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Table 3.2.3-1 MCPR OPERATING LIMITS

MCPR Operating Limit

	t		UD to 105% Lore Flow				
	Cycle Exposure	Equipment Status	GE/Fuel	ANF Fue T	SVEA -96 LFA FUEL		
1.	0 MWD - 3750 MWD MTU MTU	* - This transmittal	. 24	1.24	1.37		
2.	3750 MWD - EOC MWD + + +	Hormal scram times**	1.35	1.31	1.48		
3.	3750 MWD - EOC MWD ***** MTU HTU	) Control rod insertion bounded by Tech. Spec. limits (3.1.3.4 - p 3/4 1-8)	1.42	1.36 Jr38	1.55		
4.	3750 MWD - EOC MWD MTU MTU	RPT inoperable Normal scram times**	1.42	<i>1.36</i> 1.38 ل	1.55		
<sup>`</sup> 5.	3750 MWD - EOC MWD MTU MTU	RPT inoperable Control rod insertion bounded by Tech. Spec. limits (3.1.3.4 - p 3/4 1-8)	. 1.48	1.40 J.42	1.61		
б.	0 MWD - EOC MWD MTU MTU	Single loop operation RPT operable Normal scram times**	1.35	1.35	1.54		

\*\*In this portion of the fuel cycle, operation with the given MCPR operating limits is allowed for both normal and Tech. Spec. scram times and for both RPT operable and inoperable.

\*\*These MCPR values are based on the ANF Reload Safety Analysis performed\_using the control rod insertion times shown below (defined as normal scram). In the event that surveillance 4.1.3.2 shows these scram insertion times have been exceeded, the plant thermal limits associated with normal scram times default to the values associated with Tech. Spec. scram times (3.1.3.4-p 3/4 1-8), and the scram insertion times must meet the requirements of Tech. Spec. 3.1.3.4.

	Position Inserted From Fully Withdrawn	Slowest measured average control rod insertion times to specified notches for all operable control rods for each group of 4 control rods arranged in a <u>a two-by-two arrav (seconds)</u>					
•	Notch 45 Notch 39 Notch 25 Notch 5		.404 .660 1.504 2.624	:			
WAS	HINGTON NUCLEAR - UNIT 2	3/4 2-7			Amendment	No.	59
	9004230574 900 PDR ADOCK 0500	413 00397					

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- The GE11 LFA fuel, the ANF LFA fuel and the GE initial core fuel are also monitored to the ANF 8x8 fuel MCPR Operating Limits (Reference; Power Distribution Limits, Bases, 3/4.2.3; Minimum Critical Power Ratio, p. B 3/4 2-3).
- \*\*\*\* For Final Feedwater Temperature Reduction rated conditions beyond all rods out point, add .02 to the MCPR for all fuel in the WNP-2 core except for the SVEA-96 LFA fuel. For the SVEA-96 LFA fuel; add .03 to the MCPR for Final Feedwater Temperature Reduction rated conditions beyond the all rods out point.

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