



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

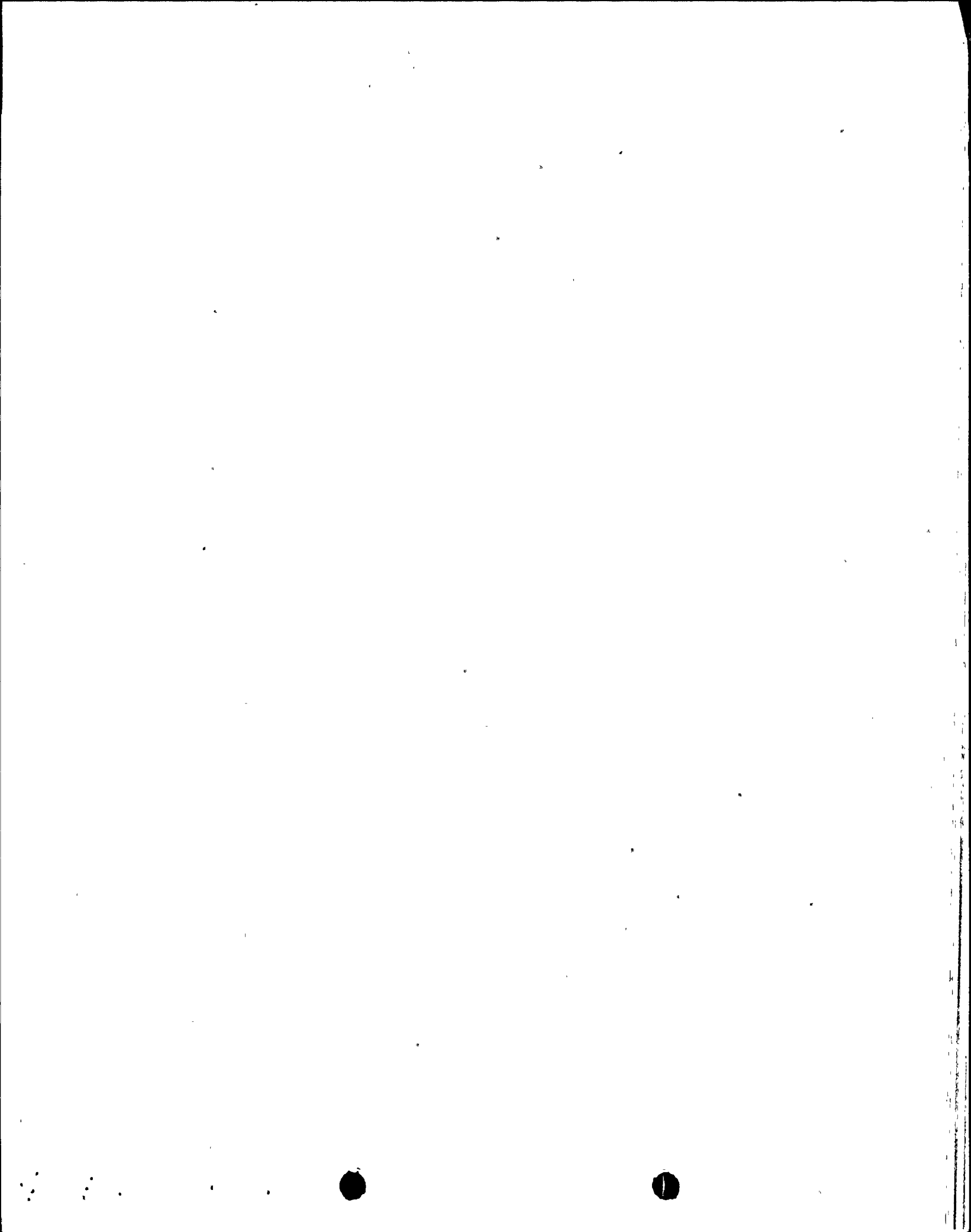
Amendment No. 27  
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Washington Public Power Supply System (the Supply System, also the licensee), dated January 17, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 27, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective as of the date of issuance.

