UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-21
Amendment No. 11

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Washington Public Power Supply System (the Supply System, also the licensee) dated May 16, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-21 is amended to revise the Technical Specifications as indicated in the attachments to this amendment and paragraph 2.C.(2) of Facility Operating License NPF-21 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 11, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to Technical Specifications

Date of Issuance: JUN 25 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 11
FACILITY OPERATING LICENSE NO. NPF-21
DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

REMOVE	INSERT
3/4 3-26	3/4 3-26
3/4 3-27	3/4 3-27
3/4 3-30	3/4 3-30
3/4 3-31	3/4 3-31
3/4 3-34	3/4 3-34
3/4 3-35	3/4 3-35

TABLE 3.3.3-1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

TRIP FUNCTION	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM(a)	APPLICABLE OPERATIONAL CONDITIONS	ACTION
A. DIVISION I TRIP SYSTEM			
1. RHR-A (LPCI MODE) & LPCS SYSTEM			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	1, 2, 3, 4*, 5*	30
b. Drywell Pressure - High	2	1, 2, 3	30
c. LPCS Pump Discharge Flow-Low (Minimum Flow)	1	1, 2, 3, 4*, 5*	31
d. Reactor Vessel Pressure-Low (LPCS Permissive)	1	1, 2, 3, 4*, 5*	32 33
e. Reactor Vessel Pressure-Low (LPCI Permissive)	1	1, 2, 3, 4*, 5*	32 33
f. LPCI Pump A Start Time Delay Relay	1	1, 2, 3, 4*, 5*	32
g. LPCI Pump A Discharge Flow-Low (Minimum Flow)	1	1, 2, 3, 4*, 5*	31
h. Manual Initiation	1/division	1, 2, 3, 4*, 5*	34
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "A"#			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	1, 2, 3	30
b. ADS Timer	1	1, 2, 3	32
c. Reactor Vessel Water Level - Low, Level 3 (Permissive)	1	1, 2, 3	32
d. LPCS Pump Discharge Pressure-High (Pump Running)	2	1, 2, 3	32
e. LPCI Pump A Discharge Pressure-High (Pump Running)	2	1, 2, 3	32
f. Manual Initiation	2/division	1, 2, 3	35
g. Inhibit Switch	1/division	1, 2, 3	35

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

TRIP FUNCTION	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM(a)	APPLICABLE OPERATIONAL CONDITIONS	ACTION
B. DIVISION 2 TRIP SYSTEM			
1. RHR B and C (LPCI MODE)			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	1, 2, 3, 4*, 5*	30
b. Drywell Pressure - High	2	1, 2, 3	30
c. Reactor Vessel Pressure-Low (LPCI Permissive)	1/valve	1, 2, 3,	32
		4*, 5*	33
d. LPCI Pump B Start Time Delay Relay	1	1, 2, 3, 4*, 5*	32
e. LPCI Pump Discharge Flow-Low (Minimum Flow)	1/pump	1, 2, 3, 4*, 5*	31
f. Manual Initiation	1/division	1, 2, 3, 4*, 5*	34
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B"#			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	1, 2, 3	30
b. ADS Timer	1	1, 2, 3	32
c. Reactor Vessel Water Level - Low, Level 3 (Permissive)	1	1, 2, 3	32
d. LPCI Pump B and C Discharge Pressure - High (Pump Running)	2/pump	1, 2, 3	32
e. Manual Initiation	2/division	1, 2, 3	35
f. Inhibit Switch	1/division	1, 2, 3	35

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
A. DIVISION 1 TRIP SYSTEM		
1. RHR-A (LPCI MODE) AND LPCS SYSTEM		
a. Reactor Vessel Water Level - Low Low Low, Level 1	≥ -129 inches*	≥ -136 inches
b. Drywell Pressure - High	≤ 1.65 psig	≤ 1.85 psig
c. LPCS Pump Discharge Flow-Low (Minimum Flow)	≥ 770 gpm	≤ 900 gpm
d. Reactor Vessel Pressure-Low (LPCS Permissive)	≥ 470 psig, decreasing	≥ 450 psig, decreasing
e. Reactor Vessel Pressure-Low (LPCI Permissive)	≥ 470 psig, decreasing	≥ 450 psig, decreasing
f. LPCI Pump A Start Time Delay Relay	≤ 5 seconds	≤ 6 seconds
g. LPCI Pump A Discharge Flow-Low (Minimum Flow)	≥ 800 gpm	≥ 650 gpm
h. Manual Initiation	N.A.	N.A.
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "A"		
a. Reactor Vessel Water Level - Low Low Low, Level 1	≥ -129 inches*	≥ -136 inches
b. ADS Timer	≤ 105 seconds	≤ 117 seconds
c. Reactor Vessel Water Level-Low, Level 3 (Permissive)	≥ 13.0 inches*	≥ 11 inches
d. LPCS Pump Discharge Pressure-High (Pump Running)	≥ 145 psig, increasing	≥ 125 psig, increasing
e. LPCI Pump A Discharge Pressure-High (Pump Running)	≥ 125 psig, increasing	≥ 115 psig, increasing
f. Manual Initiation	N.A.	N.A.
g. Inhibit Switch	N.A.	N.A.

