



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 24  
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for amendment filed by the Washington Public Power Supply System (the Supply System, also the licensee), dated January 17, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2.. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 24, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Elinor G. Adensam*

Elinor G. Adensam, Director  
BWR Project Directorate No. 3  
Division of BWR Licensing

Date of Issuance: May 13, 1986

ENCLOSURE TO LICENSE AMENDMENT NO. 24

FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 6-22  
3/4 6-25  
3/4 6-26  
3/4 8-26

INSERT

3/4 6-22  
3/4 6-25  
3/4 6-26  
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TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>VALVE GROUP(a)</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
a. <u>Automatic Isolation Valves (Continued)</u>		
Equipment Drain (Radioactive)	4	15
EDR-V-19		
EDR-V-20		
Floor Drain (Radioactive)	4	15
FDR-V-3		
FDR-V-4		
Fuel Pool Cooling/Suppression Pool Cleanup	4	35
FPC-V-149		
FPC-V-153(f)		
FPC-V-154(f)		
FPC-V-156		
Reactor Recirculation Hydraulic Control(e)	4	5
HY-V-17A,B		
HY-V-18A,B		
HY-V-19A,B		
HY-V-20A,B		
HY-V-33A,B		
HY-V-34A,B		
HY-V-35A,B		
HY-V-36A,B		
Traversing Incore Probe Valve	4	5
TIP-V-6,7,8,9,10 (Probe Line Ball Valves)		
TIP-V-11 (N <sub>2</sub> Gate Valve)		

WASHINGTON NUCLEAR - UNIT 2

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Amendment No. 24

TABLE 3.6.3-1 (Continued)  
PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>VALVE GROUP(a)</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
b. <u>Excess Flow Check Valves (e) (Continued)</u>		
Reactor Pressure Vessel		
N.A.		
PI-EFC-X12A, B, C		
PI-EFC-X18A, B, C, D		
PI-EFC-X37e, f		
PI-EFC-X38a, b, c, d, e, f		
PI-EFC-X39a, b, d, e		
PI-EFC-X40c, d		
PI-EFC-X41c, d		
PI-EFC-X42a, b		
PI-EFC-X44Aa, Ab, Ac, Ad, Ae, Af, Ag, Ah, Aj, Ak, Al, Am		
PI-EFC-X44Ba, Bb, Bc, Bd, Be, Bf, Bg, Bh, Bj, Bk, Bl, Bm		
PI-EFC-X61a, b		
PI-EFC-X62c, d		
PI-EFC-X66		
PI-EFC-X67		
PI-EFC-X69a, b, e		
PI-EFC-X70a, b, c, d, e, f		
PI-EFC-X71a, b, c, d, e, f		
PI-EFC-X72a		
PI-EFC-X73a		
PI-EFC-X74a, b, e, f		
PI-EFC-X75a, b, c, d, e, f		
PI-EFC-X78b, c, f		
PI-EFC-X79a, b		
PI-EFC-X82b		
PI-EFC-X84a		
PI-EFC-X106		
PI-EFC-X107		
PI-EFC-X108		



TABLE 3.6.3-1 (Continued)  
PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>VALVE GROUP(a)</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
b. <u>Excess Flow Check Valves (e) (Continued)</u>		
Reactor Pressure Vessel (Continued)		
PI-EFC-X109		N.A.
PI-EFC-X110		
PI-EFC-X111		
PI-EFC-X112		
PI-EFC-X113		
PI-EFC-X114		
PI-EFC-X115		
PI-EFC-X119		
Other		
PI-EFC-X40e,f		N.A.
PI-EFC-X41e,f		
PI-EFC-X86A,B		
PI-EFC-X87A,B		
c. <u>Manual Containment Isolation Valves</u>		
Demineralized Water		
DW-V-156		N.A.
DW-V-157		
Containment Air System		
CAS-VX-82e		N.A.
Service Air		
SA-V-109		N.A.





TABLE 3.8.4.3-1

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>	<u>SYSTEM(S) VALVE NUMBER</u>	<u>AFFECTED</u>
a. CAC-V-2 CAC-V-4 CAC-V-6 CAC-V-8 CAC-V-11 CAC-V-13 CAC-V-15 CAC-V-17	Containment Atmospheric Control System	g. MSLC-V-1A MSLC-V-1B MSLC-V-1C MSLC-V-1D MSLC-V-2A MSLC-V-2B MSLC-V-2C MSLC-V-2D MSLC-V-3A MSLC-V-3B MSLC-V-3C MSLC-V-3D MSLC-V-4 MSLC-V-5 MSLC-V-9 MSLC-V-10	Main Steam Isolation Valve Leakage Control System
b. CIA-V-20 CIA-V-30A CIA-V-30B	Containment Instrument Air System	h. RCC-V-5 RCC-V-6 RCC-V-17A RCC-V-17B RCC-V-21 RCC-V-40 RCC-V-71A RCC-V-71B RCC-V-71C RCC-V-72A RCC-V-72B RCC-V-104 RCC-V-129 RCC-V-130 RCC-V-131	Reactor Closed Cooling Water System
c. FPC-V-149 FPC-V-153 FPC-V-154 FPC-V-156	Fuel Pool Cooling System	i. RCIC-V-1 RCIC-V-8 RCIC-V-10 RCIC-V-13 RCIC-V-19 RCIC-V-22 RCIC-V-31	Reactor Core Isolation Cooling System
d. HPCS-V-1 HPCS-V-4 HPCS-V-10 HPCS-V-11 HPCS-V-12 HPCS-V-15 HPCS-V-23	High Pressure Core Spray System		
e. LPCS-V-1 LPCS-V-5 LPCS-FCV-11 LPCS-V-12	Low Pressure Core Spray System		
f. MS-V-1 MS-V-2 MS-V-5 MS-V-16 MS-V-19 MS-V-20 MS-V-67A MS-V-67B MS-V-67C MS-V-67D MS-V-146	Main Steam System		