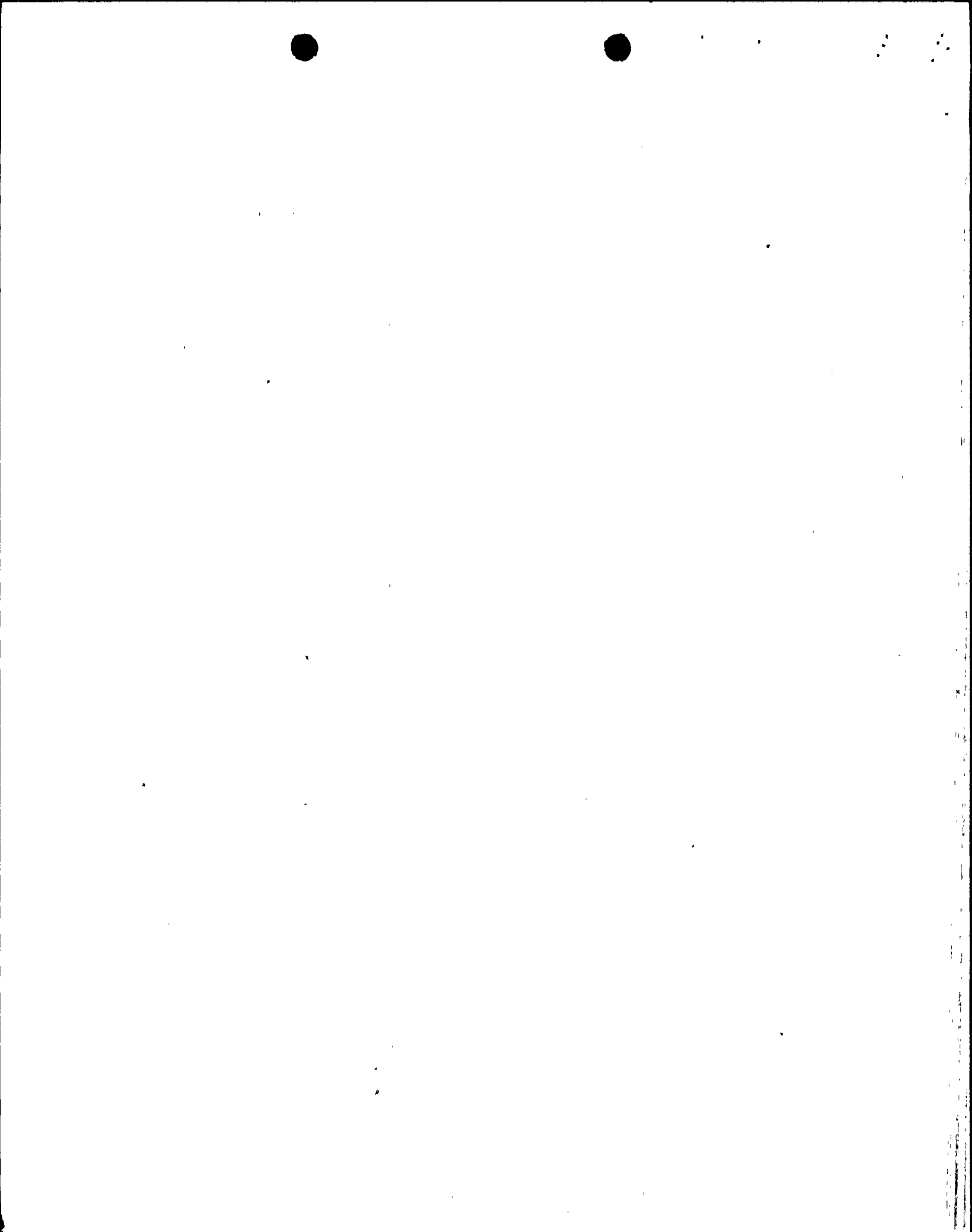
FOR THE NUCLEAR REGULATORY COMMISSION

*Elinor G. Adensam*

Elinor G. Adensam, Director  
BWR Project Directorate No. 3  
Division of BWR Licensing

Enclosure:  
Changes to the Technical  
Specifications

Date of Issuance: May 23, 1986



ENCLOSURE TO LICENSE AMENDMENT NO. 27

FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 8-26  
3/4 8-27  
3/4 8-28

