ACCELERATED

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

DEMONSTRATION

SYSTEM

DISTRIBUTION

ACCESSION FACIL: 50 AUTH.NA WASHING POWERS,0 RECIP.1	N NBR:8905040189 D-397 WPPSS Nuclear AME AUTHOR A FON,S.L. Washingto C.M. Washingto NAME RECIPIEN	DOC.DATE Project, FFILIATIO n Public n Public T AFFILIA	: 89/04/26 NOTARIZED Unit 2, Washington P N Power Supply System Power Supply System TION	: NO ublic Powe	DOCKET # 05000397 R
SUBJECT DISTRIBU TITLE: !	: LER 88-030-00:on power on both RPS JTION CODE: IE22D 50.73/50.9 Licensee	880825,RP divs due COPIES RE Event Re	S actuations caused b to misapplication of CEIVED:LTR / ENCL / port (LER), Incident	y loss of switch type W/8 ltr SIZE: Rpt, etc.	. I 2 D 2 S
NOTES:	н. 1910 — П.		,		1
INTERNAL:	RECIPIENT ID CODE/NAME PD5 LA SAMWORTH,R ACRS MICHELSON ACRS WYLIE AEOD/DSP/TPAB DEDRO NRR/DEST/ADE 8H	COPIES LTTR ENC 1 1 1 1 1 1 1 1 1 1 1 1 1 1	RECIPIENT L ID CODE/NAME PD5 PD ACRS MOELLER AEOD/DOA AEOD/ROAB/DSP IRM/DCTS/DAB NRR/DEST/ADS 7E	COPIES LTTR ENCL 1 1 2 2 1 1 2 2 1 1 2 2 1 1 1 0	A D D S
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EXTERNAL:	EG&G WILLIAMS,S L ST LOBBY WARD NRC PDR NSIC MURPHY,G.A	4 4 1 1 1 1 1 1	FORD BLDG HOY,A LPDR NSIC MAYS,G	1 1 1 1 1 1	-I D

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NRC Form 366 (9-83)			LIC	ENSEE EV	ENT RE	PORT	(LER)	U.S. NU	CLEAR REGULATORY (APPROVED OMB NO. 3 EXPIRES: 8/31/88	COMMISSION 150-0104	
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Washingt	<u>on Nuclea</u> r	<u>^ Plant - U</u>	nit 2			<u> </u>		0 5 0 0	03971	<u> 0F 0 5</u>	
TITLE (4) RPS	Actuation	ns Caused B	y Loss	Of Power	On Bot	n RPS	Division	s - Due T	o Mis-Applio	cation.	
Of Switch Type											
EVENT DAT	E (5)	LER NUMBER (6)	REPORT D	ATE (7)		OTHER	FACILITIES INVO	LVED (8)		
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This sup from not	plemental reportab	report ch le per 10CF	anges t R50.73	the repor to repor	tabili table.	ty st	atus of	the Augus	t 26, 1988	event	
On Augus Reactor performa Protecti which si	t 25, 19 Protection nce of a on Assemb multaneous	88 at 2211 on System channel t ly (EPA) I sly deenerg	hours (RPS) function preakers ized bo	and aga actuati nal test s. The oth divis	in on on oc and cause ions of	Augus currec calibr of tl f RPS.	t 26, 19 d. Both ration of hese ever	88 at 15 events f the RP its is su	53 hours, a occurred S Bus Elec witch overt	during trical ravel,	
The loss of both divisions of RPS power also causes the isolation of Nuclear Steam Supply Shutoff System (NS4) Groups 1,2,4 (partial only), 5,6 and 7. All NS4 isolation valves either actuated as designed or were closed prior to the NS4 trip. The outboard mainsteam line isolation valves were closed prior to both events. Both Residual Heat Removal Shutdown Cooling Loop B and Reactor Water Cleanup System valves were isolated just prior to each event in anticipation of the momentary loss of power of an RPS Bus which would, by design, isolate the outboard and/or inboard containment isolation valves for these systems.											
Plant Operators responded to both events by returning the RPS power supply select switch to the normal position and resetting the scram. For both events, RHR Shutdown Cooling Loop B was returned to operation within 10 minutes and the RWCU System was placed back in service within 12 minutes.											
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NRC Form 366 (9 83)	-	·		s.							

