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 FACIL: STN-50-528 Palo Verde Nuclear Station, Unit 1, Arizona Public 05000528  
 STN-50-529 Palo Verde Nuclear Station, Unit 2, Arizona Public 05000529  
 AUTH. NAME AUTHOR AFFILIATION  
 VAN BRUNT, E.E. Arizona Public Service Co. *MAI*  
 RECIP. NAME RECIPIENT AFFILIATION  
 SPENCER, G.S. Region 5, San Francisco, Reactor Construction & Engineer

SUBJECT: Interim deficiency report re excessive shear stresses in concrete slabs supporting safety injection tanks 1B, 2A & 2B. Seismic loads on safety injection tank supports will be reevaluated by C-E. Final report to be submitted by 801231.

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 TITLE: Construction Deficiency Report (10CFR50.55E)

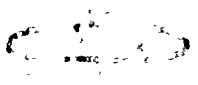
NOTES: Standardized Plant. 05000528  
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September 26, 1980  
ANPP-16430-BSK/JAR

U. S. Nuclear Regulatory Commission  
Region V  
Walnut Creek Plaza - Suite 202  
1990 North California Boulevard  
Walnut Creek, California 94596

Attention: Mr. G. S. Spencer, Chief  
Reactor Construction and  
Engineering Support Branch

Subject: A 50.55(e) Potentially Reportable Deficiency Relating  
to Excessive Shear Stresses in Units 1 and 2 Concrete  
Slabs Supporting the Safety Injection Tanks 1B, 2A  
and 2B  
Interim Report  
File: 80-019-026

Reference: Telephone Conversation between R. Dodds and B. S. Kaplan  
on August 28, 1980 (DER 80-24)

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the  
referenced telephone conversation. At that time, it was estimated  
that a determination of reportability would be made within thirty  
(30) days.

Due to the extensive investigation and evaluation required, an interim  
report is attached. It is now expected that this information will be  
finalized by December 31, 1980, at which time a complete report will  
be submitted.

Very truly yours,

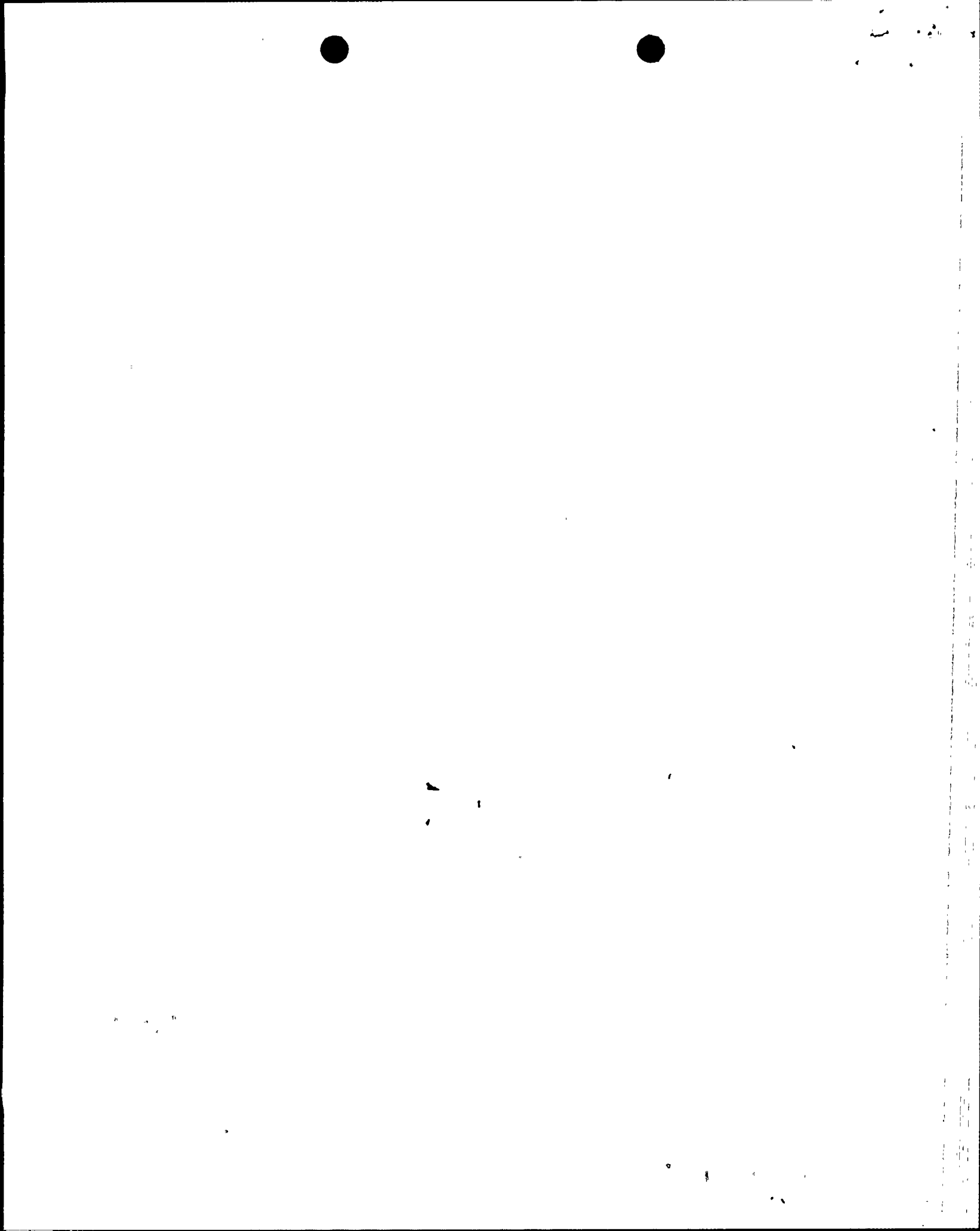
E. E. Van Brunt, Jr.  
APS Vice President  
Nuclear Projects  
ANPP Project Director

