



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

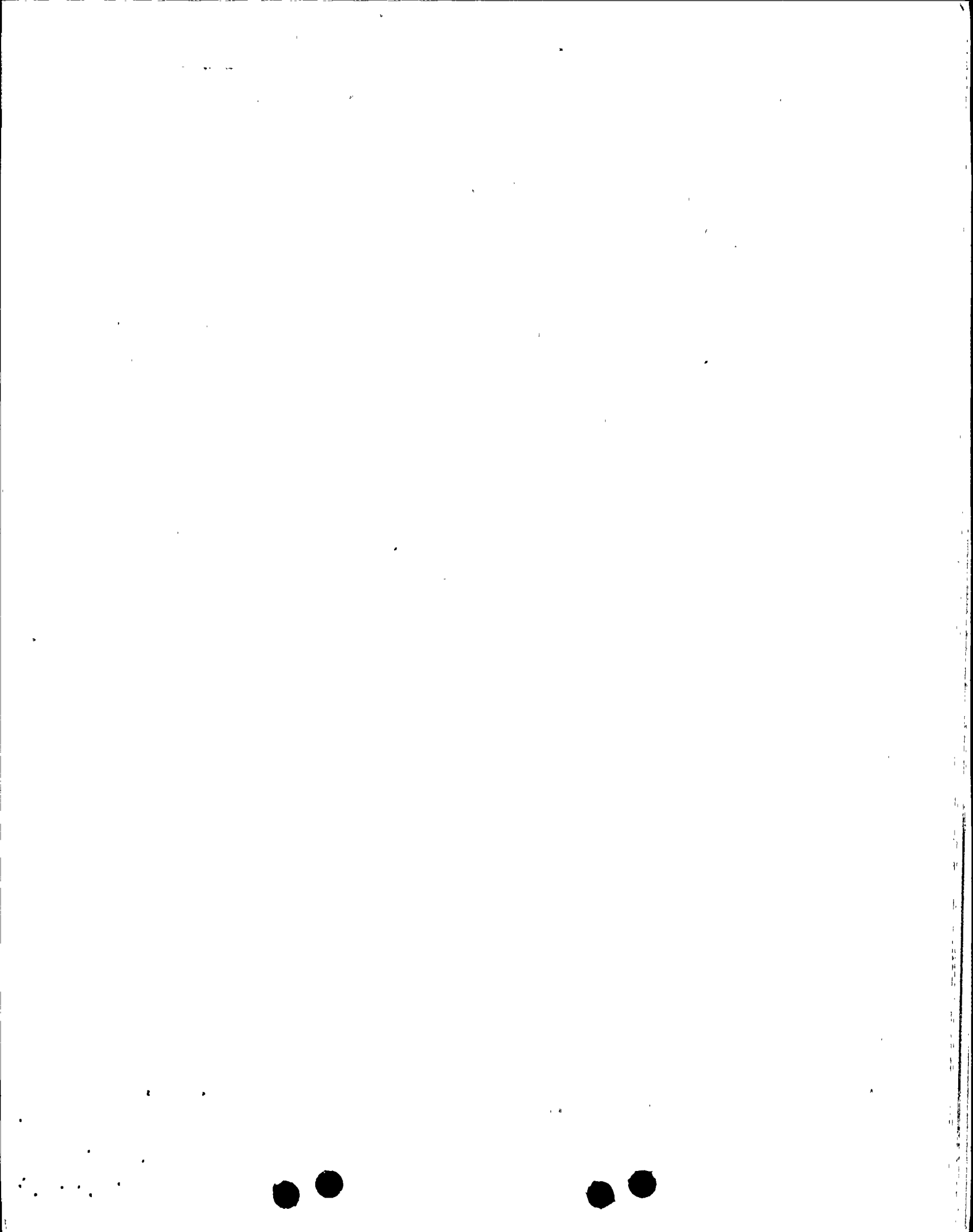
License No. NPF-21
Amendment No. 12

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Washington Public Power Supply System (the Supply System, also the licensee) dated March 13, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. NPF-21 is amended to revise the Technical Specifications as indicated in the attachments to this amendment and paragraph 2.C.(2) of Facility Operating License NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 12, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to Technical Specifications

