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 OXSEN, A.L. Washington Public Power Supply System

SUBJECT: Requests addl info re Special Rept LER 88-024 re temp in reactor containment in excess of Tech Spec limit of 150 f.

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	NRR/DEST/ICSB 7	1 1	NRR/DEST/MEB 9H	1 1
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NOV 18 1988

Docket No. 50-397

Washington Public Power Supply System
P. O. Box 968
Richland, Washington 99352

Attention: Mr. A. L. Oxsen
Assistant Managing Director for Operations

Gentlemen:

Subject: Special Report LER 88-024

This refers to the Special Report LER 88-024, forwarded to our office on August 2, 1988, which reported temperatures in the reactor containment in excess of the Technical Specification limit of 150F. Our review of this report has raised several questions about the event which we would like you to address in a supplemental report. These questions are as follows:

1. The "Immediate Corrective Action" section stated that "In an effort to increase cooling to the drywell, reactor closed cooling water system (RCC) valve lineup and flow paths were verified, non critical RCC loads were shed and a third RCC pump was started." This apparently did not correct the problem because the condition lasted for approximately 20 days. What happened after July 20, 1988 to reduce these temperatures?
2. Under the "Equipment" section you stated that acoustic monitors associated with relief valves 1B and 1C failed to operate properly due to degraded signal conditions. Were these failures, and the later failure (on September 7, 1988, which was after LER 88-24) of the monitor for relief valve 4D, related to the higher than expected temperatures? If there is a relationship between the higher temperatures and the failures, please explain the relationship.
3. In the "Environmental Qualification of Electrical Equipment" section, the report states that the main steam relief valve (MSRV) solenoid valves have a qualified life of 6.4 years at 150F, and that two years at 100F is equivalent to 0.2 years at 150F. These solenoids were said to have 3.2 years of life remaining at temperatures of 150F or less. At what temperatures do the solenoids operate, and what is the decrease in life expectancy for operation at the higher temperatures?
4. The same section of your report discussed the acoustic monitors and their associated electric cables. Both the monitors and the cables were evaluated at 150F, and the acoustic monitors were mounted directly on the MSRV tailpipes which have had observed temperatures as high as approximately 240F for extended periods of time. Your report does not document how these temperatures affect the acoustic monitors or the cable. Your report stated that "Evaluation of the qualified life of the acoustical

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monitors and associated cables indicates that adequate qualified life exists to reach the next scheduled refueling outage, even if the acoustic monitor cables experience 210F for the entire period of actual operation from initial fuel load (1983) until the next scheduled refueling." Your report did not address how the 240F tailpipe temperatures were related to the temperatures at the acoustic monitor or the associated cables. Please explain how the acoustic monitors and the associated cables are affected by the actual temperatures seen by this equipment. The justification statements in your report do not appear to support the conclusions reached.

- 5. Under the "Further Corrective Action" section, a statement was made that "Repeated periods when the area temperatures limit for containment will be exceeded may occur in the coming period of high makeup water temperature and high ambient air temperatures. Subsequent Special Reports will be issued at 30 day intervals, if required, in compliance with the Technical Specification requirements." The report does not contain any indication that corrective actions to reduce the recurrence of this condition are planned. Since the environmental conditions for the equipment in the drywell are expected to be less than 150F it is prudent to expect that either these conditions be satisfied or that additional design evaluation be undertaken to satisfy the license requirements.

Pursuant to 10 CFR 50.73.c, you are hereby requested to submit, within 30 days from the date of this letter, a supplement to this Special Report which provides the supplemental information requested above. If all requested information cannot be provided at that time, please indicate a date by which an additional supplemental report can be provided.

Should you have any questions concerning this request, we will be pleased to discuss them with you.

Sincerely,

Original signed

Dennis P. Kirsch, Director
Division of Reactor Safety
and Projects

bcc:
Resident Inspector
Project Inspector
docket file

B. Faulkenberry
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