REGUINFORMATION DISTRIBUTEN STEM (RIDS)

|   | ACCESSION NBR:<br>FACIL: 50-397 | 8706120233 DOC.DATE: 87/06/05 NOTARIZED: NO<br>WPPSS Nuclear Project, Unit 2, Washington Public Powe | DDCKET #<br>05000397 |
|---|---------------------------------|--|----------------------|
|   | AUTH. NAME                      | AUTHOR AFFILIATION   |                      |
|   | ARBUCKLE, J. D.                 |  |                      |
|   | POWERS, C. M.                   | Washington Public Power Supply System  |                      |
| Ì | RECIP, NAME                     | RECIPIENT AFFILIATION  |                      |

SUBJECT: LER 87-009-00: on 870506, control room emergency filtration sys automatically initiated due to failure to reset trip logic. Caused by oversight during procedure rev & revieu process. Procedures will be revised. W/870605 ltr.

NOTES:

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|           | RECIPIENT<br>ID CODE/NAME<br>PD5 LA<br>SAMWORTH,R | COPIE<br>LTTR<br>1<br>1 |        | RECIPIENT<br>ID CODE/NAME<br>PD5 PD | COP<br>LTTR<br>1 | IES<br>ENCL<br>1 |
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|           | NSIC HARRIS, J                                    | 1                       | 1      | MSIC MAYS, G                        | 1                | 1                |

TOTAL NUMBER OF COPIES REQUIRED: LTTR 42 ENCL 40

| NRC Form 306<br>(9-83)   |   |  |  |   | LEAR REGULATORY COMMISSI<br>ROVED OMB NO, 3150-0104  | ION      |  |  |  |  |
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| TITLE (4)  |   |  |  |   | <u></u>  | <u> </u> |  |  |  |  |
| Control Room Emergency Filtra  | tion System Actu  | ation Due  | to Incorn  | rect Proce  | dure   |          |  |  |  |  |
| EVENT DATE (5) LER NUMBER (6)  | REPORT DATE (   |  |  | FACILITIES INVOL  |  |          |  |  |  |  |
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| COMPLETE ONE   | E LINE FOR EACH COMPONENT P   | AILURE DESCRIBE  | D IN THIS REPOR  | 17 (13)   |  |          |  |  |  |  |
|  | PORTABLE<br>O NPRDS   | CAUSE SYSTEM   | COMPONENT  | MANUFAC-<br>TURER   | REPORTABLE<br>TO NPRDS   |          |  |  |  |  |
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| SUPPLEMENTA  | AL REPORT EXPECTED (14)   | · · · · · · · · · · · · · · · · · · ·  |  | EXPECTE   |  | FAR      |  |  |  |  |
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| ABSTRACT (Limit to 1400 paces, La. approximately fitteen single<br>On May 6, 1987 at 0625 h<br>automatically initiated.<br>trip logic prior to the<br>procedure. During the 1<br>Cabinet (RC-1 and RC-2)<br>alert Operations of a ha<br>the Engineered Safety Fu<br>provided indication that<br>annunciator cleared.<br>The cause of the event i<br>process. When procedure<br>and RC-2, the steps in t<br>removed based on the ass<br>annunciator cleared. How<br>recognize that the ESF f<br>annunciator clears.<br>There is no safety signi<br>initiating condition and<br>Ventilation System in an | ours, the Contro<br>The initiation<br>performance of a<br>ast outage, Divi<br>trip circuitry w<br>If-trip condition<br>nction (ESF) for<br>all half-trip I<br>s an oversight d<br>s were revised t<br>he procedures wh<br>umption that all<br>wever, personnel<br>or Control Room<br>ficance associat<br>all equipment o<br>isolation condi | was the n<br>Reactor N<br>sions 1 an<br>as modifie<br>n in these<br>Control P<br>ogic was n<br>uring the<br>ogic was n<br>uring the<br>trip cond<br>involved<br>Ventilation<br>ed with the<br>perated co | result of<br>lessel Low<br>ad 2 Balan<br>ed and and<br>panels.<br>Room Vent<br>reset when<br>reset when<br>rate the n<br>ind logic<br>ditions we<br>in the pro-<br>on does not<br>n the pro- | a failure<br>w Level su<br>nce of Pla<br>nunciators<br>With the<br>ilation, t<br>n the half<br>e revision<br>modification<br>reset state<br>ere reset<br>rocess fai<br>ot reset end<br>as there | to reset the<br>rveillance<br>int Relay<br>installed to<br>exception of<br>the modification<br>f-trip<br>and review<br>ion made to RC-1<br>itus were<br>once the<br>led to<br>even though the<br>was no actual |          |  |  |  |  |
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| NRC Form 306<br>(9.83)   |   |  |  |   |  |          |  |  |  |  |

