UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION

AMENDMENT NO.6 TO NPF-21

WPPSS NUCLEAR PROJECT NO.2

DOCKET NO. 50-397

Introduction

By letters dated March 10 and March 13, 1984 and as supplemented by letters dated April 25 and May 4, 1984, the licensee requested the following changes to the WNP-2 Technical Specifications.

(1) Technical Specification 3.3.7.9, Fire Detection Instrumentation: In table 3.3.7.9-1, under Reactor Building Elev. 606'-10.5", align the number 6/0 under column UD, move instruments in Control room (PGCC) from column SD to TD and add two thermal detectors. Also in Technical Specification 3.7.6.2.a., add item 4 to indicate where the pre-action and deluge spray and sprinkler system will also be operable.

(2) Technical Specification 3/4.5.1., Emergency Core Cooling System - Operating: For surveillance requirements for automatic depressurization system, change initiations setpoints in item 3(c) to greater than or equal to 140 psig from 140 $\pm$ 3 psig in decreasing pressure and alarm setpoint to greater than or equal to 135 psig from 135 $\pm$ 3 psig on decreasing pressure.

(3) Technical Specification 3/4.6.6. Primary Containment Atmosphere Control - Drywell and suppression chamber Hydrogen Recombiner System: In paragraph 4.6.6.1 item a., line 2, add the word "heater" after "minimum recombiner."

(4) Technical Specification 3/4.8.4. Electrical Equipment Protective Devices - A.C. Circuits Inside Primary Containment: In paragraph 3.8.4.1, item a., change breaker 2BR to 8AR. Add item d. stating that "Circuits supplied by breakers in cubicles 2BL, 1D, and 2CR of MC-3DA." Revise Table 3.8.4.2-1 to include the new fuse sizes on some of the primary circuits and delete entries for MT-HOI-18 and MT-HOI-19c from the table.

(5) Technical Specification 3.8.4.3, Motor Operated Valves Thermal Overload Protection: From Table 3.8.4.3-1, delete valves RCIC-V-12, RC1C-V-64, RHR-V-11A, RHR-V-11B, RHR-V-12A, RHR-V-12B, RHR-V-26A, RHR-V-26B, RHR-V-52A, RHR-V-52B, RHR-V-87A, RHR-V-87B, RHR-V-124A, RHR-V-124B, RHR-V-125A, RHR-V-125B, RRC-V-67A, and RRC-V-67B.

(6) Technical Specification 6.0, Administrative Controls: In Figure 6.2.1-1, change "Power Generations Director to Assistant Managing Director for Operations, Technology Director to Engineering Director and move Support Services Director Functions under Assistant Managing Director for Operations." In Figure 6.2.2-1a, change "Technical Training Manager to

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Technical Training Coordinator." Under Assistant Plant Manager, add Planning and Scheduling Supervisor, and under Technical Manager, add Plant Engineering Supervisor I & II. Also, change Health Physicist to Health Physics/Chemistry Support Supervisor, and under Maintenance Manager, delete, Spare Parts Material Supervisor and add Procurement Manager.

On page 6-8, paragraph 6.4, line 2, change wording "Training Manager to Technical Training Coordinator." Also in paragraph 6.5.1.2, change "Plant QA/QC Manager to Plant QA Manager." On page 6-9, item e & k, change designation "Director of Power Generation to Assistant Managing Director for Operations." Change item f to read "Review of all REPORTABLE EVENTS;". In item i, delete the last word "and" in item k, line 4 change "Director of Power Generation to Assistant Managing Director for Operations" and add word "and" at the end. On page 6-10, at the end of paragraph b., add "as defined in 10 CFR 50.59." In paragraph c. and section 6.5.1.8, change the title "Director of Power Generation to Assistant Managing Director for Operations." On page 6-11, Section 6.5.2.2, change lines 3 to 6 to read as, "Supply System. He shall designate from members, a Chairman and Alternate Chairman, and an Executive Secretary. The plant organization and the Directorates of Engineering Support Services, and Licensing and Assurance shall be represented. The qualifications of all members shall meet the". Section 6.6 should be reworded to "REPORTABLE EVENT ACTION". In section 6.6.1, Item b. and Section 6.7.1, Item a. and c. change "Director of Power Generation to Assistant Managing Director for Operations." Also, in a. & c. delete words "within 24 hours" and "within 14 days of the violation" respectively. Change the first sentence of paragraph 2 of Section 6.8.2 to read "In addition, the review and approval of the implementing procedures supporting item k in Specification 6.8.1 will be coordinated by the Director of Support Services who will provide review and approval control." On page 6-16, delete the note at the bottom of the page.