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NRC Form 366A		U.S. NUCLEAR REGULATORY COMMISSION
(9-83)	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION	APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/88

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Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	8 8 _ 0 3 0 _ 0 1	0 2 0 0 0 5
TEXT (If more space is required, use additional NRC Form 366A's) (17)			

Abstract (cont'd)

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The root cause of these events is the design limitations of the power transfer switch. The switch is a break-before-make configuration and is subject to overtravel during manipulation.

An engineering analysis will be performed to evaluate options for a more reliable power transfer scheme. Additionally, a caution tag was placed on the switch to serve as a reminder that the switch is not mechanically prevented from overtraveling the desired position.

There is no safety significance associated with these RPS actuations since there was no actual initiating event. All safety actuations occurred as designed. This event posed no threat to the health and safety of the public or Plant personnel.

Plant Conditions

Power Level - 0% PLant Mode - 4 (Cold Shutdown)

Event Description

This supplemental report changes the August 26, 1988 RPS actuation from not reportable per the requirements of IOCFR50.73 to reportable. The change in reportability status is a corrective action in our response to Notice of Violation "B" Inspection Report 88-24.

On August 25, 1988, at 2211 hours, while performing a channel functional test and calibration of the Reactor Protection System (RPS) Bus "A" Electrical Protection Assembly (EPA) breakers, a full RPS actuation occurred due to a momentary loss of power to both divisions of RPS. At the time of the event, a Plant Operator was transferring RPS "A" power from its alternate to normal power supply as called for in the procedure. During the transfer the RPS Power Supply Select switch overtraveled, deenergizing both RPS Bus "B" and "A".

On August 26, at 1553 hours, a second identical event occurred while performing a channel functional test and calibration of the RPS Bus "B" EPA breakers. The full RPS actuation occurred due to the momentary loss of power to both divisions of RPS. Because of the RPS actuation on the previous day, it was discussed and recognized that transferring from the alternate to normal power supply could cause a full RPS actuation. Management direction was given to proceed with the power supply transfer to help in understanding the event of the previous day.

The loss of both divisions of RPS power causes the isolation of Nuclear Steam Supply Shutoff System Groups 1,2,4 (partial only), 5,6, and 7. For these events the following actuations occurred:

NS⁴ Group 1 - Main Steamline Isolation and Drainline Valves (MSIVs). For the August 25, 1988 event, the inboard (MSIVs) isolated. The inboard (MSIVs) remained closed after the August 25, 1988 event and were closed at the time of the August 26, 1988 event. The outboard MSIVs were closed prior to both events. All drainline valves were closed prior to both events.