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| NRC Form 366A |   | 0.5. |
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| (9-83)        | LICENSEE EVENT REPORT (LER) TEXT CONTINUATION |      |

NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

| FA | CIL | ITY | NAME | (1) |
|----|-----|-----|------|-----|

| FACILITY NAME (1)  |                          | DOCKET NUMBER (2)        |                        |                       |                          |                         |                         | Т                     | LER NUMBER (6) |                  |                 |                   |                   |                     |                 |            |             | PAGE (3) |    |    |   |   |
|--|--------------------------|--------------------------|------------------------|-----------------------|--------------------------|-------------------------|-------------------------|-----------------------|----------------|------------------|-----------------|-------------------|-------------------|---------------------|-----------------|------------|-------------|----------|----|----|---|---|
|  |                          |                          |                        |                       |                          |                         |                         |                       |                | EAR              |                 | 58                | AUE               | NTIA                | - 🛞             |            |             | )N<br>R  |    |    |   |   |
| Washington Nuclear Plant - Unit 2  | 0                        | 5                        | 0                      | ١٥                    | 0                        | 3                       | β                       | 7 ا                   | 8              | 7_               | ·               | p                 | 10                | 9 (                 | _               | _0         | ρ           | ¢        | f. | OF | 0 | 3 |
| TEXT (If more space is required, use additional NRC Form 308A's) (17)  |                          |                          |                        |                       |                          |                         |                         |                       |                |                  |                 |                   |                   |                     |                 |            |             |          |    |    |   |   |
| Plant Conditions   |                          |                          |                        |                       |                          |                         |                         |                       |                |                  |                 |                   |                   |                     |                 |            |             |          |    |    |   | , |
| a) Power Level - 0%<br>b) Plant Mode - 5 (Refueling)   |                          |                          |                        |                       |                          |                         |                         |                       |                |                  |                 |                   |                   |                     |                 |            |             |          |    |    |   |   |
| Event Description  |                          |                          |                        |                       |                          |                         |                         |                       |                |                  |                 |                   |                   |                     |                 |            |             |          |    |    |   |   |
| On May 6, 1987 at 0625 hours, a C<br>(WMA-FN-54B) automatically starte<br>failure to reset the trip logic p<br>7.4.3.2.1.2, "Division 1, Channel<br>CFT/CC." The procedure provides<br>calibration of Reactor Water Leve | ed.<br>prio<br>C,<br>ins | Th<br>ir t<br>Is<br>itri | ne<br>to<br>sol<br>uct | au<br>pe<br>lat<br>io | ito<br>erfo<br>cio<br>ns | na:<br>orr<br>n /<br>f( | tic<br>min<br>Act<br>or | : s<br>ig<br>iv<br>pe | ta<br>Pl<br>at | rt<br>ant<br>ion | was<br>Pi<br>Re | s t<br>roc<br>eac | the<br>ceo<br>cto | e re<br>iur<br>or l | esı<br>e<br>Lev | ult<br>vel | : ot<br>  2 | fa       | 1  |    |   |   |

During the last outage, Divisions 1 and 2 Balance of Plant Relay Cabinet (RC-1 and RC-2) trip circuitry was modified and annunciators were installed in the Control Room to alert Operations of a half-trip condition in these panels. With the exception of the Engineered Safety Function (ESF) for Control Room Ventilation, the modification provided indication that all half-trip logic was reset when the half-trip annunciator cleared.

