



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

NUCLEAR PROJECT NO. 2

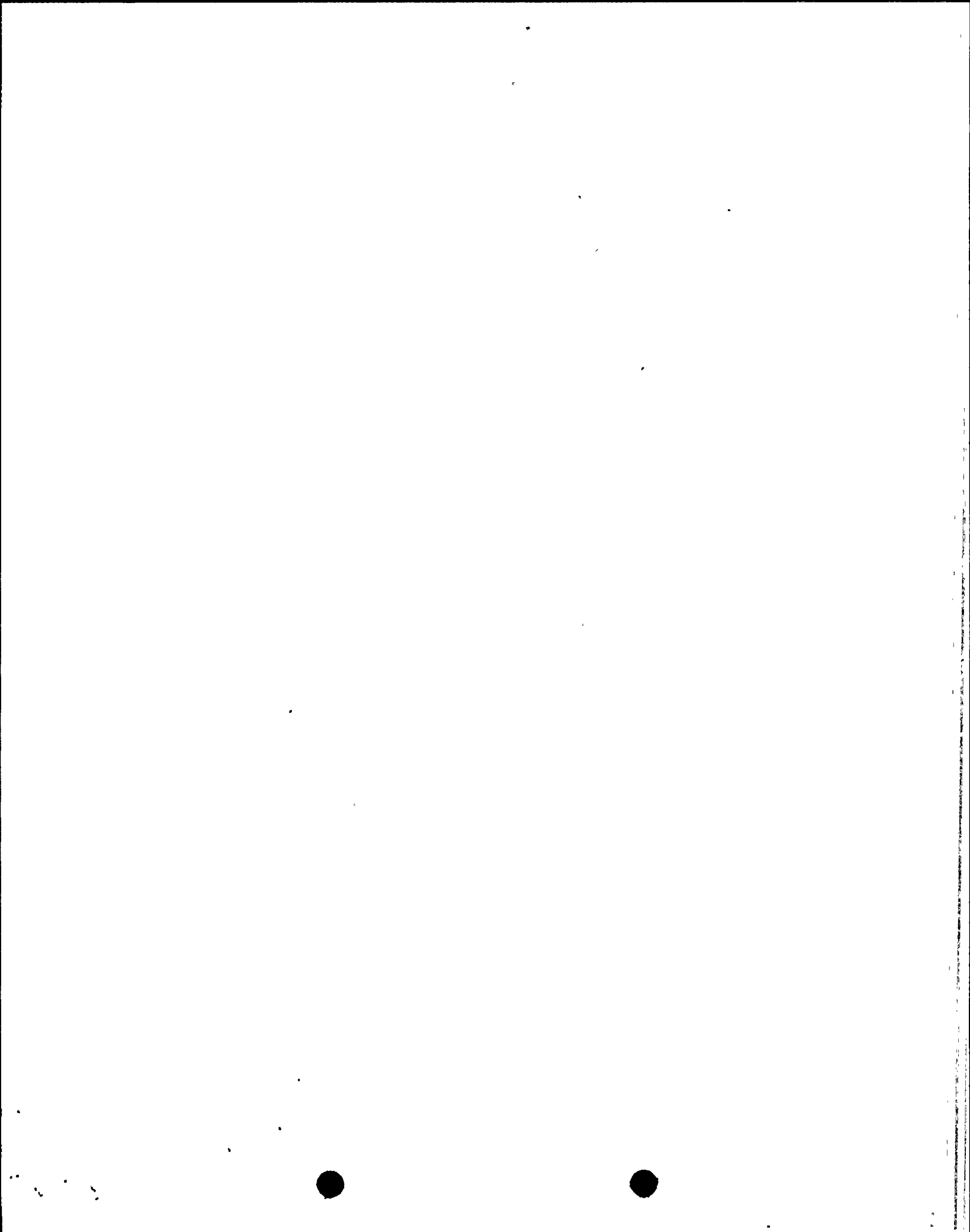
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Washington Public Power Supply System (licensee) dated May 5, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:

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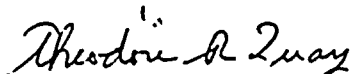


(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 127 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective the date of issuance.

