



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 129 TO FACILITY OPERATING LICENSE NO. NPF-12

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-395

1.0 INTRODUCTION

By letter dated June 30, 1995, as supplemented on August 11, 1995, South Carolina Electric & Gas Company (the licensee), submitted a request for changes to the Virgil C. Summer Nuclear Station, Unit No. 1, (VCSNS) Technical Specifications (TS). The proposed amendment would revise the allowed outage times for the power operated relief valves (PORVs) and block valves. This amendment request has been submitted in response to Generic Issue 70 (GI-70) as discussed in Generic Letter 90-06 (GL 90-06), "Resolution of Generic Issue 70, 'Power-Operated Relief Valve and Block Valve Reliability,' and Generic Issue 94, 'Additional Low-Temperature Overpressure Protection for Light-Water Reactors,' Pursuant to 10 CFR 50.54(f)." The licensee's August 11, 1995, supplemental letter corrected an error in the original submittal and did not change the staff's initial proposed determination of no significant hazards consideration or expand the scope of the original Federal Register Notice.

In a related matter, the licensee requested a TS amendment to address Generic Issue 94 in a March 11, 1994 letter. The staff granted the licensee's request in License Amendment No. 125, which was issued on August 11, 1995.

2.0 EVALUATION

GI 70 involves the evaluation of the reliability of power-operated relief valves (PORVs) and block valves and their safety significance in PWR plants. The generic letter discussed how PORVs are increasingly being relied on to perform safety-related functions and the corresponding need to improve the reliability of both PORVs and their associated block valves. Proposed staff positions and improvements to the plant's technical specifications were recommended to be implemented at all affected facilities.

The actions proposed by the NRC staff to improve the reliability of PORVs and block valves represent a substantial increase in overall protection of the public health and safety and a determination was made that the costs are

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justified in view of this increased protection. The technical findings and the regulatory analysis related to GI-70 are discussed in "Technical Findings and Regulatory Analysis Related to Generic Issue 60" (NUREG-1316).

The licensee proposed a revision to TS 3/4.4.4 and its associated Bases by modifying the limiting condition for operation of PORVs and block valves. The licensee based its proposal on the model TS given in GL 90-06 and "Standard Technical Specifications, Westinghouse Plants," (NUREG-1431, Rev.1), as modified for plant specific reasons (as discussed in the following paragraph).

The model TS proposed by the staff in GL 90-06 would require licensees to maintain all PORVs in an operable status. However, the number of PORVs that need to be operable is actually determined by the capacity of each PORV. In its submittal, the licensee stated that one PORV is sufficient to mitigate a design basis steam generator tube rupture accident. Therefore, only two PORVs are required to be operable by TS (thereby retaining adequate protection assuming a single failure). In this case the extra (i.e., third PORV) can be inoperable without actions. Thus, the licensee has justified its deviation from the GL 90-06 model TS.

In its original submittal, TS 3.4.4.d would have allowed the licensee to operate with three inoperable (and not capable of being manually cycled) PORVs for approximately 73 hours. In its August 11, 1995 supplemental submittal, the licensee stated the original submittal was in error and they intended that this TS allow a one hour outage time with all three PORVs out of service. Their supplemental submittal included a revised TS page which corrected the original TS 3.4.4.d.1.a by replacing the word "or" with "and."