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C Form 366A		T (LER) TE	XT CON	TINU	ATIS		U.S. 1	UCLEAR RE APPROVED (EXPIRES: 8/3	GULATOR	Y COM	MISSION
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NS ⁴ Group 2 - Reactor W events.	ater Sample	e Valves	. These	e va	lves	were	close	ed prio	r to	bot	:h [`
NS ⁴ Group 4 - Miscella group (floor drain and v valves isolated during b	neous Bala equipment o oth events	ance of drain val	Plant Ives) a	(BOP re co). (ontro	Only 11ed	four by RI	valves PS powe	in er.	thi Thes	s je
Group 5 - RHR and Trav closed prior to both eve	ersing Inc nts.	ore Prob	e (TIP)). ' /	11 1	alve	s in	this g	ròup	wer	e
Group 6 - RHR - Shutdo discussion above, the o momentary loss of an RPS in this group. As a r events.	wn Cooling perating R Bus which esult, all	System. HR "B" would i valves	Prior Loop wa solate in thi	r to s is the s gr	both olate outbo oup	eve ed ir bard were	ents, anți and/o close	as not cipatio r inboa d prio	ed in on of ard v r to	n th f th alve bot	ie ie :s :h
Group 7 - RWCU System. and/or inboard isolatio during the power supply events.	Again, pri n•valve c transfer, a	or to bo losing d all valve	th even ue to es in tl	ts, i the his g	in an momer group	ticip Itary Were	ation loss clos	of the of an ed pric	e out n RPS or to	boar Bu bot	rd is :h
In addition, the design loses power for an exten Exhaust Plenum Process Engineered Safety Featur power transfer and for b the capacitance inheren circuitry.	of the RPS ded period Radiation re Trip) ir ooth of the t in the	power su of time Monitor the aft se event power su	upply is , the t 's ("Z" fected s, the upplies	s suc rip Sig RPS o trip of	h th relay nal) divis rela the	at if /s of wil ion. Ays w Proce	f eith the I dee Duri ere u ess Ra	er RPS Reacton nergize ng a r naffect adiatio	powe Bui (A norma ced du n Mo	r bu ldir Nor I RF ue t nitc	is ig i 'S ;o
Immediate Corrective Act	ion							T			[
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Plant Operators responde switch to normal and res to operation at 1601 hou other NS ⁴ isolations wer	ed to the A setting the rs. The R e reset and	August 20 scram. WCU syste d returne	5, 1988 RHR Sh em was l ed to th	ever utdo back leir	nt by wn Co in s pre-e	pla polin ervic event	cing g Loop e at lineu	the RPS D B was 1603 he P•	S tra s res ours.	nsfe tore Al	r d 1
Further Evaluation and C	orrective A	Action		-			,				
A. Further Evaluation							-				
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NRC Form 366A (9-83)	PORT (LER) TEXT CONTI	NUATION U.S. NUCLEA	REGULATORY COMMISSION /ED OMB NO. 3150-0104 : 8/31/88
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Washington Nuclear Plant - Unit	2 0 15 10 10 10 13 1 9	7818_01310_0	10140F 015

TEXT (If more space is required, use additional NRC Form 366A's) (17) The cause of both of these events is RPS power supply transfer switch overtravel. The root cause is the design limitation of the switch, which has too little operational tolerance. The overtravel was caused by a lack of understanding of the design limitations of the switch. The switch is a three position switch which is normally in the normal (middle) position. The switch can be moved to a left or right position to transfer either RPS bus to the The switch uses a break-before-make contact alternate power source. arrangement, thus when the switch is moved, the existing power supply circuit is broken before the new power supply circuit is made. The design limitation is that any movement past the middle switch position can break the other RPS circuit contacts, causing a loss of RPS power to both buses. Because of this break-before-make design, operators are sensitive to the need to transfer power quickly and in doing so, can cause the switch to overtravel.

The RPS power supply transfer switch is manufactured by General Electric, Model Number SBM.

B. Further Corrective Action

A Technical Evaluation Request was initiated to evaluate options for a more reliable power transfer scheme.

A caution tag was placed on the switch to serve as a reminder that the switch is not mechanically prevented from overtraveling the desired postion and that overtravel could cause a full RPS actuation.

LER 88-030-00 was read by all licensed personnel on the Operations Staff.

As part of requalification training, LER 88-030-00 was reviewed with all operations staff. In conjunction with this review, direction was provided on proper RPS power supply select switch operation.

Safety Significance

There is no safety significance associated with these RPS actuations since there was no actual initiating event. All safety actuations occurred as designed. The RHR Shutdown Cooling System was restored within 10 minutes following both events. The Technical Specification requirements for restoring Shutdown Cooling while in a Cold Shutdown condition is 1 hour. Accordingly, this event posed no threat to the health and safety of the public or Plant personnel.