Date of Issuance: JUN 25 1985



ATTACHMENT TO LICENSE AMENDMENT NO. 12
FACILITY OPERATING LICENSE NO. NPF-21
DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

REMOVE

3/4 3-13
3/4 3-14
3/4 3-16
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3/4 3-18
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INSERT

3/4 3-13
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6-23



TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>VALVE GROUPS OPERATED BY SIGNAL</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (a)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
3. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>				
a. Δ Flow - High	7	1	1, 2, 3	22
b. Heat Exchanger Area Temperature - High	7	1	1, 2, 3	22
c. Heat Exchanger Area Ventilation Δ Temp. - High	7	1	1, 2, 3	22
d. Pump Area Temperature - High				
1) Pump Room A	7	1	1, 2, 3	22
2) Pump Room B	7	1	1, 2, 3	22
e. Pump Area Ventilation Δ Temp. - High				
1) Pump Room A	7	1	1, 2, 3	22
2) Pump Room B	7	1	1, 2, 3	22
f. SLCS Initiation	7(f)	N.A.	1, 2, 3	22
g. Reactor Vessel Water Level - Low Low, Level 2	7	2	1, 2, 3	22
h. RWCU/RCIC Line Routing Area Temperature - High	7	1	1, 2, 3	22
i. RWCU Line Routing Area Temperature - High				
Room 509	7	1	1, 2, 3	22
Room 511	7	1	1, 2, 3	22
Room 408	7	1	1, 2, 3	22
Room 409	7	1	1, 2, 3	22
j. Manual Initiation	7	1/group	1, 2, 3	24

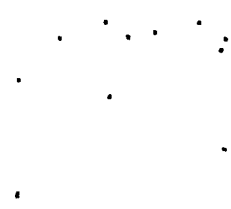


TABLE 3.3.2-1 (Continued)

		<u>ISOLATION ACTUATION INSTRUMENTATION</u>			
<u>TRIP FUNCTION</u>	<u>VALVE GROUPS OPERATED BY SIGNAL</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (a)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>	
4. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>					
a.	RCIC Steam Line Flow - High	8	1	1, 2, 3	22
b.	RCIC/RHR Steam Line Flow - High	8	1	1, 2, 3	22
c.	RCIC Steam Supply Pressure - Low	8, 9	2	1, 2, 3	22
d.	RCIC Turbine Exhaust Diaphragm Pressure - High	8	2	1, 2, 3	22
e.	RCIC Equipment Room Temperature - High	8	1	1, 2, 3	22
f.	RCIC Equipment Room Δ Temperature - High	8	1	1, 2, 3	22
g.	RWCU/RCIC Steam Line Routing Area Temperature - High	8	1	1, 2, 3	22
h.	Drywell Pressure - High	9	2	1, 2, 3	22
i.	Manual Initiation(h)	8	1	1, 2, 3	24
5. <u>RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>					
a.	Reactor Vessel Water Level - Low, Level 3	6	2	1, 2, 3	26
b.	Reactor Vessel (RHR Cut-in Permissive) Pressure - High	6	1	1, 2, 3	26
c.	Equipment Area Temperature - High	6	1	1, 2, 3	26
d.	Equipment Area Ventilation Δ Temp. - High	6	1	1, 2, 3	26
e.	Shutdown Cooling Suction Flow Rate - High	6	1	1, 2, 3	26
f.	RHR Heat Exchanger Area Temperature - High				
	Room 606	6	1	1, 2, 3	26
	Room 507	6	1	1, 2, 3	26
	Room 605	6	1	1, 2, 3	26
	Room 505	6	1	1, 2, 3	26
g.	Manual Initiation	6	1/group	1, 2, 3	24

