UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.117.
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Washington Public Power Supply System (licensee) dated March 10, 1993, with additional information provided March 24, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 117 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Theodore R. Quay

Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical Specifications

Date of Issuance: June 10, 1993

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 117 TO FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE	INSERT
3/4 3-72	3/4 3-72
3/4 3-75	3/4 3-75

TABLE 3.3.7.5-1 (Continued)

ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	REQUIRED NUMBER OF CHANNELS	MINIMUM CHANNELS OPERABLE	APPLICABLE OPERATIONAL CONDITIONS	ACTION
14. Neutron Flux:				
APRM	2	1	1, 2	80
IRM	2	1	1, 2	80
SRM	2	1	1, 2	80
15. RCIC Flow	1	1	1, 2	80
16. HPCS Flow	1	1	1, 2	80
17. LPCS Flow	1	1	1, 2	80
18. Standby Liquid Control System Flow	1	1	1, 2	80
19. Standby Liquid Control System Tank Level	1	1	1, 2	80
20. RHR Flow	1/loop	1/loop	1, 2	80
21. RHR Heat Exchanger Outlet Temperature	1/heat exchanger	1/heat exchanger	1, 2	80
22. Standby Service Water Flow	1/loop	1/loop	1, 2	80
23. Standby Service Water Spray Pond Temperature	2	1	1, 2	80
24.				
25. Emergency Ventilation Damper Position	2/duct	1/duct	1, 2	80
26. Standby Power and Other Energy Sources	2/source	1/source	1, 2	80
27. Primary Containment Valve Position	1/valve	1/line	1, 2	80
28. Primary Containment Gross Radiation Monitors#	2	1	1, 2, 3	81
29.				
30.				
31. Reactor Building Effluent Monitoring System	1	1	1*, 2*, 3*	81

#High range monitors.

*On a one-time basis, within 30 days of the return to power following the 1993 refueling outage, to allow baseline calibration of this system under normal plant operating conditions, the provisions of Specification 3.0.4 are not applicable.

TABLE 4.3.7.5-1 (Continued)

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION	APPLICABLE OPERATIONAL CONDITIONS
18. Standby Liquid Control System Flow	M	R	1, 2
19. Standby Liquid Control System Tank Level	M	R	1, 2
20. RHR Flow	M	R	1, 2
21. RHR Heat Exchanger Outlet Temperature	M	R	1, 2
22. Standby Service Water Flow	M	R	1, 2
23. Standby Service Water Spray Pond Temperature	M	R	1, 2
24.			
25. Emergency Ventilation Damper Position	M	R	1, 2
26. Standby Power and Other Energy Sources	M	R	1, 2
27. Primary Containment Valve Position	M	R	1, 2
28. Primary Containment Gross Radiation Monitors	M	R*	1, 2, 3
29.			
30.			
31. Reactor Building Effluent Monitoring System	M	R	1**, 2**, 3**

TABLE NOTATION

*CHANNEL CALIBRATION shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/h and a one point calibration check of the detector below 10 R/h with an installed or portable gamma source.

**On a one-time basis, within 30 days of the return to power following the 1993 refueling outage, to allow baseline calibration of this system under normal plant operating conditions, the provisions of Specification 3.0.4 are not applicable.