Similar Events

LER 87-20 - A failure of the same switch described in LER 87-20 is different from the overtravel of the switch in this event. In the LER 87-20 event, the stop tab on the switch was physcially broken and did not prevent the switch from overtravelling when moved to an alternate power supply position. In this event the switch functioned as designed and there were no component failures.

" LICENSEE EVENT REPO	DRT (LER) TEXT CONTINU	JATIO		APPROVED O EXPIRES: 8/31	MB NO, 3 1/88	150-01	104
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			System	Compor	ient		
Reactor Protection System (RPS)			JC				
RPS Electrical Protection Assemb	1y'(EPA) .	`	JC	BKF	2		
RPS-Bus-A and B			JC	BU			
RPS Power Supply Select Switch	•		JC	JS			
Nuclear Steam Supply Shutoff Sys	tem (NS ⁴)		BD				7
Reactor Water Sample Valves			AD	V			
Miscellaneous Balance of Plant (BOP)		BD				
RHR and Traversing Incore Probe	(TIP) System		IG				
RHR Shutdown Cooling System			BD				
Reactor Water Cleanup (RWCU) Sys	tem		CE				
Reactor Building Exhaust Plenum	Process Radiation Syst	cem .					
Standby Gas Treatment System (SG			BH				
Control Room Emergency Filtratio	n System		V M V A				
Reactor Building ventilation Sys			VA CD		~ -		

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ACCELERATED DISTRIBUTION DEMONSTRATION

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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

SYSTEM

ACCESSION FACIL:50 AUTH.N. WASHING POWERS,0	N NBR:8905040189 0-397 WPPSS Nuclear AME AUTHOR A TON,S.L. Washingto C.M. Washingto	DOC.DATE: Project, AFFILIATION on Public F on Public F	89/04/26 NOTARIZED Unit 2, Washington P Ower Supply System Power Supply System	: NO DO ublic Powe OS	OCKET # 5000397
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Reacto	or Pr	otect	non	System	(KPS functi) lon	act	uatio	n oc and	curr	eo be	1. BOTH	l f	event +ho	S DDC	OCCU Rus	rrec s Fl	loctr	ring ical
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which	simul	tanen	usjv Usjv	deenero	ized	bo.	th d	ivisi	ons o	FRPS	s.			. 13	эп		010		,
	STINUT	vungu	43 i J	accillery			V				- •								•
The lo	oss of	both	div	isions c	of RPS	p	ower	also	caus	es t	:he	e isolati	on	of N	luc]	ear	Ştea	am Su	pply
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valves	s eith	ier ac	tuat	ed as c	lesign	ed	or	were	close	ed p	ri	or to th	ne	NS4	trij	p	The	outb	oard
mainst	team	line	isol	ation va	alves	We	ere	close	d pri	or 1	to	both ev	'en	ts.	Bot	h R	esid	ual	Heat
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wnich	WOUIO	, Dy vetom	aesi	iyn, 150	Idte	ciie	e ou	inoar	u diia	/ Ur	11	ibuaru Cu	116	arnine	116	1201	aut	/11 VQ	1753
	11636 2	y s cell	3.			1													
Plant	Opera	tors	resp	onded to	o botł	n e	event	s by	retu	ning	g .	the RPS	po	wer s	upp	ly s	elec	t sw:	itch
to th	e norr	nal p	osit	ion and	reset	tti	ing t	the s	cram.	Fo	r	both eve	ent	ts, RI	HR	Shut	down	Coo	ling
Loop E	B was	retur	ned	to opera	ation	wi	thin	10 m	inute	s an	d	the RWCU	JS	ystem	wa	s pl	aced	i bac	k in
servio	ce wit	hin 1	2 mi	nutes.										-	~ "	7			1
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NRC Form 366 (9 83)

