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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 116 License No. NPF-21

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Washington Public Power Supply System (licensee) dated October 15, 1991 and supplemented by letters dated May 20, 1992 and July 28, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:

The Technical Specifications contained in Appendix A, as revised through Amendment No. 116 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Theodore A. Quay

Theodore R. Quay, Director Project Directorate V Division of Reactor Projects III/IV/V Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

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Date of Issuance: June 10, 1993

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ATTACHMENT TO LICENSE AMENDMENT .

AMENDMENT NO. 116 TO FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

<u>INSERT</u> 3/4 3-48

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3/4 3-48	3/4 3-48
3/4 3-49	3/4 3-49
3/4 3-51	3/4 3-51
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INSTRUMENTATION

3/4.3.5 REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.5 The reactor core isolation cooling (RCIC) system actuation instrumentation channels shown in Table 3.3.5-1 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3.5-2.

<u>APPLICABILITY</u>: OPERATIONAL CONDITIONS 1, 2 and 3 with reactor steam dome pressure greater than 150 psig.

ACTION:

- a. With a RCIC system actuation instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values
 column of Table 3.3.5-2, declare the channel inoperable until the channel is restored to OPERABLE status with its trip setpoint adjusted consistent with the Trip Setpoint value.
- b. With one or more RCIC system actuation instrumentation channels inoperable, take the ACTION required by Table 3.3.5-1.

SURVEILLANCE REQUIREMENTS

4.3.5.1 Each RCIC system actuation instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.5.1-1.

4.3.5.2 LOGIC SYSTEM FUNCTIONAL TESTS and simulated automatic operation of all channels shall be performed at least once per 18 months.

TABLE 3.3.5-1

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>FUNCTIO</u>	NAL_UNITS	MINIMUM OPERABLE CHANNELS _ <u>PER_TRIP_SYSTEM(a)</u> _	<u>ACTION</u>
a.	Reactor Vessel Water Level - Low Low, Level 2	× 2	50
b.	Reactor Vessel Water Level - High, Level 8	2(b)	51
c.	Condensate Storage Tank Water Level - Low Low	2(c)	52
d.	Manual Initiation	1(d)	53

(a) A channel may be placed in an inoperable status for up to 6 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.

(c) One trip system with one-out-of-two logic.

(d) One trip system with one channel.

⁽b) One trip system with two-out-of-two logic.

TABLE 3.3.5-1 (Continued)

REACTOR CORE ISOLATION COOLING SYSTEM

ACTUATION INSTRUMENTATION

ACTION STATEMENTS

- ACTION 50 With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement:
 - a. For one trip system, place the inoperable channel(s) and/or that trip system in the tripped condition within 24 hours or declare the RCIC system inoperable.
 - b. For both trip systems, declare the RCIC system inoperable.
- ACTION 51 With the number of OPERABLE channels less than required by the minimum OPERABLE channels per Trip System requirement, restore the number of operable channels to that required by the minimum OPERABLE channels per Trip System requirement within 24 hours or declare the RCIC system inoperable.
- ACTION 52 With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement, place at least one inoperable channel in the tripped condition within 24 hours or declare the RCIC system inoperable.
- ACTION 53 With the number of OPERABLE channels one less than required by the Minimum OPERABLE Channels per Trip System requirement, restore the inoperable channel to OPERABLE status within 24 hours or declare the RCIC system inoperable.

TABLE 3.3.5-2

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

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FUNCTIONAL UNITS		TRIP SETPOINT	ALLOWABLE VALUE
a.	Reactor Vessel Water Level - Low Low, Level 2	<u>></u> - 50 inches*	\geq -57 inches
b.	Reactor Vessel Water Level - High, Level 8	<u><</u> 54.5 inches*	56 inches
c.	Condensate Storage Tank Level - Low Low**	> 448 ft 3 in. elevation (1 ft 9 in. tank level)	> 448 ft 0 in. elevation (1 ft 6 in. tank level)
d.	Manual Initiation	N.A.	N.A.

*See Bases Figure B 3/4 3-1.

**Provides automatic transfer from Condensate Storage Tank to the Suppression Pool.

TABLE 4.3.5.1-1

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REACTOR CORE_ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION_SURVEILLANCE_REQUIREMENTS

FUNCTIC	NAL UNITS	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL <u>CALIBRATION</u>
a.	Reactor Vessel Water Level - (Low Low, Level 2)	S	Q	R
b.	Reactor Vessel Water Level - High, Level (8)	S	Q	R
с.	Condensate Storage Tank Level - Low	S	Q	R
d.	Manual Initiation	N.A.	R	N.A.

INSTRUMENTATION

3/4.3.6 CONTROL ROD BLOCK INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.6. The control rod block instrumentation channels shown in Table 3.3.6-1 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3.6-2.

APPLICABILITY: As shown in Table 3.3.6-1.

ACTION:

- a. With a control rod block instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3.6-2, declare the channel inoperable until the channel is restored to OPERABLE status with its trip setpoint adjusted consistent with the Trip Setpoint value.
- b. With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, take the ACTION required by Table 3.3.6-1.

SURVEILLANCE REQUIREMENTS

4.3.6 Each of the above required control rod block trip systems and instrumentation channels shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations for the OPERATIONAL CONDITIONS and at the frequencies shown in Table 4.3.6-1.