

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 116 TO FACILITY OPERATING LICENSE NO. NPF-21 WASHINGTON PUBLIC POWER SUPPLY SYSTEM

NUCLEAR PROJECT NO. 2

DOCKET NO. 50-397

1.0 INTRODUCTION

By letter dated October 15, 1991, and supplemented by letters dated May 20, 1992, and July 28, 1992, Washington Public Power Supply System submitted a request for changes to the Technical Specifications (TS) for Washington Nuclear Project No. 2 (WNP-2). The proposed change for the Emergency Core Cooling System (ECCS) and the reactor core isolation cooling (RCIC) system instrumentation increases the Allowable Outage Time (AOT) for the surveillance test from 2 hours to 6 hours and specified a time limit for the AOT for repair to 24 hours. Also Surveillance Test Intervals (STIs) have been increased from monthly to quarterly. The staff in its safety evaluation report (memo from S. Newberry to J. Dyer, dated July 15, 1991) had accepted the changes for the ECCS instrumentation and had denied the changes for the RCIC instrumentation. The reason for the staff's denial was that the GE Topical Report GENE-770-06-2 was under staff review at that time. Since then the NRC has accepted the GE Topical Report on a generic basis and each licensee was required to show the specific applicability of this report to their plant. Also each licensee was required to confirm that any increase in instrument drift due to the extended STIs is properly accounted for in the setpoint calculation methodology.

2.0 EVALUATION

By letter dated October 15, 1991, WPPSS resubmitted the request for the proposed changes to the TS for the RCIC instrumentation. In this request the licensee has stated that they reviewed the setpoint drift characteristics and confirmed that the setpoint will remain within existing allowances for the requested STI extension. The licensee has documented this analysis and it is available for future staff audit. This satisfies one of the conditions set in the staff's SER of the Topical Report.

The licensee submittal did not address the second condition which required the licensee to confirm the applicability of the generic analysis to WNP-2. By letter dated May 20, 1992, the licensee confirmed the applicability of the generic report to WNP-2 by an analysis performed by GE and documented in a GE Report RE-024, DRF A00-02558 dated March 1987. This report compares the RCIC

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system design, support systems and instrumentation between the plant specific configuration versus configuration used in the generic analysis.

The report identified two plant specific differences from the generic configuration. However, it did not provide any basis for the acceptability of these differences. The differences dealt with RCIC instrumentation with respect to high drywell pressure and high suppression pool water level. In the generic plant analysis high drywell pressure concurrent with low steam line pressure is used for isolation of the RCIC turbine exhaust system to protect against operation at high pressures. In WNP-2, the high drywell pressure signal is not used for isolation purposes. Also in the generic plant analysis a high suppression pool water level signal automatically switches RCIC suction from the condensate storage tank to the suppression pool to ensure that suppression pool loads, as a result of excess water, are not exceeded. At WNP-2, this transfer is done manually.

The licensee by their letter of July 28, 1992, provided the basis for the acceptability of these differences. In both cases, the differences do not significantly affect the RCIC availability and overall water injection failure frequency and, therefore, the generic analysis is applicable to WNP-2.

Based on the review of the plant specific analysis for the RCIC system the staff concludes that the licensee has adequately addressed the conditions set forth by the staff in endorsing the GE Topical Report GENE-770-06-2 and has provided acceptable support for the proposed extensions for STIs for RCIC from monthly to quarterly and AOT from 2 hours to 6 hours for test and 1 hour to 24 hours for repair.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Washington State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 20520). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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