INSERT

3/4 8-26  
3/4 8-27  
3/4 8-28

TABLE 3.8.4.3-1

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>	<u>SYSTEM(S) VALVE NUMBER</u>	<u>AFFECTED</u>
a. CAC-V-2 CAC-V-4 CAC-V-6 CAC-V-8 CAC-V-11 CAC-V-13 CAC-V-15 CAC-V-17	Containment Atmospheric Control System	g. MSLC-V-1A MSLC-V-1B MSLC-V-1C MSLC-V-1D MSLC-V-2A MSLC-V-2B MSLC-V-2C MSLC-V-2D MSLC-V-3A MSLC-V-3B MSLC-V-3C MSLC-V-3D MSLC-V-4 MSLC-V-5 MSLC-V-9 MSLC-V-10	Main Steam Isolation Valve Leakage Control System
b. CIA-V-20 CIA-V-30A CIA-V-30B	Containment Instrument Air System		
c. FPC-V-149 FPC-V-153 FPC-V-154 FPC-V-156 FPC-V-172 FPC-V-173 FPC-V-175 FPC-V-181A FPC-V-181B FPC-V-184	Fuel Pool Cooling System		
d. HPCS-V-1 HPCS-V-4 HPCS-V-10 HPCS-V-11 HPCS-V-12 HPCS-V-15 HPCS-V-23	High Pressure Core Spray System	h. RCC-V-5 RCC-V-21 RCC-V-40 RCC-V-104 RCC-V-129 RCC-V-130 RCC-V-131	Reactor Closed Cooling Water System
e. LPCS-V-1 LPCS-V-5 LPCS-FCV-11 LPCS-V-12	Low Pressure Core Spray System	i. RCIC-V-1 RCIC-V-8 RCIC-V-10 RCIC-V-13 RCIC-V-19 RCIC-V-22 RCIC-V-31	Reactor Core Isolation Cooling System
f. MS-V-1 MS-V-2 MS-V-5 MS-V-16 MS-V-19 MS-V-20 MS-V-67A MS-V-67B MS-V-67C MS-V-67D MS-V-146	Main Steam System		



TABLE 3.8.4.3-1 (Continued)

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>	<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>
i. RCIC-V-45	Reactor Core Isolation Cooling System	RHR-V-42C	
RCIC-V-46		RHR-V-47A	
RCIC-V-59		RHR-V-47B	
RCIC-V-63		RHR-V-48A	
RCIC-V-68		RHR-V-48B	
RCIC-V-69		RHR-V-49	
RCIC-V-76		RHR-V-53A	
RCIC-V-110		RHR-V-53B	
RCIC-V-113		RHR-V-64A	
		RHR-V-64C	
j. RFW-V-65A	Reactor Feedwater System	RHR-V-68A	
RFW-V-65B		RHR-V-68B	
		RHR-V-73A	
		RHR-V-73B	
		RHR-V-74A	
		RHR-V-74B	
		RHR-V-115	
		RHR-V-116	
		RHR-V-123A	
		RHR-V-123B	
	RHR-V-134A		
	RHR-V-134B		
k. RHR-V-3A	Residual Heat Removal System	1. RRC-V-16A	Reactor Recirculation System
RHR-V-3B		RRC-V-16B	
RHR-V-4A			
RHR-V-4B			
RHR-V-4C			
RHR-V-6A			
RHR-V-6B			
RHR-V-8			
RHR-V-9			
RHR-V-16A			
RHR-V-16B			
RHR-V-17A			
RHR-V-17B			
RHR-V-21			
RHR-V-23			
RHR-V-24A			
RHR-V-24B			
RHR-V-27A			
RHR-V-27B			
RHR-V-40			
RHR-V-42A			
RHR-V-42B			
		m. RWCU-V-1	Reactor Water Cleanup System
		RWCU-V-4	
		RWCU-V-40	



TABLE 3.8.4.3-1 (Continued)

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>	<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>
n. SGT-V-1A	Standby Gas Treatment System	o. AS-V-68A	Auxiliary Steam System
SGT-V-1B		AS-V-68B	
SGT-V-3A1		p. SW-V-2A	Standby Service Water System
SGT-V-3A2		SW-V-2B	
SGT-V-3B1		SW-V-4A	
SGT-V-3B2		SW-V-4B	
SGT-V-4A1		SW-V-4C	
SGT-V-4A2		SW-V-12A	
SGT-V-4B1		SW-V-12B	
SGT-V-4B2		SW-V-24A	
SGT-V-5A1		SW-V-24B	
SGT-V-5A2		SW-V-24C	
SGT-V-5B1		SW-V-29	
SGT-V-5B2	SW-V-44		
	SW-V-54		
	SW-V-75A		
	SW-V-75B		
	SW-V-90		
	SW-V-187A		
	SW-V-187B		
	SW-V-188A		
	SW-V-188B		