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(RC Form 366A 9-83) LICENSEE EV	ENT REPORT (LER) TEXT CONTIN	UATION	APPROVED OM EXPIRES: 8/31/8	LATORY COMM B NO. 3150-010 8
ACILITY NAME (1)	DOC	KET NUMBER (2)	LER NUMBER	(6)	PAGE (3)
			YEAR SEQUENTI	AL WREVISION	
Washington Nuclear Plant -	Unit 2 o	5 0 0 0 3 9 7	8 8 - 0 3 0	0 <mark>_0 1 0</mark>) 2 0F
EXT (If more spece is required, use additional NRC Form 366A's) (Abstract (cont'd)	(17)				
The root cause of thes switch. The switch i overtravel during manipu	e events is s a break-l lation.	the design limi pefore-make conf	tations of t iguration an	he power d is sub	transfe ject to
An engineering analysis power transfer scheme. serve as a reminder that the desired position.	will be per Additionally the switch	formed to evalua /, a caution ta is not mechanica	te options fo g was placed ly prevented	r a more on the s from overt	reliable witch te craveling
There is no safety signi no actual initiating eve posed no threat to the h	ficance assoc ent. All safe ealth and saf	ciated with these ety actuations of ety of the publi	RPS actuatio courred as des c or Plant per	ns since t igned. Th sonnel.	chere wa: nis even
Plant Conditions					
Power Level - 0% PLant Mode - 4 (Cold Sh	utdown)				
Event Description					
This supplemental repo reportable per the re reportability status is "B" Inspection Report 88	rt changes quirements o a corrective -24.	the August 26, f 10CFR50.73 t action in our f	1988 RPS ac o reportable. response to N	tuation f The cl otice of \	from no nange i /iolatio
On August 25, 1988, at calibration of the Read Assembly (EPA) breakers power to both divisions transferring RPS "A" pow in the procedure. Du overtraveled, deenergizi	2211 hours, ctor Protecti , a full RPS ; of RPS. A wer from its wring the tung both RPS B	while performing on System (RPS) actuation occur t the time of t alternate to non ransfer the RPS us "B" and "A".	a channel fi Bus "A" Ele red due to a he event, a l mal power sup S Power Supp	unctional ctrical Pi momentary Plant Oper oply as ca ly Select	test an rotectio loss o ator wa lled fo switc
On August 26, at 1553 I channel functional test RPS actuation occurred o Because of the RPS actu that transferring from actuation. Management o to help in understanding	hours, a seco and calibrat lue to the mon nation on the the alternate lirection was the event of	ond identical even ion of the RPS H mentary loss of previous day, to normal power given to proceed the previous da	ent occurred b Bus "B" EPA b power to both it was discus supply could with the pow y.	while perf reakers. divisions sed and re cause a ver supply	orming The ful of RPS ecognize full RP transfe
The loss of both divisio Shutoff System Groups following actuations occ	ns of RPS pou 1,2,4 (partia urred:	ver causes the is al only), 5,6,	solation of Nu and 7. For	clear Stea these ev	am Suppl ents th
NS ⁴ Group 1 - Main St August 25, 1988 event, closed after the August 1988 event. The outboa valves were closed prior	ceamline Isol the inboard 25, 1988 ever ard MSIVs wer to both ever	ation and Drain (MSIVs) isolated ot and were close oe closed prior ots.	line Valves . The inboar d at the time to both event	(MSIVs). d (MSIVs) of the Au ts. All o	For th remaine Igust 26 Irainlin

