

October 26, 1993

Docket No. 50-397

Mr. J. V. Parrish (Mail Drop 1023)
Assistant Managing Director, Operations
Washington Public Power Supply System
P. O. Box 968
Richland, Washington 99352

Dear Mr. Parrish:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION, POWER UPRATE REVIEW
(TAC NO. M87075)

On July 9, 1993, you submitted a request to amend the Facility Operating License and the Technical Specifications for the Washington Public Power System Nuclear Project No. 2. The proposed amendment would increase licensed power from 3323 MWt to 3486 MWt with extended load line limit and a change in safety relief valve setpoint tolerance. Enclosure 1 is a list of the additional information required by our reactor systems reviewers. Please provide the requested information promptly to enable our review on this portion of the submittal to continue on schedule.

Sincerely,

Original signed by:

James W. Clifford, Senior Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

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OGC, 15B18
R. Frahm

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JClifford
ACRS (10), P315

PDV Reading File
TQuay
GKalman
KPerkins, RV

OFFICE	PDV/LA	PDV/PE	PDV/PM	PDV/D
NAME	EBarnhill <i>EB</i>	GKalman <i>GK</i>	JClifford:mc	TQuay <i>TQ</i>
DATE	10/26/93	10/26/93	10/26/93	10/26/93

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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James W. Clifford, Senior Project Manager
Project Directorate V
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Office of Nuclear Reactor Regulation

Enclosure:
As stated

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WNP-2 REQUEST FOR INFORMATION

1. Should RPS and RRC be defined in the glossary of terms?
2. In Section 2.4 of NEDC-32141P, the stability discussion did not contain a commitment to follow the guidance contained in NRC Bulletin 88-07 and the Supplement 1. Do you plan to follow this guidance?
3. In Section 3.4 of NEDC-32141P, the licensee did not commit to perform power uprate tests on the RRC system to demonstrate flow control over the entire recirculation flow range that WNP-2 plans to operate, as discussed in the GE generic report NEDO-31897 (power uprate generic guidance report). These tests provide assurance that no undue vibration or other phenomena occur at power uprate and/or Extended Load Line Limit (ELLL) operation. Is the licensee planning to perform these tests prior to operating in these regimes?
4. In Section 9.3.2 of NEDC-32141P, the Station Blackout (SBO) event is discussed. Will this event be evaluated with power uprate and including ELLL operating conditions to assure that systems needed to respond after power has been restored are designed to operate for the peak suppression pool temperature expected for this event?
5. In Section 9.3.1 of NEDC-32141P, provide an explanation as to why the Loss Of Feedwater (LOFW) event is more limiting for WNP-2 than the Loss Of Offsite Power (LOOP) in the generic Anticipated Transients Without Scram (ATWS) analysis. Provide assurance that the generic evaluation referenced by WNP-2 is valid for SNP fuel. Provide the results of the analysis or reference where they are contained if they are elsewhere in your submittal.
6. The transient analysis results for uprate are presented in Table 9-2 of NEDC-32141P. Please define MCPR_f for the Δ MCPR column for the Slow Recirculation Increase transient.
7. In your SAFER/GESTR-LOCA analysis report (NEDC-32115P) you provide a discussion for Single Loop Operation (SLO) thermal limits. The evaluation assumes there will be no MAPLHGR or PLHGR reduction for SLO. The SLO nominal and Appendix K PCTs are higher than for two loop operation. Provide assurance that the upper bound PCT is less than 1600° F. Are there any reductions on these thermal limits for operating in the ELLL regions such that the PCTs for SLO will be less than for two loop operation? Will there be any increase in the SLMCPR for SLO because of increased uncertainties?
9. In your GE-NE-187-24-0992, Rev.2 report discussing SRV setpoint tolerance and the number of SRVs out-of-service (OOS) evaluation, there appears to be no reference to a generic GE Nuclear Energy Licensing Topical Report (NEDC-31753P). This report explains what plant specific analyses are needed to assure no safety limits are exceeded. There also was an NRC Safety Evaluation Report prepared upon review of this generic report. Why did you not reference these reports?

Mr. J. V. Parrish
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 25, 1993

Docket No. 50-397

Mr. J. V. Parrish (Mail Drop 1023)
Assistant Managing Director, Operations
Washington Public Power Supply System
P.O. Box 968
Richland, Washington 99352

Dear Mr. Parrish:

SUBJECT: BWR TRANSIENT ANALYSIS MODEL (TAC NOS. M77048 AND M81723)

A Safety Evaluation Report (SER) and related Technical Evaluation Report (TER) prepared in response to your Topical Report WPPSS-FTS-129, Rev. 1, BWR Transient Analysis Model, and submitted to the Nuclear Regulatory Commission (NRC) by letter dated September 24, 1990, are enclosed. These reports include evaluations of supplementary information submitted by you in letters dated July 15, 1991, March 5, 1992, and September 11, 1992. Also enclosed is a list of questions prepared by the staff following review of the referenced documents and related information contained in WPPSS-FTS-131, Application Topical Report for BWR Design and Analysis dated September 19, 1991.

As noted in our SER, the staff concluded that WPPSS-FTS-129, Rev. 1 with the supporting submittals is acceptable. However, numerous questions relative to the WNP-2 RETRAN base model and the applications topical, WPPSS-FTS-131, have not been adequately addressed and need to be resolved before either WPPSS-FTS-129, Rev.1 or WPPSS-FTS-131 can be applied at WNP-2. The open issues contained in our TER and the list of questions contained in Enclosure 2 will need to be resolved before licensing basis analyses can be performed using the proposed methodologies. Our acceptance of WPPSS-FTS-129, Rev. 1 is an interim step in the approval process. The TER open issues and questions contained in the enclosures will be addressed as part of the review and approval process of WPPSS-FTS-131.

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Mr. J. V. Parrish .

- 2 -

To expedite approval of WPPSS-131, we propose that you submit a preliminary response to the questions contained in Enclosure 2 and schedule a meeting with the NRC to discuss your responses and the open issues identified in the TER. The NRC is ready to hold the proposed meeting at your earliest convenience. This letter serves as the formal close-out for TAC No. M77048, our tracking number associated with the review of WPPSS-FTS-129.

Sincerely,



James Clifford, Senior Project Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Safety Evaluation
2. Question List

cc: See next page

Mr. J. V. Parrish
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

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Mr. J. V. Parrish

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Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

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Region V, DRP
KPerkins, RV

* See previous concurrence

OFFICE	PDV/LA*	PDV/PE*	PDV/PM <i>✓</i>	SRXB*	PDV/D <i>✓</i>
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DATE	10/20/93	10/19/93	10/19/93	10/14/93	10/25/93

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