve a significant reduction in a margin of safety.

A discussion of these standards as they relate to the amendment request follows:



Standard 1--Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated, because the proposed change does not alter the current design of the facility. The proposed Technical Specifications change would allow continued operation of the unit while the concentration of Sb-124 discharged from secondary system liquid waste to the onsite evaporation ponds is above the lower limit of detectability but within the limits of 10 CFR Part 20, Appendix B, Table II, Column 2. This would allow for cleanup (decontamination) activities of radioactive liquids resulting from a previous primary to secondary leak, while maintaining the unit in an operational condition. Therefore, this change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Standard 2--Create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated because the proposed amendment does not vary, effect, or provide any physical changes to the facility. This proposed change allows for discharge of Sb-124 which has been generated during processing/regeneration of condensate demineralizer resins. The small amounts (5×10^{-6} to 5×10^{-7} $\mu\text{Ci/ml}$) of total Sb-124 activity present in regeneration wastes, which will be discharged into the onsite evaporation ponds, are within the limits of 10 CFR Part 20, Appendix B,

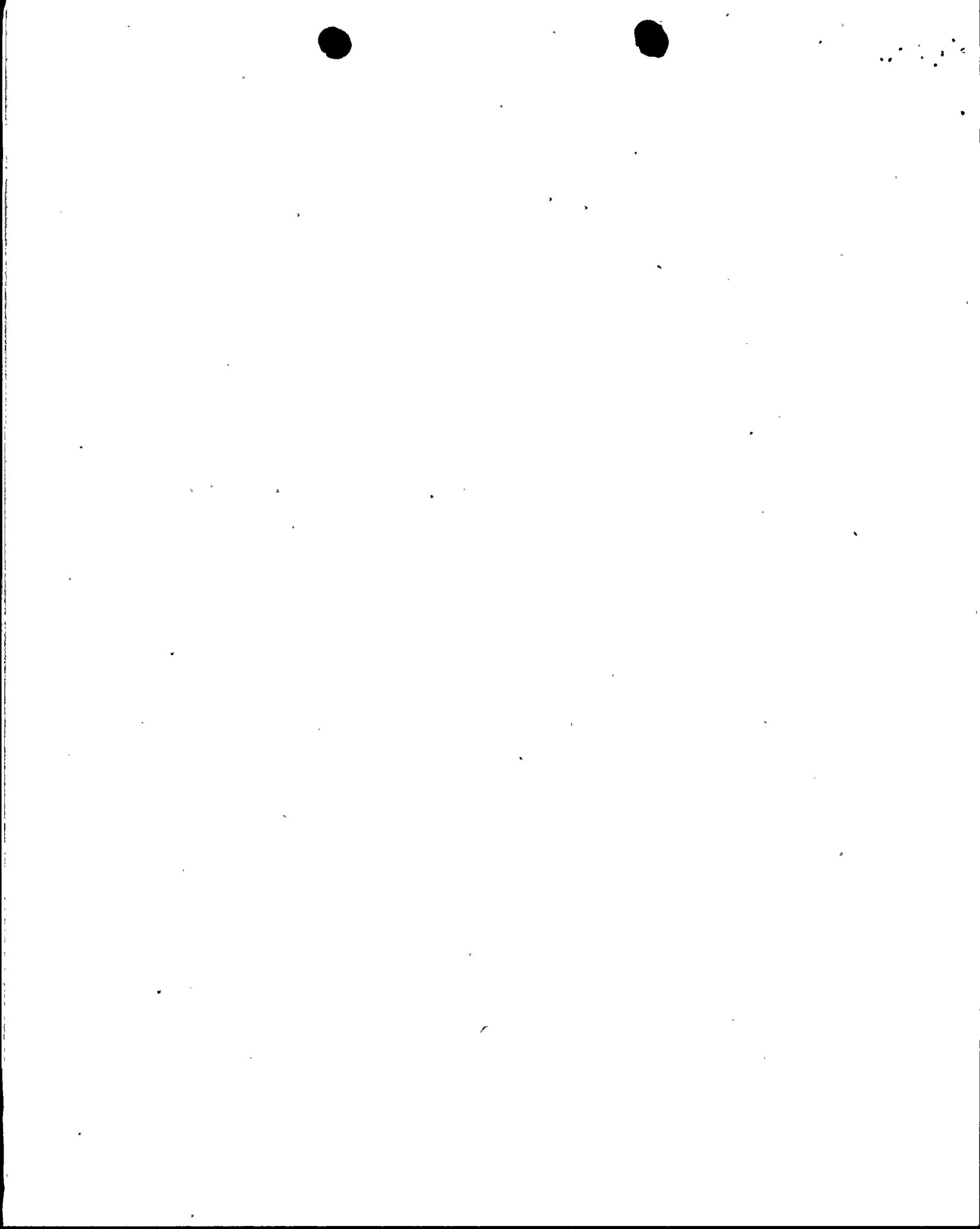
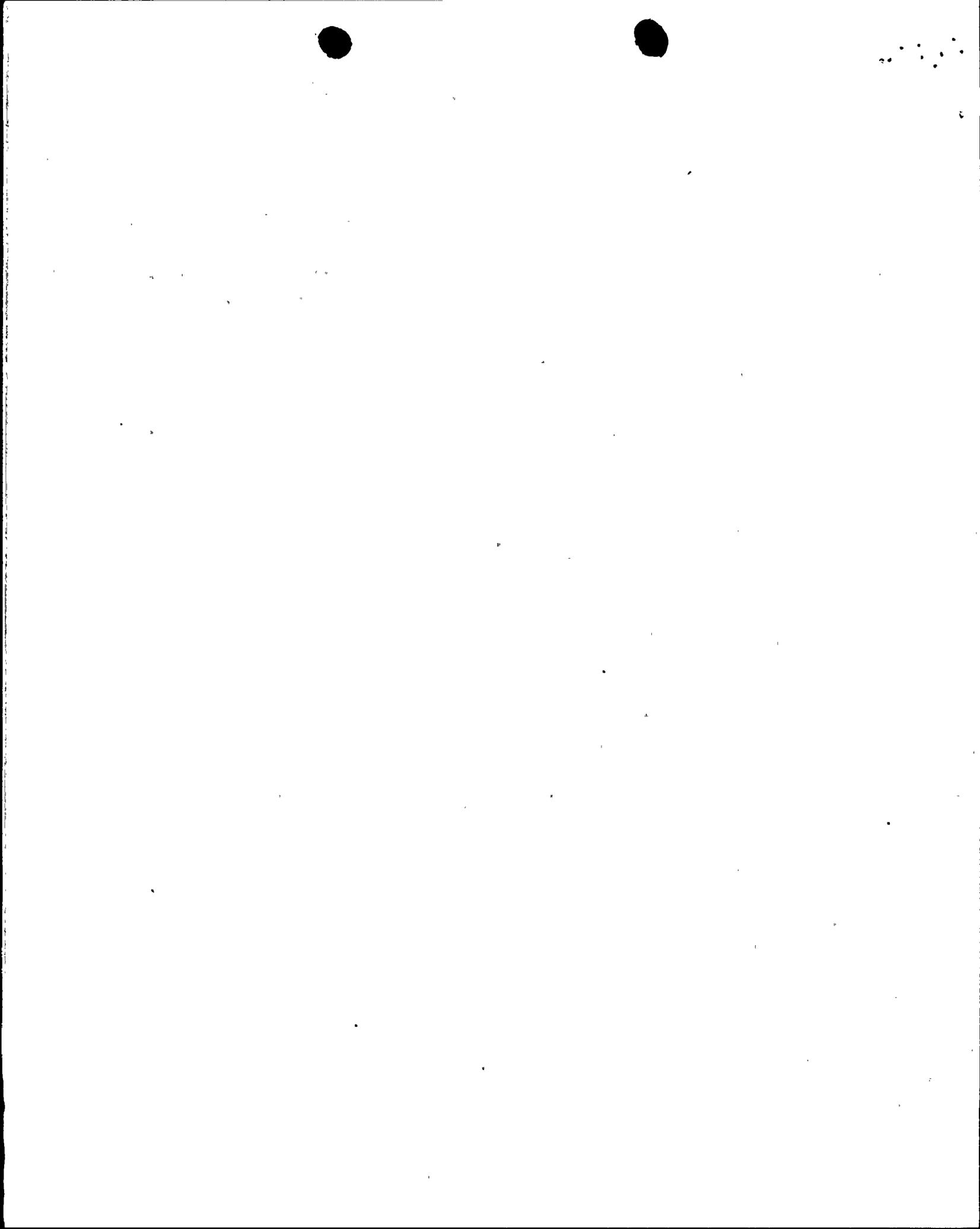