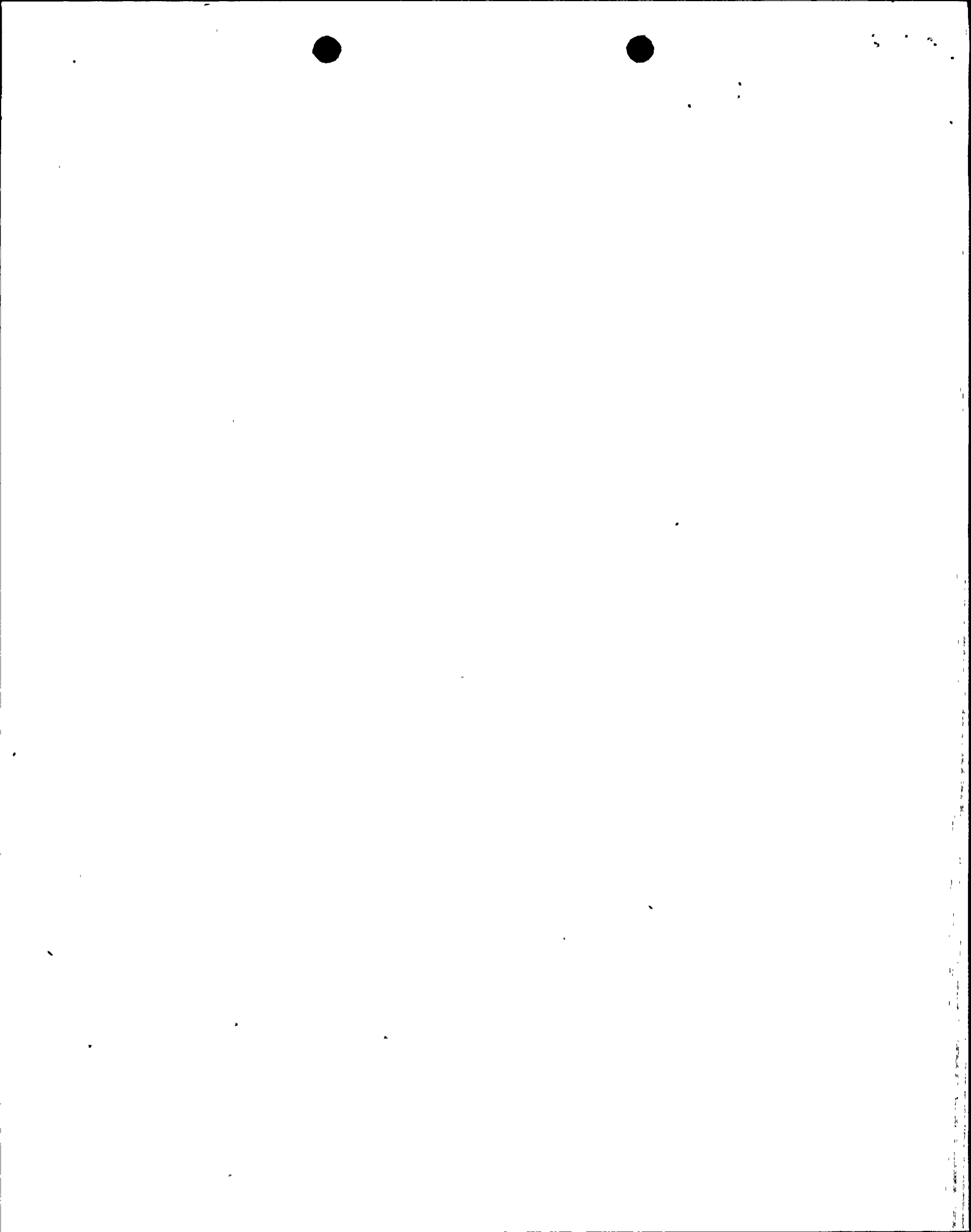
FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director
Project Directorate IV-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 28, 1994



ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 127 TO FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