(7) Paragraph 4.8.2.1.d.2 of Technical Specification 3/4.8.2 is changed to address the following. The battery capacity is adequate to supply D.C. power to the Class I load as required for safe shutdown while maintaining the battery terminal voltage greater than or equal to 21 volts for the $\pm$ 24 volt battery, 105 volts for the 125-volt battery and 210 volts for 250-volts battery.

(8) The time during which the battery is capable of supplying and maintaining the actual emergency loads in operable status is changed from 8 hours to 2 hours. Paragraph 4.8.2.1.d.1 is changed to read as follows: "The battery capacity is adequate to supply and maintain in OPERABLE status all of the actual emergency loads for 2 hours for Divisions 1, 2 and 3 when the battery is subjected to a battery service test, or," Note (a) in Table 4.8.2.1-1 is changed to clarify the level correction as follows: "Corrected for electrolyte temperature. Level correction will be used when electrolyte level is outside the normal range."

Evaluation

The proposed changes to the tables, which identify word processing errors and addition of two thermal detections in the control room, have no safety significance other than to permit the Technical Specifications to accurately reflect conditions as they now exist in the plant. The addition of pre-actions and deluge spray and sprinkler system in the control room area (elevation 501') of the radwaste building also has no safety significance except that it makes the requirements more stringent. Thus the proposed changes are acceptable.

The changes proposed in item (2) above, eliminates the upper bound on both the initiation setpoint and the low pressure alarm setpoint for the ADS accumulator backup compressed gas system. This is conservative since a higher setpoint results in earlier activation of the system and/or alarm. The proposed technical specification change is, therefore, acceptable.

The licensee has indicated that surveillance requirement 4-6.6.1.a which requires monitoring the recombiner outlet temperature during the warmup test is incorrect. The surveillance requirement should require monitoring the recombiner heater outlet temperature. As indicated by the licensee, the inlet temperature to the recombiner catalyst bed must be approximately 500° to prevent degradation from halogens. The recombiner consists of electric heaters directly above the catalyst bed, both of which are contained in the pressure vessel. Hence, heater outlet temperature and catalyst bed inlet temperature are one and the same. The staff, therefore, finds that proposed change to specification 4-6.6.1.a. acceptable.

In item 4 above, the licensee proposed to add fuses to provide primary and backup protection for electrical penetration assemblies. Some primary protection fuse sizes have been changed to better coordinate with the backup fuses. These changes provide better protection to the penetration assemblies and better coordination to the penetration assembly protective devices and, therefore, are acceptable. In addition, Table 3.8.4.2-1 has also been updated to reflect these changes.

In item 5 above, the licensee proposed to change Technical Specification Table 3.8.4.3-1 to delete the valves associated with the Steam Condensing Mode deactivation. NRC staff agrees that no credit was taken in the WNP-2 FSAR Chapter 15 Accident Analysis for this mode of operation as discussed in a letter GO2-83-834, G. C. Sorensen (SS) to A. Schwencer (NRC), "Deletion of Residual Heat Removal Steam Condensing Mode," dated September 15, 1983. Amendment 33 to the WNP-2 FSAR incorporated the deletion of this mode of operation. Additionally, valves RRC-V-67A and 67B are deleted from Table 3.8.4.3-1. These valves are non-Class 1E valves which were listed in error. Also deleted is RCIC-V-12 as it is a manually operated valve with no motor operation. Based on the above evaluation, we find no safety consequences result from these proposed technical specification changes and they are, therefore, acceptable.

In Item 6 above, the licensee proposed several corporate and plant staff changes. In the corporate area, the title of Power Generations Director has been changed to Assistant Managing Director for Operation. The title of Technology Director has been changed to Engineering Director. The organizational unit under the Support Services Director, which reported to the Office of the Managing Director, now reports to the Assistant Managing Director for Operations. There has been no deletion of support for WNP-2 or change of personnel. The staff finds these changes acceptable as they meet the acceptance criteria of Standard Review Plan Section 13.1.1 (NUREG-0800).