Table II, Column 2. (The current inventory of Antimony in the secondary system is estimated to be less than one Curie.) For these reasons, the proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

Standard 3--Involve a significant reduction in a margin of safety.

The requested amendment does not involve a significant reduction in a margin of safety, because the proposed change does not affect the design basis of the plant. The existing limits for concentrations of radioactive material discharged from secondary system liquid waste to the onsite evaporation ponds will remain at 5×10^{-7} $\mu\text{Ci/ml}$ for principal gamma emitters, except for Sb-124. Releases of Sb-124 may be allowed to exceed 5×10^{-7} $\mu\text{Ci/ml}$ for period not to exceed 180 days, but will be limited to 10 CFR 20, Appendix B, Table II, Column 2 concentrations. Because of the short half life (60 days) and the limited time involved (292 days), the effect of the proposed action on the previous accident analysis, i.e., the dose to the public from accumulated particulates in the evaporation ponds after the three units have been operating for 40 years, is negligible. For these reasons, it has been determined that the change does not involve a significant reduction in the margin of safety.

The staff has reviewed the licensee's no significant hazards consideration determination and agrees with the licensee's analysis. Accordingly, the Commission proposes to determine that the proposed amendment involves no significant hazards consideration.



The licensee states and the Commission has agreed that failure to act in a timely way would result in derating and subsequent loss of power generation commencing in late May 1987. Therefore, the Commission has insufficient time to issue its usual 30-day notice of the proposed action for public comment.

If the proposed determination becomes final, an opportunity for a hearing will be published in the Federal Register at a later date and any hearing request will not delay the effective date of the amendment.

If the Commission decides in its final determination that the amendment does involve a significant hazards consideration, a notice of opportunity for a prior hearing will be published in the Federal Register and, if a hearing is granted, it will be held before any amendment is issued.

The Commission is seeking public comments on this proposed determination of no significant hazards consideration. Comments on the proposed determination may be telephoned to George W. Knighton, Director, Project Directorate V, by collect call to 301-492-7331 or submitted in writing to the Rules and Procedures Branch, Division of Rules and Records, Office of Administration and Resource Management, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, and should cite the publication date and page number of this Federal Register Notice. Written comments may also be delivered to Room 4000, Maryland National Bank Building, 7735 Old Georgetown Road, Bethesda, Maryland, from 8:15 A.M. to 5:00 P.M. All comments received by June 3, 1987,



will be considered in reaching a final determination. A copy of the application may be examined at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Phoenix Public Library, Business, Science and Technology Department, 12 East McDowell Road, Phoenix, Arizona 85007.

Dated at Bethesda, Maryland, this 14th day of May, 1987.

FOR THE NUCLEAR REGULATORY COMMISSION



E. A. Licitra, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III/IV/V
& Special Projects