3/4 6-24

3/4 6-33

INSERT

3/4 6-24

3/4 6-33

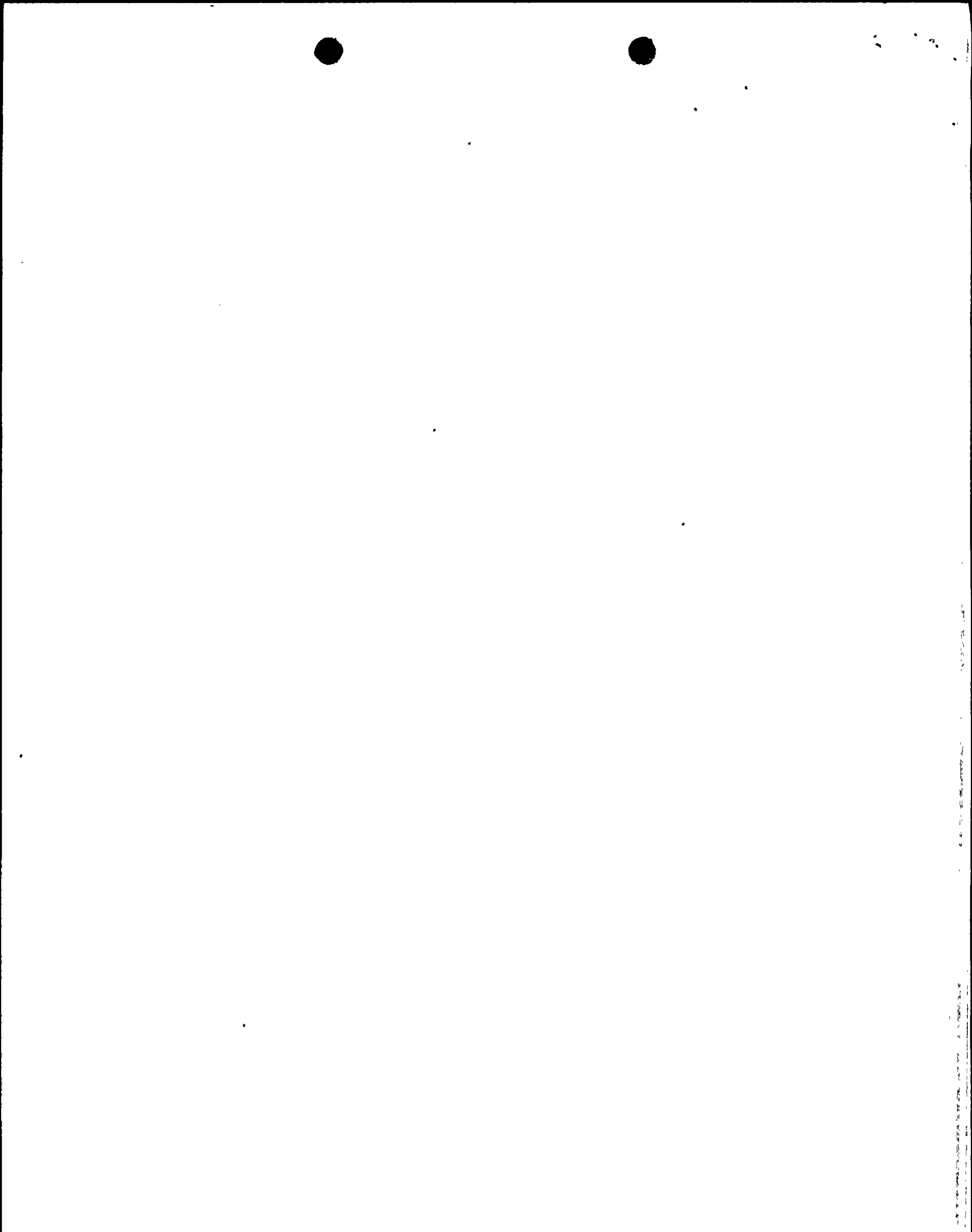


TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

| <u>VALVE FUNCTION AND NUMBER</u> | <u>VALVE GROUP(a)</u> | <u>MAXIMUM ISOLATION TIME (Seconds)</u> |
|--|-----------------------|---|
| a. <u>Automatic Isolation Valves (Continued)</u> | | |
| Reactor Closed Cooling | 4 | 60 |
| RCC-V-5 | | |
| RCC-V-21 | | |
| RCC-V-40 | | |
| RCC-V-104 | | |
| Radiation Monitoring Supply & Return | 4 | 5 |
| PI-VX-250 | | |
| PI-VX-251 | | |
| PI-VX-253 | | |
| PI-VX-256 | | |
| PI-VX-257 | | |
| PI-VX-259 | | |
| Residual Heat Removal | | |
| RHR-V-123A,B(g) | 5 | 15 |
| RHR-V-8(g)(k) | 6 | 40 |
| RHR-V-9(g) | 6 | 40 |
| RHR-V-23(g) | 6 | 90 |
| RHR-V-53A,B(g) | 6 | 40 |
| RHR-V-24A,B(c) | 10 | 270 |
| RHR-V-21 | 10 | 270 |
| RHR-V-27A,B(c) | 10 | 36 |
| Reactor Water Cleanup System | 7 | |
| RWCU-V-1(d) | | 30(j) |
| RWCU-V-4 | | 21(j) |