The title of Health Physicist has been changed to Health Physics/Chemistry Support Supervisor and the title Technical Training Manager has been changed to Technical Training Coordinator. The position of Planning and Scheduling Supervisor who reports to the Plant Manager has been added. The position of Spare Parts Material Supervisor has been deleted with the functions now included in the new position of Procurement Manager. The Shift Technical Advisors are now shown as part of the Reactor Engineering Staff under the Technical Manager, and two Plant Engineering Supervisors have been added to the staff of the Technical Manager. In addition, there were several minor administrative changes. There has been no deletion of support to WNP-2. The staff finds these changes acceptable as they meet the acceptance criteria of Standard Review Plans Section 13.1.2 (NUREG-0800).

In item (7) & (8), changes are made in Technical Specification 4.8.2.1.d.1 and 4-8.2.1.d.2 to correct the time during which the battery is capable of of supplying and maintaining the actual emergency loads in operable status with the minimum voltage of 105 volts for 125-volt battery, 210 volts and 250-volt battery, and 21 volts for $\pm$ 24-volt battery. WNP-2 design requires D.C. power to emergency loads for 2 hours without A.C. source. The discharge duration change from 8 hours to 2 hours does not change the D.C. battery capacity and, therefore, is acceptable. The battery capacity dummy load profile is expanded for clarification of the battery capacity testing as required. Additionally a change related to electrolyte level which provides clarification has been proposed. We find no safety consequences result from these proposed changes to the technical specification and are therefore, acceptable.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (49 FR 25379) on June 20, 1984. No public comments were received.

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical in the common defense and security or to the health and safety of the public.

Dated: OCT 12 1984

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

October 4, 1984

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DOCKET No. 50-397

MEMORANDUM FOR: Docketing and Service Branch
Office of the Secretary of the Commission

FROM: Office of Nuclear Reactor Regulation

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

One signed original of the *Federal Register* Notice identified below is enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (6 Ø) of the Notice are enclosed for your use.

☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).

☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.

☐ Notice of Consideration of Issuance of Amendment to Facility Operating License.

☐ Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.

☐ Notice of Availability of NRC Draft/Final Environmental Statement.

☐ Notice of Limited Work Authorization.

☐ Notice of Availability of Safety Evaluation Report.

☐ Notice of Issuance of Construction Permit(s).

☐ Notice of Issuance of Facility Operating License(s) or Amendment(s).

☐ Order.

☐ Exemption.

☐ Notice of Granting of Relief.

☒ Other: Notice for WNP-2

Office of Nuclear Reactor Regulation

Enclosure:
As stated

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OFFICE	DL:LB#2						
SURNAME	EGHylton						
DATE	10/4/84						

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U. S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-21, issued to Washington Public Power Supply System (the licensee), for operation of the WNP-2 located in Richland, Washington.

The amendment would modify the license condition 2.C.(11) of Facility Operating License NPF-21 which states that, " The licensee shall complete construction of deferred shield walls as identified in Attachment 3 to this license or prior to operation of the permanent solid radioactive waste solidification system, whichever occurs first." The amended license condition will read as follows: "The Licensee shall complete construction of the deferred shield walls and window as identified in Attachment 3, as amended by this license amendment and as dictated by ongoing ALARA reviews identifying the need for additional shielding." The request is in accordance with the licensee's application for amendment dated August 15, 1984, and as supplemented on September 7, 1984.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The basis for this proposed determination is that the original design anticipated radiation sources in the solid radwaste system and condenser tube access areas requiring shielding for protection of personnel in adjacent areas. Since, at present, there is no schedule for installation of the solid radwaste solidification system, the source originally anticipated is non-existent. Without a source, there is no requirement for construction of the subject shield walls in this area for personnel protection. Additionally, ALARA reviews of the west condenser tube access area indicates that the radiation levels for this area are significantly below the original levels for which the shield wall was designed. As a result, the radiation levels in this area do not represent a hazard to personnel at this time and thus there is no need to provide further shielding. WNP-2 will continue to perform

ALARA reviews to evaluate the need for the subject shield walls and window, and will construct this additional shielding as dictated by the findings of these ALARA reviews.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Comments should be addressed to the Secretary of the Commission, U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attn: Docketing and Service Branch.

By , the licensee may file a report for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Request for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the

Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR §2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases

for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to A. Schwencer: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Executive Legal Director, U. S. Nuclear Regulatory

Commission, Washington, D.C. 20555, and to Nicholas Reynolds Esquire, Bishop, Cook, Liberman, Purcell and Reynolds, 1200 Seventeenth Street, N.W., Washington, D.C. 20036, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent as a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board Designated to rule on the petition and/or request, that the petitioner has made a substantial showing of good cause for the granting of a late petition and/or request. That determination will be based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment which is available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Richland City Library, Swift and Northgate Streets, Richland, Washington.