WASHINGTON NUCLEAR - UNIT 2

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Amendment No. 12



TABLE 3.3.2-2
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>1. PRIMARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level		
1) Low, Level 3	> 13.0 inches*	> 11.0 inches
2) Low Low, Level 2	> -50 inches*	> -57 inches
b. Drywell Pressure - High	< 1.68 psig	< 1.88 psig
c. Main Steam Line		
1) Radiation - High	< 3.0 x full power background	< 3.6 x full power background
2) Pressure - Low	> 831 psig	> 811 psig
3) Flow - High	< 105.5 psid	< 108 psid
d. Main Steam Line Tunnel Temperature - High	< 150°F	< 170°F
e. Main Steam Line Tunnel Δ Temperature - High	< 80°F	< 90°F
f. Condenser Vacuum - Low	> 23 inches Hg absolute pressure	> 24.5 inches Hg absolute pressure
g. Manual Initiation	N.A.	N.A.
<u>2. SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Building Vent Exhaust Plenum Radiation - High	< 13.0 mR/h	< 16.0 mR/h
b. Drywell Pressure - High	< 1.68 psig	< 1.88 psig
c. Reactor Vessel Water Level - Low Low, Level 2	> -50 inches*	> -57 inches
d. Manual Initiation	N.A.	N.A.



TABLE 3.3.2-2 (Continued)
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
3. REACTOR WATER CLEANUP SYSTEM ISOLATION		
a. Δ Flow - High	≤ 58.5 gpm	≤ 65.5 gpm
b. Heat Exchanger Area Temperature - High	$\leq 150^\circ\text{F}$	$\leq 160^\circ\text{F}$
c. Heat Exchanger Area Ventilation Δ Temp. - High	$\leq 60^\circ\text{F}$	$\leq 70^\circ\text{F}$
d. Pump Area Temperature - High Pump Room A	$\leq 160^\circ\text{F}$	$\leq 180^\circ\text{F}$
Pump Room B	$\leq 160^\circ\text{F}$	$\leq 180^\circ\text{F}$
e. Pump Area Ventilation Δ Temp. - High Pump Room A	$\leq 70^\circ\text{F}$	$\leq 100^\circ\text{F}$
Pump Room B	$\leq 70^\circ\text{F}$	$\leq 100^\circ\text{F}$
f. SLCS Initiation	N.A.	N.A.
g. Reactor Vessel Water Level - Low Low, Level 2	≥ -50 inches*	≥ -57 inches
h. RWCU/RCIC Line Routing Area Temperature - High	$\leq 160^\circ\text{F}$	$\leq 180^\circ\text{F}$
i. RWCU Line Routing Area Temperature - High Room 409	$\leq 160^\circ\text{F}$	$\leq 175^\circ\text{F}$
Room 511	$\leq 160^\circ\text{F}$	$\leq 180^\circ\text{F}$
Room 408	$\leq 160^\circ\text{F}$	$\leq 180^\circ\text{F}$
Room 409	$\leq 160^\circ\text{F}$	$\leq 175^\circ\text{F}$
j. Manual Initiation	N.A.	N.A.
4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION		
a. RCIC Steam Line Flow - High	$\leq 290\%$ of rated flow	$\leq 300\%$ of rated flow
b. RHR/RCIC Steam Line Flow - High	≤ 101.5 inches H ₂ O	≤ 107.5 inches H ₂ O
c. RCIC Steam Supply Pressure - Low	≥ 62 psig	≥ 58 psig
d. RCIC Turbine Exhaust Diaphragm Pressure - High	≤ 10.0 psig	≤ 20.0 psig
e. RCIC Equipment Room Temperature - High	$\leq 160^\circ\text{F}$	$\leq 180^\circ\text{F}$