EEVBJr/BSK:skc

Attachment

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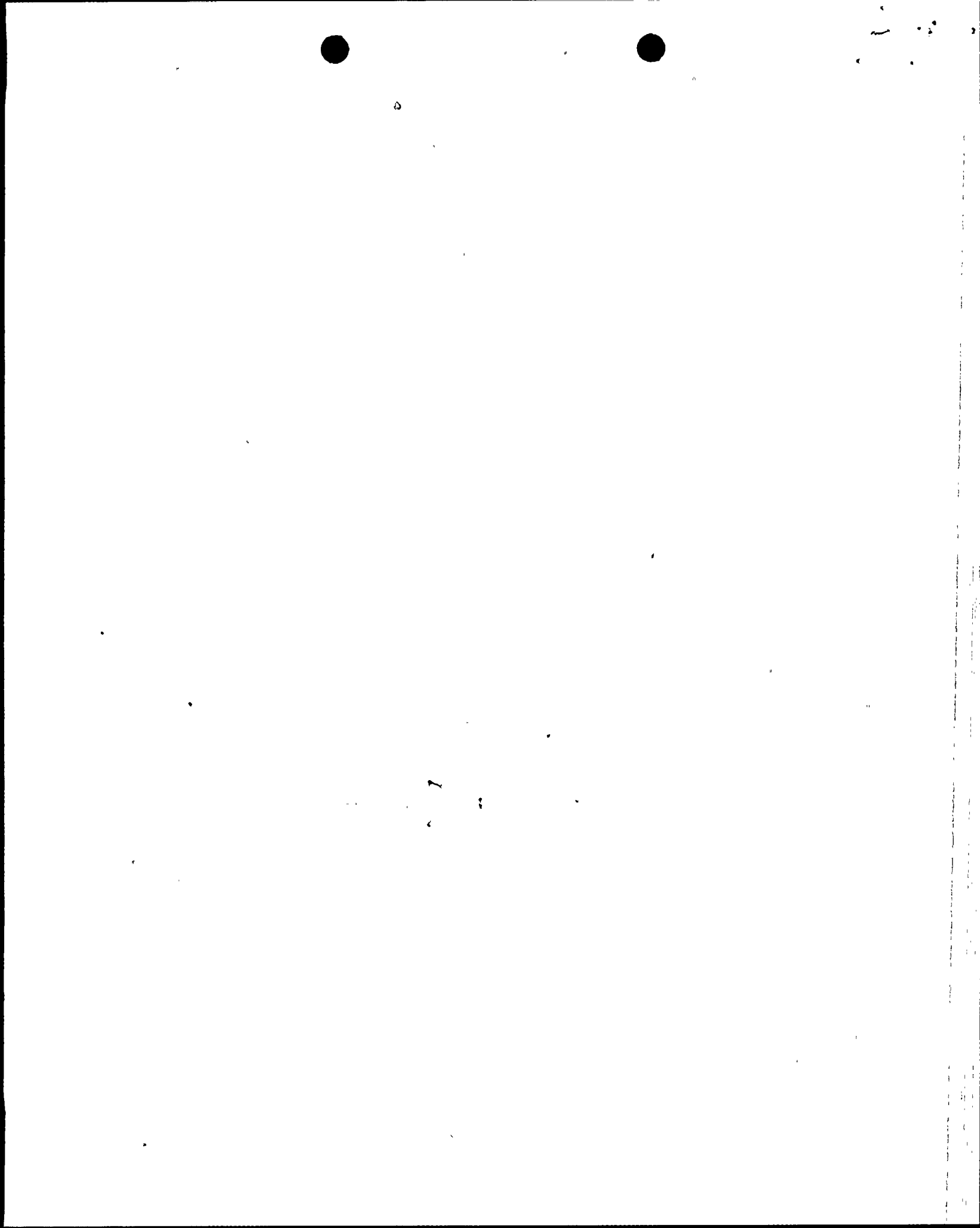