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NRC Form 366A (9-83)		T (LER) TEXT C	ONTINUATIO		CLEAR REGULATO	RY COMMISSION 3150-0104
FACILITY NAME (1)		DOCKET NUMBER (2)	YEAF	LER NUMBER (6)	REVISION	PAGE (3)
Washington Nuclear	<u> Plant - Unit 2</u>	0 5 0 0 0	3 9 7 8 8	<u>3 — p 3 0 –</u>	-0 1 0 3	3 OF 0 5
TEXT (If more space is required, use edditional NRC NS ⁴ Group 2 - Re events.	Form 3884's) (17) actor Water Sampl	e Valves. Th	ese valves	were closed	l prior to	both
NS ⁴ Group 4 - M group (floor drai valves isolated d	liscellaneous Bal n and equipment uring both events	ance of Plan drain valves) •	t (BOP). are contr	Only four volled by RPS	valves in S power.	this These
Group 5 - RHR an closed prior to b	nd Traversing Inc oth events.	core Probe (T	IP). A11	valves in t	his group	were
Group 6 - RHR - discussion above momentary loss of in this group. events.	Shutdown Cooling the operating I an RPS Bus which As a result, all	g System. Pr RHR "B" Loop h would isola I valves in t	ior to bot was isola te the out his group	th events, a ted in antic board and/or were closed	s noted i ipation o inboard y prior to	n the of the valves both
Group 7 - RWCU Sy and/or inboard i during the power events.	stem. Again, pri solation valve c supply transfer,	ior to both ev losing due to all valves in	ents, in a the mome this grou	nticipation entary loss p were close	of the ou of an RP d prior to	tboard 'S Bus o both
In addition, the loses' power for a Exhaust Plenum I Engineered Safety power transfer ar the capacitance circuitry.	design of the RPS in extended period Process Radiation Feature Trip) i id for both of the inherent in the	power supply d of time, the Monitors (" n the affecte ese events, th power suppli	is such t trip rel Z" Signal d RPS divi ne trip re es of the	hat if eithe ays of the R) will deen sion. Durir lays were un Process Rad	r RPS powe eactor Bu ergize (A ng a norma affected d diation Me	er bus ilding Non- al RPS due to onitor
Immediate Correct	ive Action					
Plant Operators r switch to normal to operation at 2 NS ⁴ isolations exception of the	esponded to the A and resetting the 220 hours. The F were reset and inboard MSIVs whi	August 25, 198 e scram. RHR RWCU system wa returned to ch were left	8 event by Shutdown s back in their pu closed.	v returning t Cooling Loop service at 2 re-event lir	he RPS tra B was re 223 hours heup, wit	ansfer turned . All h the
Plant Operators switch to normal to operation at l other NS ⁴ isolati	responded to the and resetting the 601 hours. The F ons were reset an	August 26, 19 e scram. RHR RWCU system wa nd returned to	88 event b Shutdown s back in their pre-	by placing t Cooling Loop service at l -event lineup	he RPS tr B was re 603 hours).	ansfer stored . All
Further Evaluation	n and Corrective	Action				
A. <u>Further Eval</u>	uation					
These event Feature Act reportabilit	s are reportable uation. This s y of the second R	e per 10CFR5 supplemental RPS actuation.	0.73(a)(2) report is	(iv) an Eng submitted	jineered to chang	Safety e the

There were no structures, systems, or components inoperable prior to this event which contributed to the event.

