

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS) MAM

ACCESSION NBR: 8312190066 , DOC. DATE: 83/12/09 NOTARIZED: NO DOCKET #
 FACIL: 50-397 WPPSS Nuclear Project, Unit 2, Washington Public Powe 05000397
 AUTH. NAME: SORENSEN, G.C. AUTHOR AFFILIATION: Washington Public Power Supply System
 RECIP. NAME: SCHWENCER, A. RECIPIENT AFFILIATION: Licensing Branch 2

SUBJECT: Forwards clarification of responses to NRC Question 423.041 re FSAR Chapter 14 subsections on load testing, verification & solid radwaste sys.

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 TITLE: Licensing Submittal; PSAR/FSAR Amdts & Related Correspondence

NOTES:

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	NRR/DSI/AEB 26	1 1	NRR/DSI/ASB	1 1
	NRR/DSI/CPB 10	1 1	NRR/DSI/CSB 09	1 1
	NRR/DSI/ICSB 16	1 1	NRR/DSI/METB 12	1 1
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	NRR/DSI/RSB 23	1 1	<u>REG FILE</u> 04	1 1
	RGNS	3 3	RM/DDAMI/MIB	1 0
EXTERNAL:	ACRS 41	6 6	BNL (AMDTS ONLY)	1 1
	DMB/DSS (AMDTS)	1 1	FEMA-REP DIV 39	1 1
	LPDR 03	1 1	NRC PDR 02	1 1
	NSIC 05	1 1	NTIS	1 1

1. The purpose of this document is to provide a comprehensive overview of the current status of the project. It is intended for use by all project team members and stakeholders. The information contained herein is confidential and should be handled accordingly.

2. The project has made significant progress since the last meeting. Key milestones have been met, and the team is on track to complete the project by the end of the quarter.

3. The following table provides a detailed breakdown of the project's progress and resource allocation.

Task ID	Task Name	Start Date	End Date	Progress (%)	Resources
1.1	Project Planning	2023-01-01	2023-01-31	100	John Doe
1.2	Requirement Gathering	2023-02-01	2023-02-28	100	Jane Smith
1.3	System Design	2023-03-01	2023-03-31	100	John Doe
1.4	Development	2023-04-01	2023-04-30	90	Jane Smith, John Doe
1.5	Testing	2023-05-01	2023-05-31	80	John Doe
1.6	Deployment	2023-06-01	2023-06-30	70	Jane Smith
1.7	Post-Deployment	2023-07-01	2023-07-31	60	John Doe
1.8	Documentation	2023-08-01	2023-08-31	50	Jane Smith
1.9	Final Review	2023-09-01	2023-09-30	40	John Doe
1.10	Project Closure	2023-10-01	2023-10-31	30	Jane Smith

Washington Public Power Supply System

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December 9, 1983
G02-83-1133

Docket No. 50-397

Director of Nuclear Reactor Regulation
Attention: Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Schwencer:

Subject: NUCLEAR PROJECT NO. 2
RESPONSE TO QUESTION 423.041

As informally requested by the NRC, the following information is provided by revised FSAR text, supplying clarification of responses to the subject NRC question.

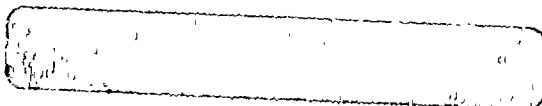
Should you have any further questions, please contact Mr. P. L. Powell, Manager, WNP-2 Licensing.

Very truly yours,

G. C. Sorensen, Manager
Regulatory Programs

SIS/tmh
Attachment

cc: R Auluck - NRC
WS Chin - BPA
AD Toth - NRC Site



Boo!
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WNP-2 STATUS THROUGH AMENDMENT 31

- 423.041
- a. FSAR Subsection 14.2.12.1.37 has not been modified to include load testing and monitoring of untested buses for voltage.
 - b.7 FSAR Subsection 14.2.12.1.48 has not been modified to include verification of 1) pumps NPSH and lack of vortexing, and 2) proper operation of basin siphon crossconnection.
 - b.9 FSAR Subsection 14.2.12.1.17 has not been modified to include the design requirement that the end product of the solid radwaste system must be free standing and contain no free liquid.
 - c. A test abstract for the control air system has not been included in FSAR Chapter 14.

*This system is not safety related,
therefore, it will not be included.*



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Signals for these tests shall be simulated from the actual initiating devices when this is practical.

4. Testing of the Diesel Generators will include the following:
 - a) Sequential loading of each diesel generator unit.
 - b) Maintenance of specified frequency and voltage during the loading sequence.
 - c) The diesel generator's capability to reject and restart their largest single load anytime after the design loading sequence is complete.
 - d) The diesel generator's capability to supply power to vital equipment during loss of station normal power conditions.

5. Electrical separation will be verified during testing by:
 - a) Verifying that operation of the Division/equipment being tested and the non actuation of de-energized buses/equipment does not affect the proper operation of the remaining buses/equipment.
 - b) Monitoring of the major distribution buses to ensure absence of voltage.