U. S. Nuclear Regulatory Commission  
Attention: Mr. G. S. Spencer, Chief  
ANPP-16430-BSK/JAR  
September 26, 1980  
Page 2

cc: Victor Stello, Jr., Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

A. C. Gehr  
Snell & Wilmer

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R. W. Welcher  
D. R. Hawkinson



INTERIM REPORT  
POTENTIAL REPORTABLE DEFICIENCY  
ARIZONA PUBLIC SERVICE COMPANY (APS)  
PVNGS UNITS #1 AND #2

I. Potential Problem

During final checking of the Containment Building calculations for the Elevation 100'-0" concrete slabs, it was discovered that the shear stresses in the concrete slabs supporting Safety Injection Tanks 1B, 2A and 2B exceed the Design Criteria allowable values due to high seismic forces. The concrete slabs were originally designed based on Safety Injection Tank support loads. This information was changed by Combustion Engineering after the slab design was issued for construction.

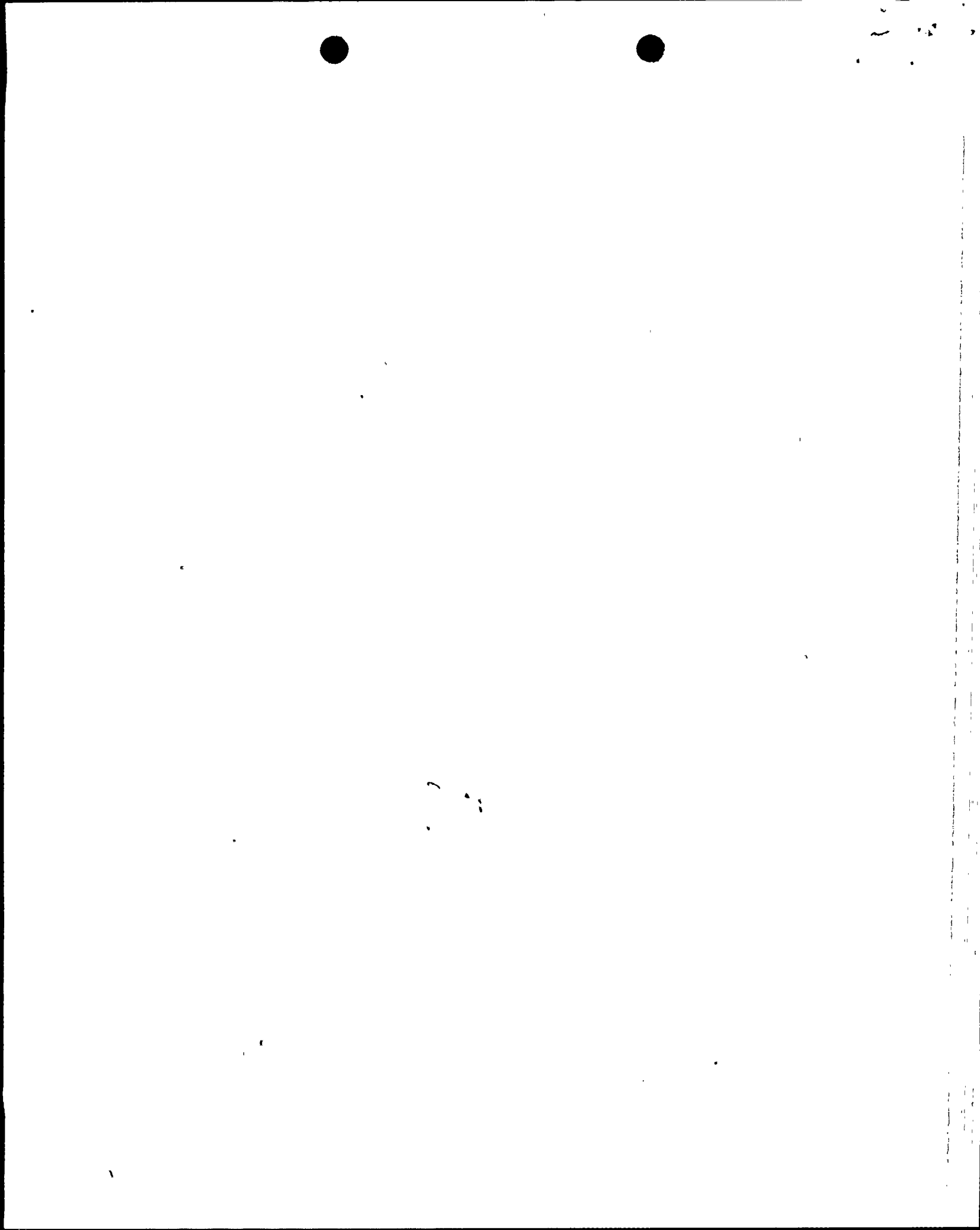
II. Approach To and Status Of Proposed Resolution

The present Safety Injection Tank seismic forces are the result of an analysis based on spring stiffnesses of a preliminary layout of the structural steel framing at Elevation 140'-0". The structural steel framing has since changed. The present steel framing plan at Elevation 140'-0" provides much greater stiffnesses at the upper lateral supports for the Safety Injection Tanks, thereby significantly reducing the seismic loads at the lower supports.

In addition to the above, the certified loads were based on a conservative seismic analysis using generic response spectrum curves. The use of Project response spectra curves will lower the resultant seismic loads.

It is our belief that using the actual spring stiffnesses of the current structural steel framing at Elevation 140'-0" and Project specific response spectra, the concrete slabs at Elevation 100'-0" will meet the Design Criteria allowable stress values.

The seismic loads on the Safety Injection Tank Supports will be reevaluated by Combustion Engineering using the actual spring stiffnesses of the current structural steel framing at Elevation 140'-0" and Project response spectra curves. Then the concrete slabs at Elevation 100'-0" will be reevaluated by Bechtel with the revised loads to see if they meet the Design Criteria allowable stress values.





III. Projected Completion of Corrective Action and Submittal  
Date of the Final Report

The Combustion Engineering/Bechtel analysis and final report is anticipated to be completed by December 31, 1980, at which time a final report will be submitted.