The cause of the event has been determined to be an oversight during the procedure revision and review process. When procedures were revised to incorporate the modification made to RC-1 and RC-2, the steps in the procedures which verified logic reset status were removed based on the assumption that all trip conditions were reset once the annunciator cleared. However, Plant Instrument and Control (I&C) personnel who revised and reviewed the procedure failed to recognize that the ESF for Control Room Ventilation does not reset even though the annunciator clears. To clear the trip condition for Control Room Ventilation, personnel must reset WMA Reset Pushbuttons 3AX and 3AY.

When the I&C Technicians began the Reactor Vessel Low Level procedure, they verified that the RC-1 and RC-2 half-trip annunciators were clear and proceeded to perform the procedure. However, unknown to either the Operators or I&C Technicians, there was a half-trip condition in the ESF Control Room Ventilation circuit already in existence (investigation did not reveal the exact cause of the half-trip condition). As a result, when the I&C Technicians pressurized Reactor Water Level Switch MS-LS-61C through its trip setpoint, a full trip of the WMA Start Logic occurred and, by design, WMA-FN-54B automatically started.

## Immediate Corrective Action

After verification that no actual initiating condition existed, the Control Room Emergency Filtration System was returned to normal lineup.

| NRC Form 366A<br>(9-83) | LICENSEE EVENT REPORT  | (LER) TEXT CO        | NTINUAT           | ION                  |                               | ULATORY COMMISSIC<br>MB NO, 3150-0104<br>1/85 |  |  |  |  |
|-------------------------|--|----------------------|-------------------|----------------------|-------------------------------|---|--|--|--|--|
| ACILITY NAME (1)        |  | DOCKET NUMBER (2)    | YE                |                      | MBER (6)<br>JENTIAL MEVISION  | PAGE (3)                                      |  |  |  |  |
| Wash ington             | Nuclear Plant - Unit 2   | 0  5   0   0   0   3 | 19 17 8 1         |                      | 019 -0 0                      | ) 13 OF() 13                                  |  |  |  |  |
|                         | red, use additional NRC Form 305A's) (17)  | <u>I</u>             | t                 |                      |                               | •   |  |  |  |  |
| Furthe                  | r Corrective Action  | A.                   | •                 | ę                    |                               |   |  |  |  |  |
|                         | ppropriate Plant Procedures<br>ogic reset status.  | will be revis        | ed to in          | clude a              | step to ver                   | ify   |  |  |  |  |
| c                       | n engineering evaluation wil<br>ontact in each RC Cabinet to<br>MA Reset Pushbuttons have be | prevent a re         | to consist of the | sider in<br>1e annun | nstalling and<br>nciator unti | other<br>1 the                                |  |  |  |  |
| Safety                  | Significance   |                      |                   |                      |                               |   |  |  |  |  |
| actual                  | is no safety significance as<br>initiating condition and al<br>1 Room Ventilation System in  | 1 equipment of       | perated (         | correct              |                               | the   |  |  |  |  |
| Simila                  | <u>r Events</u>  |                      |                   |                      |                               |   |  |  |  |  |
| LERs 8                  | 5-027 and 85-036   |                      |                   |                      |                               |   |  |  |  |  |
| EIIS I                  | nformation   |                      |                   |                      |                               |   |  |  |  |  |
| <u><u> </u></u>         | ext Reference  |                      | <u> </u>          | EIIS Ret             | ference                       |   |  |  |  |  |
|                         |  | ,                    | System            | n                    | Component                     |   |  |  |  |  |
| C                       | ontrol Room Emergency Filtra   | tion System          | VH                |                      |                               |   |  |  |  |  |
| м                       | S-LS-61C   |                      | SB                |                      | LS                            |   |  |  |  |  |
|                         |  |                      | 3                 |                      | -                             |   |  |  |  |  |
| L                       |  |                      |                   |                      |                               |   |  |  |  |  |
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P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352

Docket No. 50-397

June 5, 1987

Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2 LICENSEE EVENT REPORT NO. 87-009

Dear Sir:

Transmitted herewith is Licensee Event Report No. 87-009 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,

Millions

C.M. Powers (M/D 927M) WNP-2 Plant Manager

CMP:1c

Enclosure: Licensee Event Report No. 87-009

cc: Mr. John B. Martin, NRC - Region V Mr. R. T. Dodds, NRC Site (M/D 901A) INPO Records Center - Atlanta, GA Ms. Dottie Sherman, ANI Mr. D. L. Williams, BPA (M/D 399)

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