Dated at Bethesda, Maryland this 4th day of October 1984.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

LICENSE AUTHORITY FILE COPY

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SEP 21 1984

DO NOT REMOVE

Posted
Amdt. 5
to NPF-21

Docket No. 50-397

Mr. G. C. Sorensen, Manager
Regulatory Programs
Washington Public Power Supply System
P. O. Box 968
3000 George Washington Way
Richland, Washington 99352

Dear Mr. Sorensen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 5 TO FACILITY OPERATING LICENSE NPF-21, WPPSS NUCLEAR PROJECT NO. 2

The U. S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 5 to Facility Operating License NPF-21 to the Washington Public Power Supply System for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington. This amendment is in response to your letter dated January 20, 1984.

This amendment incorporates administrative changes which achieve agreement between the WNP-2 Technical Specifications and new regulations which became effective on January 1, 1984. Specifically, in Title 10 of the Code of Federal Regulations, Section 50.72 has been revised and a new Section 50.73 has been added. Section 50.72 revises the immediate notification requirements for operating nuclear power reactors. The new Section 50.73 provides for a revised Licensee Event Report System.

A copy of the related safety evaluation supporting Amendment No. 5 to Facility Operating License No. NPF-21 is enclosed.

Sincerely,

A. Schwencer

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 5 to Facility Operating License NPF-21
2. Safety Evaluation

cc w/enclosures:
See next page

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WNP-2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-21
Amendment No. 5

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Washington Public Power Supply System (WPPSS, also the licensee) dated January 20,1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-21 is hereby amended as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 5, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to the Technical Specifications

Date of Issuance: SEP 21 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 5
FACILITY OPERATING LICENSE NO. NPF-21
DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

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PURGE - PURGING

1.33 PURGE or PURGING shall be the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

RATED THERMAL POWER

1.34 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 3323 MWt.

REACTOR PROTECTION SYSTEM RESPONSE TIME

1.35 REACTOR PROTECTION SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its trip setpoint at the channel sensor until deenergization of the scram pilot valve solenoids. The response time may be measured by any series of sequential, overlapping, or total steps such that the entire response time is measured.

REPORTABLE EVENT

1.36 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

ROD DENSITY

1.37 ROD DENSITY shall be the number of control rod notches inserted as a fraction of the total number of control rod notches. All rods fully inserted is equivalent to 100% ROD DENSITY.

SECONDARY CONTAINMENT INTEGRITY

1.38 SECONDARY CONTAINMENT INTEGRITY shall exist when:

a. All secondary containment penetrations required to be closed during accident conditions are either:

1. Capable of being closed by an OPERABLE secondary containment automatic isolation system, or

2. Closed by at least one manual valve, blind flange, or deactivated automatic valve secured in its closed position.

b. All secondary containment hatches and blowout panels are closed and sealed.

c. The standby gas treatment system is in compliance with the requirements of Specification 3.6.5.3.

RESPONSIBILITIES

6.5.1.6 The POC shall be responsible for:

a. Review of (1) all proposed procedures required by Specification 6.8 and changes thereto, (2) all proposed programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety;

b. Review of all proposed tests and experiments that affect nuclear safety;

c. Review of all proposed changes to the Appendix A Technical Specifications;

d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety;

e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Director of Power Generation and to the Corporate Nuclear Safety Review Board;

f. Review of all REPORTABLE EVENTS;

g. Review of unit operations to detect potential hazards to nuclear safety;

h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Plant Manager or the Corporate Nuclear Safety Review Board;

i. Review of the Security Plan and implementing procedures and submittal of recommended changes to the Corporate Nuclear Safety Review Board; and

j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Corporate Nuclear Safety Review Board.

k. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Director of Power Generation and to the Corporate Nuclear Safety Review Board.

l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.

AUTHORITY

6.5.1.7 The POC shall:

a. Recommend in writing to the Plant Manager approval or disapproval of items considered under Specification 6.5.1.6a. through d. prior to their implementation.

REVIEW (Continued)

c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;

d. Proposed changes to Technical Specifications or this Operating License;

e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;

f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;

g. All REPORTABLE EVENTS;

h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and

i. Reports and meeting minutes of the POC.

j. Audit reports and summary reports of audits.