TABLE 3.3.3-2 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
B. DIVISION 2 TRIP SYSTEM		
1. RHR B AND C (LPCI MODE)		
a. Reactor Vessel Water Level - Low Low Low, Level 1	≥ -129 inches*	≥ -136 inches
b. Drywell Pressure - High	≤ 1.65 psig	≤ 1.85 psig
c. Reactor Vessel Pressure-Low (LPCI Permissive)	≥ 470 psig, decreasing	≥ 450 psig, decreasing
d. LPCI Pump B Start Time Delay Relay	≤ 5 seconds	≤ 6 seconds
e. LPCI Pump Discharge Flow-Low (Minimum Flow)	≥ 800 gpm	≥ 650 gpm
f. Manual Initiation	N.A.	N.A.
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B"		
a. Reactor Vessel Water Level - Low Low Low, Level 1	≥ -129 inches*	≥ -136 inches
b. ADS Timer	≤ 105 seconds	≤ 117 seconds
c. Reactor Vessel Water Level-Low, Level 3 (Permissive)	≥ 13.0 inches*	≥ 11 inches
d. LPCI Pump B and C Discharge Pressure-High (Pump Running)	≥ 125 psig, increasing	≥ 115 psig, increasing
e. Manual Initiation	N.A.	N.A.
f. Inhibit Switch	N.A.	N.A.

TABLE 4.3.3.1-1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
A. DIVISION I TRIP SYSTEM				
1. RHR-A (LPCI MODE) AND LPCS SYSTEM				
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	R	1, 2, 3, 4*, 5*
b. Drywell Pressure - High	N.A.	M	R	1, 2, 3
c. LPCS Pump Discharge Flow-Low (Minimum Flow)	N.A.	M	R	1, 2, 3, 4*, 5*
d. Reactor Vessel Pressure-Low (LPCS Permissive)	N.A.	M	R	1, 2, 3, 4*, 5*
e. Reactor Vessel Pressure-Low (LPCI Permissive)	N.A.	M	R	1, 2, 3, 4*, 5*
f. LPCI Pump A Start Time Delay Relay	N.A.	M	Q	1, 2, 3, 4*, 5*
g. LPCI Pump A Flow-Low (Minimum Flow)	N.A.	M	R	1, 2, 3, 4*, 5*
h. Manual Initiation	N.A.	R	N.A.	1, 2, 3, 4*, 5*
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "A"#				
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	R	1, 2, 3
b. ADS Timer	N.A.	M	Q	1, 2, 3
c. Reactor Vessel Water Level - Low, Level 3 (Permissive)	S	M	R	1, 2, 3
d. LPCS Pump Discharge Pressure-High (Pump Running)	N.A.	M	R	1, 2, 3
e. LPCI Pump A Discharge Pressure-High (Pump Running)	N.A.	M	R	1, 2, 3
f. Manual Initiation	N.A.	R	N.A.	1, 2, 3
g. Inhibit Switch	N.A.	M	N.A.	1, 2, 3

TABLE 4.3.3.1-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
B. DIVISION 2 TRIP SYSTEM				
1. RHR B AND C (LPCI MODE)				
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	R	1, 2, 3, 4*, 5*
b. Drywell Pressure - High	N.A.	M	R	1, 2, 3
c. Reactor Vessel Pressure-Low (LPCI Permissive)	N.A.	M	R	1, 2, 3, 4*, 5*
d. LPCI Pump B Start Time Delay Relay	N.A.	M	Q	1, 2, 3, 4*, 5*
e. LPCI Pump Discharge Flow-Low (Minimum Flow)	N.A.	M	R	1, 2, 3, 4*, 5*
f. Manual Initiation	N.A.	R	N.A.	1, 2, 3, 4*, 5*
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B"#				
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	R	1, 2, 3
b. ADS Timer	N.A.	M	Q	1, 2, 3
c. Reactor Vessel Water Level - Low, Level 3 (Permissive)	S	M	R	1, 2, 3
d. LPCI Pump B and C Discharge Pressure-High (Pump Running)	N.A.	M	R	1, 2, 3
e. Manual Initiation	N.A.	R	N.A.	1, 2, 3
f. Inhibit Switch	N.A.	N	N.A.	1, 2, 3