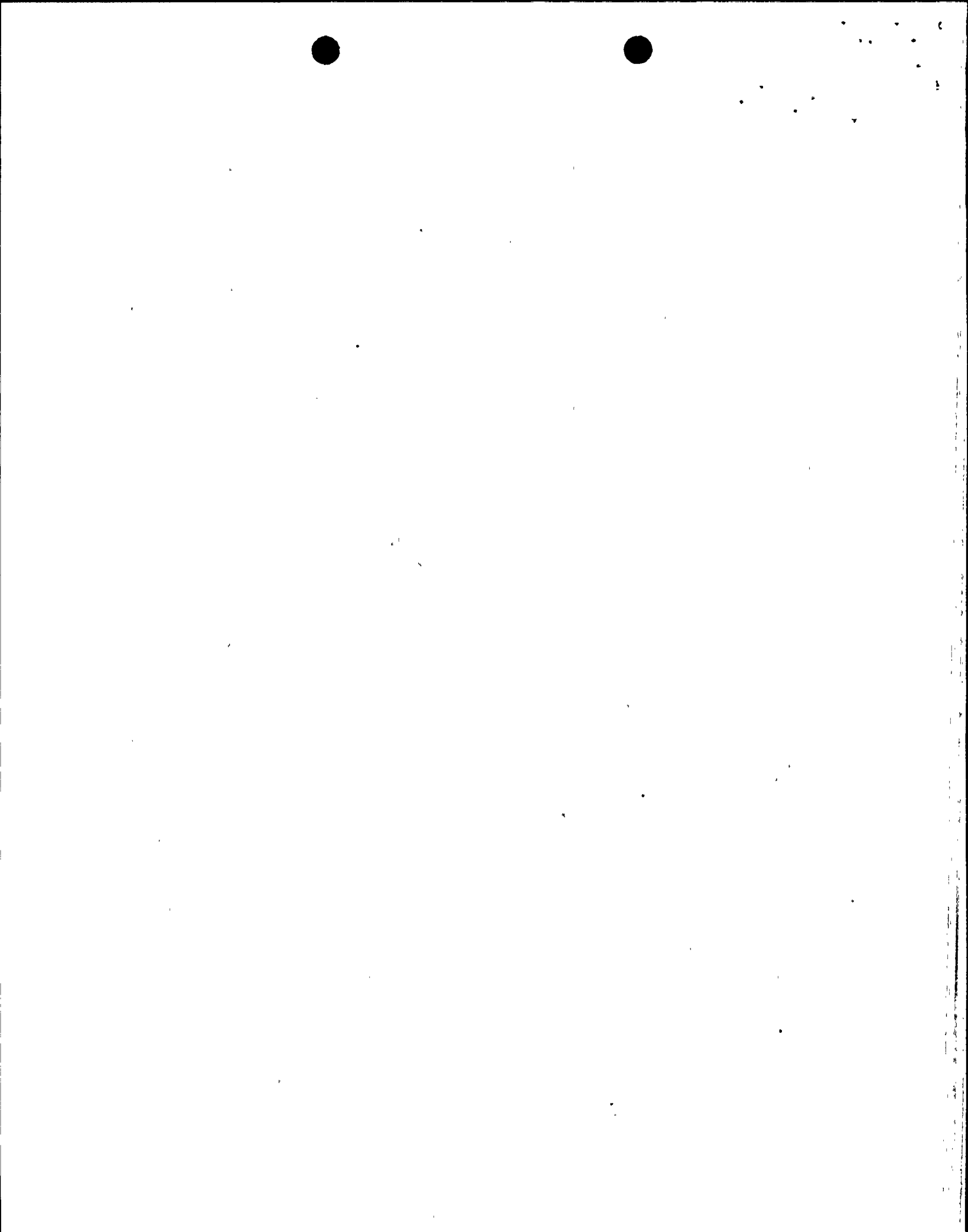
The proposed changes to the VCSNS TS are consistent with the staff's guidance in the generic letter. The proposed changes involve plant operations with the block valves closed due to leaking PORVs (or otherwise inoperable but capable of being manually cycled) and appropriate plant mode for PORV stroke testing. The licensee has adopted the staff position by stipulating power be maintained to a closed block valve so it remains operable and may be subsequently opened to allow use for control of reactor coolant system pressure. Since the proposed modifications are consistent with the staff's position previously stated in the generic letter and justified in the NUREG-1316 regulatory analysis, mentioned above, the staff finds the proposed modifications acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of South Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no