TABLE 3.3.2-2 (Continued)
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
4. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION (Continued)</u>		
f. RCIC Equipment Room Δ Temperature - High	$\leq 50^{\circ}\text{F}$	$\leq 60^{\circ}\text{F}$
g. RWCU/RCIC Steam Line Routing Area Temperature - High	$\leq 160^{\circ}\text{F}$	$\leq 180^{\circ}\text{F}$
h. Drywell Pressure - High	≤ 1.65 psig	≤ 1.85 psig
i. Manual Initiation	N.A.	N.A.
5. <u>RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Level 3	≥ 13.0 inches*	≥ 11.0 inches
b. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	≤ 125 psig	≤ 135 psig
c. Equipment Area Temperature - High Pump Room A	$\leq 140^{\circ}\text{F}$	$\leq 150^{\circ}\text{F}$
Pump Room B	$\leq 140^{\circ}\text{F}$	$\leq 150^{\circ}\text{F}$
d. Equipment Area Ventilation Δ Temp. - High Pump Room A	$\leq 55^{\circ}\text{F}$	$\leq 70^{\circ}\text{F}$
Pump Room B	$\leq 55^{\circ}\text{F}$	$\leq 70^{\circ}\text{F}$
e. Shutdown Cooling Return Flow Rate - High	≤ 174 inches H ₂ O	≤ 183 inches H ₂ O
f. RHR Heat Exchanger Area Temperature - High Room 606	$\leq 130^{\circ}\text{F}$	$\leq 140^{\circ}\text{F}$
Room 507	$\leq 150^{\circ}\text{F}$	$\leq 160^{\circ}\text{F}$
Room 605	$\leq 140^{\circ}\text{F}$	$\leq 150^{\circ}\text{F}$
Room 505	$\leq 130^{\circ}\text{F}$	$\leq 140^{\circ}\text{F}$
g. Manual Initiation	N.A.	N.A.

TABLE NOTATIONS

*See Bases Figure B 3/4 3-1.



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TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
<u>4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>	
a. RCIC Steam Line Flow - High	< 13(a)
b. RHR/RCIC Steam Line Flow - High	≤ 13(a)
c. RCIC Steam Supply Pressure - Low	≤ 13(a)
d. RCIC Turbine Exhaust Diaphragm Pressure - High	N.A.
e. RCIC Equipment Room Temperature - High	N.A.
f. RCIC Equipment Room Δ Temperature - High	N.A.
g. RWCU/RCIC Steam Line Routing Area Temperature - High	N.A.
h. Drywell Pressure - High	N.A.
i. Manual Initiation	N.A.
<u>5. RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>	
a. Reactor Vessel Water Level - Low, Level 3	≤ 13(a)
b. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	N.A.
c. Equipment Area Temperature - High	N.A.
d. Equipment Area Ventilation Δ Temp. - High	N.A.
e. Shutdown Cooling Return Flow Rate - High	N.A.
f. RHR Heat Exchanger Area Temperature - High	N.A.
g. Manual Initiation	N.A.

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

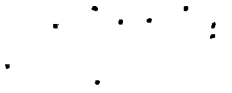
<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u>
4. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION (Continued)</u>				
g. RWCU/RCIC Steam Line Routing Area Temperature - High	S	M	R	1, 2, 3
h. Drywell Pressure - High	N.A.	M	R	1, 2, 3
i. Manual Initiation	N.A.	R	N.A.	1, 2, 3
5. <u>RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>				
a. Reactor Vessel Water Level - Low, Level 3	S	M	R	1, 2, 3
b. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	N.A.	M	R	1, 2, 3
c. Equipment Area Temperature - High	S	M	R	1, 2, 3
d. Equipment Area Ventilation Δ Temp. - High	S	M	R	1, 2, 3
e. Shutdown Cooling Return Flow Rate - High	N.A.	M	R	1, 2, 3
f. RHR Heat Exchanger Area Temperature - High	S	M	R	1, 2, 3
g. Manual Initiation	N.A.	R	N.A.	1, 2, 3

TABLE NOTATIONS

* When reactor steam pressure \geq 1037 psig and/or any turbine stop valve is open.

** When handling irradiated fuel in the secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.

During CORE ALTERATION and operations with a potential for draining the reactor vessel.



ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- e. Records of changes made to the procedures required by Specification 6.8.1.
- f. Records of radioactive shipments.
- g. Records of sealed source and fission detector leak tests and results.
- h. Records of annual physical inventory of all sealed source material of record.

6.10.3 The following records shall be retained for the duration of the unit Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report (FSAR).
- b. Records of new and irradiated fuel inventory, fuel transfers, and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient or operational cycles for those unit components identified in Table 5.7.1-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the unit staff.
- h. Records of inservice inspections performed pursuant to these Technical Specifications.
- i. Records of quality assurance activities required by the Operational Quality Assurance Manual not listed in Section 6.10.2.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the POC and the CNSRB.
- l. Records of the service lives of all hydraulic and mechanical snubbers required by Specification 3.7.4 including the date at which the service life commences and associated installation and maintenance records.
- m. Records of analysis required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.