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NRC Form 366A (9-83)	LICENSEE EVENT REPORT (LER) TEXT CONTINU	ATION U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/88
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
		YEAR SEQUENTIAL REVISION NUMBER
Washin	gton Nuclear Plant - Unit 2 o 5 o o 3 9 7	8 8 0 3 0 0 1 0 4 of 0
TEXT (If more spece is	The cause of both of these events is RPS overtravel. The root cause is the design limit too little operational tolerance. The overtra understanding of the design limitations of the position switch which is normally in the normal can be moved to a left or right position to to alternate power source. The switch uses arrangement, thus when the switch is moved, the is broken before the new power supply circuit i is that any movement past the middle switch po circuit contacts, causing a loss of RPS power t break-before-make design, operators are sensitiv quickly and in doing so, can cause the switch to	power supply transfer switch ation of the switch, which has avel was caused by a lack of switch. The switch is a three (middle) position. The switch transfer either RPS bus to the a break-before-make contact e existing power supply circuit s made. The design limitation sition can break the other RPS to both buses. Because of this e to the need to transfer power overtravel.
	The RPS power supply transfer switch is manufact Number SBM.	ured by General Electric, Model
В.	Further Corrective Action	
	A Technical Evaluation Request was initiated treliable power transfer scheme.	o evaluate options for a more
	A caution tag was placed on the switch to serve is not mechanically prevented from overtravelin overtravel could cause a full RPS actuation.	e as a reminder-that the switch og the desired postion and that
	LER 88-030-00 was read by all licensed personnel	on the Operations Staff.
	As part of requalification training, LER 88- operations staff. In conjunction with this re- proper RPS power supply select switch operation.	-030-00 was reviewed with all view, direction was provided on
Safe	ty Significance	
Ther no a Shut Tech Shut and	e is no safety significance associated with these actual initiating event. All safety actuations down Cooling System was restored within 10 minu- nical Specification requirements for restoring S down condition is 1 hour. Accordingly, this ever safety of the public or Plant personnel.	e RPS actuations since there was occurred as designed. The RHR tes following both events. The hutdown Cooling while in a Cold at posed no threat to the health
Simi	lar Events	
LER the the when func	87-20 - A failure of the same switch described overtravel of the switch in this event. In the L switch was physcially broken and did not prevent moved to an alternate power supply position ctioned as designed and there were no component fa	in LER 87-20 is different from ER 87-20 event, the stop tab on the switch from overtravelling I. In this event the switch ilures.