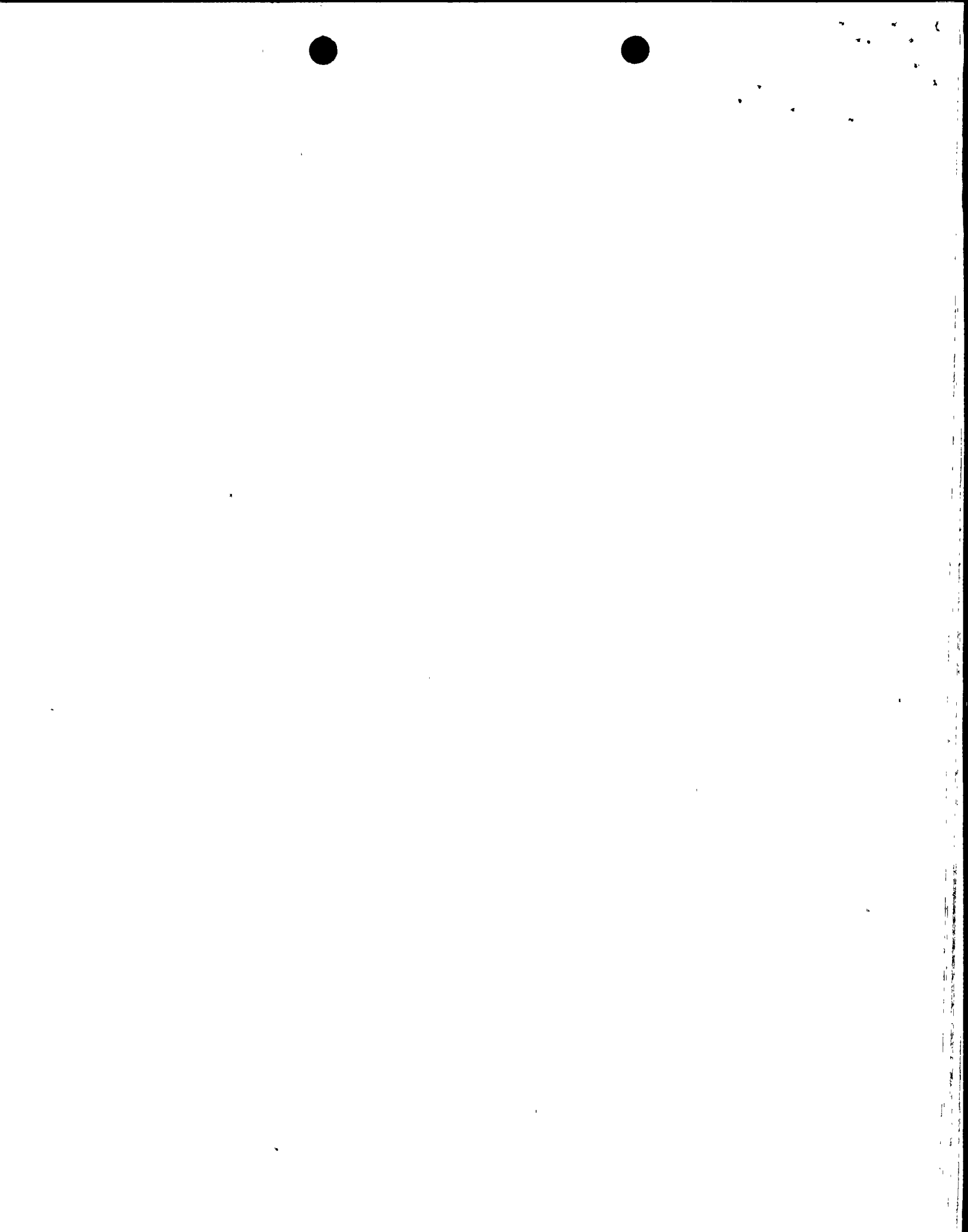
significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 42608). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Dembek

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Mr. Gary J. Taylor
South Carolina Electric & Gas Company

VIRGIL C. SUMMER NUCLEAR STATION

cc:

Mr. R. J. White
Nuclear Coordinator
S.C. Public Service Authority
c/o Virgil C. Summer Nuclear Station
Post Office Box 88, Mail Code 802
Jenkinsville, South Carolina 29065

J. B. Knotts, Jr., Esquire
Winston & Strawn Law Firm
1400 L Street, N.W.
Washington, D.C. 20005-3502

Resident Inspector/Summer NPS
c/o U.S. Nuclear Regulatory Commission
Route 1, Box 64
Jenkinsville, South Carolina 29065

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta St., NW., Ste. 2900
Atlanta, Georgia 30323

Chairman, Fairfield County Council
Drawer 60
Winnsboro, South Carolina 29180

Mr. Virgil R. Autry
Director of Radioactive Waste Management
Bureau of Solid & Hazardous Waste Management
Department of Health & Environmental Control
2600 Bull Street
Columbia, South Carolina 29201

Mr. R. M. Fowlkes, Manager
Nuclear Licensing & Operating Experience
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
Post Office Box 88
Jenkinsville, South Carolina 29065