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

| <u>VALVE FUNCTION AND NUMBER</u> | <u>VALVE GROUP (a)</u> | <u>MAXIMUM ISOLATION TIME (Seconds)</u> |
|--|------------------------|---|
| a. <u>Automatic Isolation Valves (Continued)</u> | | |
| Reactor Core Isolation Cooling | | |
| RCIC-V-8 | 8 | 26 |
| RCIC-V-63 | 8 | 16 |
| RCIC-V-76 | 8 | 22 |
| Low Pressure Core Spray | | |
| LPCS-V-12 | 10 | 180 |
| High Pressure Core Spray | | |
| HPCS-V-23 | 11 | 180 |
| b. <u>Excess Flow Check Valves(e)</u> | | |
| Containment Atmosphere | | N.A. |
| PI-EFC-X29b | | |
| PI-EFC-X29f | | |
| PI-EFC-X30a | | |
| PI-EFC-X30f | | |
| PI-EFC-X42c | | |
| PI-EFC-X42f | | |
| PI-EFC-X61c | | |
| PI-EFC-X62b | | |
| PI-EFC-X69f | | |
| PI-EFC-X78a | | |

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

| <u>VALVE FUNCTION AND NUMBER</u> | <u>VALVE GROUP(a)</u> | <u>MAXIMUM ISOLATION TIME (Seconds)</u> |
|--|-----------------------|---|
| d. <u>Other Containment Isolation Valves (Continued)</u> | | |
| Radiation Monitoring | | N.A. |
| PI-V-X72f/1 | | |
| PI-V-X73e/1 | | |
| Transversing Incore Probe System | | N.A. |
| TIP-V-6 | | |
| TIP-V-7,8,9,10,11(e) | | |

TABLE NOTATIONS

*But greater than 3 seconds.

#Provisions of Technical Specification 3.0.4 are not applicable.

- (a) See Technical Specification 3.3.2 for the isolation signal(s) which operate each group.
- (b) Valve leakage not included in sum of Type B and C tests.
- (c) May be opened on an intermittent basis under administrative control.
- (d) Not closed by SLC actuation signal.
- (e) Not subject to Type C Leak Rate Test.
- (f) Hydraulic leak test at 38.2 psig.
- (g) Not subject to Type C test. Test per Technical Specification 4.4.3.2.2
- (h) Tested as part of Type A test.
- (i) May be tested as part of Type A test. If so tested, Type C test results may be excluded from sum of other Type B and C tests.
- (j) Reflects closure times for containment isolation only.
- (k) During operational conditions 1, 2 & 3 the requirement for automatic isolation does not apply to RHR-V-8. Except that RHR-V-8 may be opened in operational conditions 2 & 3 provided control is returned to the control room, with the interlocks reestablished, and reactor pressure is less than 135 psig.

CONTAINMENT SYSTEMS

3/4.6.4 VACUUM RELIEF

SUPPRESSION CHAMBER - DRYWELL VACUUM BREAKERS

LIMITING CONDITION FOR OPERATION

3.6.4.1 Seven of the nine pairs of suppression chamber - drywell vacuum breakers shall be OPERABLE and all nine pairs shall be closed.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With one or more vacuum breakers in up to two pairs of suppression chamber - drywell vacuum breakers inoperable for opening, verify both vacuum breakers of each pair to be closed within two (2) hours.
- b. With one or more vacuum breakers in three or more pairs of suppression chamber - drywell vacuum breakers inoperable for opening but known to be closed, restore the inoperable pairs of vacuum breakers such that a minimum of seven pairs are in an OPERABLE status within 72 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With one suppression chamber - drywell vacuum breaker open, verify the other vacuum breaker in the pair to be closed within 2 hours; restore the open vacuum breaker to the closed position within 72 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- d. With one closed position indicator of any suppression chamber - drywell vacuum breaker inoperable:
 1. Verify the other vacuum breaker in the pair to be closed within 2 hours and at least once per 15 days thereafter, or
 2. Verify the vacuum breaker(s) with the inoperable position indicator to be closed by conducting a test which demonstrates that the *P is maintained at greater than or equal to 0.5 psi for 1 hour without makeup within 24 hours and at least once per 15 days thereafter.
 3. Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.