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AGILITY MAME (1) DOCKET NUMBER (2) LER NUMBER (6) PAGE (3) Washington Nuclear Plant - Unit 2 0 5 0 0 0 3 9 7 8 8 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 10 5 0 0 0 0 10 10 10 5 0 0 0 0 10 10 10 10 10 10 10	-83)	LICENSEE	EVENT REF	PORT (LER) T		UATIO	N	U.S.	APPROVEC EXPIRES: 8	EGULA O OMB N /31/88	10RY 0	-010	м
VEAR Internation Ext Reference Its Reference Its Reference System Component Reactor Protection System (RPS) JC Reactor Protection System (RPS) JC Reactor Protection System (RPS) JC Reactor Protection Assembly (EPA) JC BUB RPS Electrical Protection Assembly (EPA) JC BUB RPS Power Supply Select Switch JC JS Nuclear Steam Supply Shutoff System (NS4) BD Reactor Water Sample Valves AD V Miscellaneous Balance of Plant (BOP) BD	ACILITY NAME (1)			DOCKET NUMB	3ER (2)		LER NUM	8ER (6)			PAG	E (3)	
Bashington Nuclear Plant - Unit 2o 15 10 10 1 3 9 78 80 1 0 5 or 0EXI II more space in required, use additional MRC Form 3664/41031EIIS InformationText ReferenceEIIS ReferenceSystem ComponentReactor Protection System (RPS)JC		`	-			YEAR		ABEA	NUMBI	R			
EXT (# more space is required, use additional WRC Form 3064*9(117) EIIS Information Text Reference System Component Reactor Protection System (RPS) JC RPS Electrical Protection Assembly (EPA) JC BKR RPS-Bus-A and B JC BU RPS Power Supply Select Switch JC JS Nuclear Steam Supply Shutoff System (NS ⁴) BD Reactor Water Sample Valves AD V Miscellaneous Balance of Plant (BOP) BD RHR and Traversing Incore Probe (TIP) System IG Reactor Water Cleanup (RWCU) System BD Reactor Building Exhaust Plenum Process Radiation System IL Reactor Building Exhaust Plenum Process Radiation System BH Standby Gas Treatment System (SGT) BH Main Steamline Isolation and Drainline Valves (MSIVs) SB ISV/V	Bashington	Nuclear Plan	t – Unit 2	0 5 0	0 0 3 9 7	8 8	<u>_þ í</u>	3 <u> </u> 0	0	1 0	5 o	of (0
Text ReferenceEIIS ReferenceReactor Protection System (RPS)JCRPS Electrical Protection Assembly (EPA)JCBKRRPS-Bus-A and BJCBURPS Power Supply Select SwitchJCJSNuclear Steam Supply Shutoff System (NS4)BDReactor Water Sample ValvesADVMiscellaneous Balance of Plant (BOP)BDRHR and Traversing Incore Probe (TIP) SystemIGReactor Water Cleanup (RWCU) SystemCEReactor Building Exhaust Plenum Process Radiation SystemBHStandby Gas Treatment System (SGT)BHMain Steamline Isolation and Drainline Valves (MSIVs)SBISV/V	EXT (If more space is required EIIS Inf	d, use additional NRC Form 36 Ormation	843/(17)										
SystemComponentReactor Protection System (RPS)JCRPS Electrical Protection Assembly (EPA)JCBKRRPS-Bus-A and BJCBURPS Power Supply Select SwitchJCJSNuclear Steam Supply Shutoff System (NS4)BDReactor Water Sample ValvesADVMiscellaneous Balance of Plant (BOP)BDRHR and Traversing Incore Probe (TIP) SystemIGReactor Water Cleanup (RWCU) SystemCEReactor Building Exhaust Plenum Process Radiation SystemILStandby Gas Treatment System (SGT)BHControl Room Emergency Filtration SystemVAMain Steamline Isolation and Drainline Valves (MSIVs)SBISV/V	Tex	t Reference			-		<u>E</u> .		Refere	nce			
Reactor Protection System (RPS)JCRPS Electrical Protection Assembly (EPA)JCBKRRPS-Bus-A and BJCBURPS Power Supply Select SwitchJCJSNuclear Steam Supply Shutoff System (NS4)BDReactor Water Sample ValvesADVMiscellaneous Balance of Plant (BOP)BDRHR and Traversing Incore Probe (TIP) SystemIGRHR Shutdown Cooling SystemBDReactor Water Cleanup (RWCU) SystemCEReactor Building Exhaust Plenum Process Radiation SystemILStandby Gas Treatment System (SGT)BHControl Room Emergency Filtration SystemVAMain Steamline Isolation and Drainline Valves (MSIVs)SBISV/V								em	Component				
	Reactor RPS Elec RPS-Bus- RPS Powe Nuclear Reactor Miscella RHR and RHR Shut Reactor Standby Control Reactor Main Ste	Protection Sy trical Protec A and B r Supply Sele Steam Supply Water Sample neous Balance Traversing In down Cooling Water Cleanup Building Exha Gas Treatment Room Emergenc Building Vent amline Isolat	stem (RPS) tion Assem ct Switch Shutoff Sy Valves of Plant core Probe System (RWCU) Sy ust Plenum System (S y Filtrati ilation Sy ion and Dr	bly (EPA) stem (NS ⁴) (BOP) (TIP) System Stem GT) on System stem ainline Valu	em diation Sys ves (MSIVs)	tem	JC JC JC BD AD BD IG BD CE IL BH VH SB		B B J V -	KR US -	-		
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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352

Docket No. 50-397

April 26, 1989

Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2 LICENSEE EVENT REPORT NO. 88-030-01

Dear Sir:

Transmitted herewith is Licensee Event Report No. 88-030-01 for the WNP-2 Plant. This report is submitted in response to a commitment made by the Supply System in our response to Notice of Violation "B" in Inspection Report 88-24. The commitment is to change the August 26, 1988 event from not reportable to reportable per the requirements of 10CFR50.73. The report discusses the items of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,

Mil Powers

C.M. Powers (M/D 927M) WNP-2 Plant Manager

CMP:1c

Enclosure: Licensee Event Report No. 88-030-01

cc: Mr. John B. Martin, NRC - Region V Mr. C.J. Bosted, NRC Site (M/D 901A) INPO Records Center - Atlanta, GA Ms. Dottie Sherman, ANI Mr. D.L. Williams, BPA (M/D 399)

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