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the CNSRB. These audits shall encompass:

a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;

b. The performance, training and qualifications of the entire unit staff at least once per 12 months;

c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety, at least once per 6 months;

d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;

e. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel;

f. The Emergency Plan and implementing procedures at least once per 12 months per 10 CFR 50.54(t).

g. The Security Plan and implementing procedures at least once per 12 months.

AUDITS (Continued)

h. The fire protection equipment and program implementation, at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer(s) or an outside independent fire protection consultant. An outside independent fire protection consultant shall be utilized at least once every third year; and

i. Any other area of unit operation considered appropriate by the CNSRB or the Managing Director.

j. The radiological environmental monitoring program and the results thereof at least once per 12 months.

k. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.

l. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

m. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

RECORDS

6.5.2.9 Records of CNSRB activities shall be prepared, approved, and distributed as indicated below:

a. Minutes of each CNSRB meeting shall be prepared, approved, and forwarded to the Managing Director 14 days following each meeting.

b. Reports of reviews encompassed by Specification 6.5.2.7 above, shall be prepared, approved, and forwarded to the Managing Director within 14 days following completion of the review.

c. Audit reports encompassed by Specification 6.5.2.8 shall be forwarded to the Managing Director and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and

b. Each REPORTABLE EVENT shall be reviewed by the (POC), and the results of this review shall be submitted to the (CNSRB) and the Director of Power Generation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Regional Administrator of the Regional Office of the NRC unless otherwise noted.

STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an Operating License, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the unit.

6.9.1.2 The startup report shall address each of the tests identified in the Final Safety Analysis Report and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the startup report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial operation) supplementary reports shall be submitted at least every 3 months until all three events have been completed.

ANNUAL REPORTS*

6.9.1.4 Annual reports covering the activities of the unit as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

*A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

ANNUAL REPORTS (Continued)

6.9.1.5 Reports required on an annual basis shall include:

a. A tabulation on an annual basis of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrems/yr and their associated man-rem exposure according to work and job functions* (e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance [describe maintenance], waste processing, and refueling). The dose assignments to various duty functions may be estimated based on pocket dosimeter, thermoluminescent dosimeter (TLD), or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole-body dose received from external sources should be assigned to specific major work functions; and

b. Documentation of all challenges to main steam line safety/relief valves.

MONTHLY OPERATING REPORTS

6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Resource Management, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Administrator of the Regional Office of the NRC, no later than the 15th of each month following the calendar month covered by the report.

REPORTABLE OCCURRENCES

6.9.1.7 DELETED

PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP

6.9.1.8 DELETED

*This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

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THIRTY DAY WRITTEN REPORTS

6.9.1.9 DELETED

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION

AMENDMENT NO. 5 TO NPF-21

WPPSS NUCLEAR PROJECT NO. 2

DOCKET NO. 50-397

Introduction

By application dated January 20, 1984, the Washington Public Power Supply System (WPPSS or the licensee) requested an amendment to the WNP-2 Technical Specifications (TS). The proposed changes are either administrative in nature or make TS consistent with upgraded regulations that became effective January 1, 1984. Specifically Section 10 CFR 50.72 has been revised and a new Section 10 CFR 50.73 has been added.

Evaluation

The revised Section 50.72 modifies the immediate notification requirements for operating nuclear power reactors and Section 50.73 provides for a revised Licensee Event Report System. The proposed changes are in the "Definitions and Administrative Control" sections of the technical specifications. The definition "Reportable Occurrence" is replaced by a new term. "Reportable Event." Also under reportable event action, paragraph 6.6.1 will be changed to read, a. the Commission shall be notified and a report submitted pursuant to the requirements of §50.73 to 10 CFR Part 50, and b. each Reportable Event shall be reviewed by the Plant Operations Committee (POC), and the results of this corporate Nuclear Safety Review Board (CNSRB) and the Director of Power Generation. Technical Specifications Sections 6.9.1.7 (Reportable Occurrences). 6.9.1.8 (Prompt Notification) and 6.9.1.9 (Thirty day written reports) have beed deleted and replaced by the requirements of Sections 50.72 and 50.73 of Title 10 of the Code of Federal Regulations.

These administrative changes were requested by the NRC staff in Generic Letter No. 83-43, "Reporting Requirements of 10 CFR Part 50, Sections 50.72 and 50.73, and Standard Technical Specifications", dated December 19, 1983. The staff has reviewed licensee's response to the staff request and finds it acceptable.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has

previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (49 FR 21849) on May 23, 1984, and consulted with the state of Washington. No public comments were received, and the state of Washington did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: SEP 21 1984