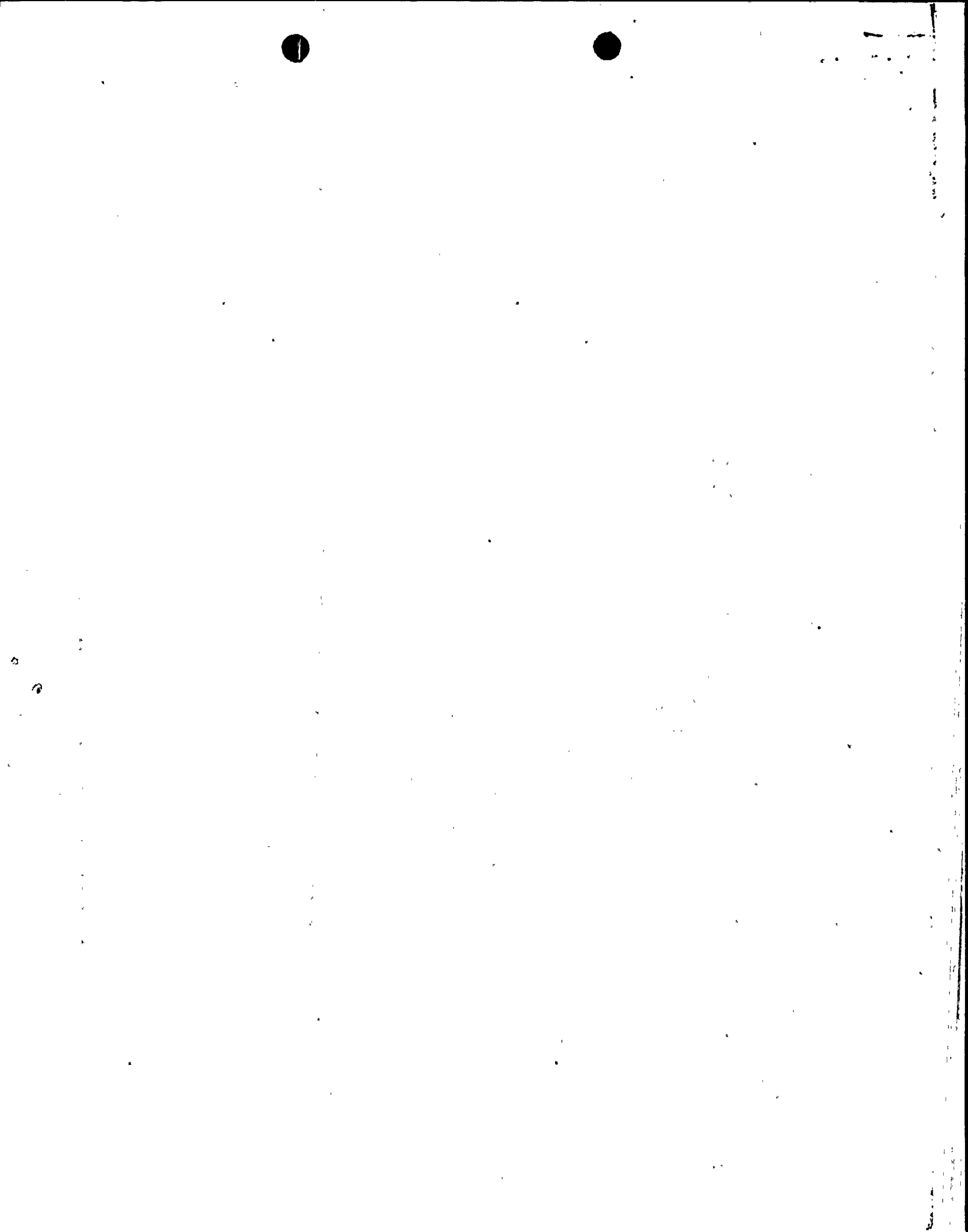
Response
to
a.

Main power transformers supplying power from the off site system cannot be full load tested, they are testing per this procedure to the design emergency load. All other in-plant power sources are load tested in their individual preoperational tests.

14.2.12.1.38 DELETED

14.2.12.1.39 DELETED

14.2.12.1.40 DELETED



c. General Test Methods and Acceptance Criteria

Standby Service Water System heat dissipation capabilities are not demonstrated during the preoperational test. Verification of this system is demonstrated by the proper integrated operation and performance of the following:

1. Pumps and related controls
2. Remote-operated valves and controls
3. Automatic-operated valves and control logic
4. Instrumentation
5. Annunciators
6. Standby Service Water System control logic response to a simulated loss of normal station power event.
7. Pumps NPSH adequate and no vortexing.
8. Proper operation of basin siphon crossconnection.

Response
to
b-7-

14.2.12.1.49 Plant Communications System Preoperational Test

a. Purpose

To demonstrate that the Plant Communications and Evacuation Alarm System will provide effective communication between various plant locations and to verify proper operation of the emergency evacuation alarm components and system.

b. Prerequisites

The System Lineup Tests have been completed and the TWG has reviewed and approved the procedure and the Test and Startup Manager has approved the initiation of testing.

c. General Test Methods and Acceptance Criteria

Proper operation of all the communication system components and the emergency evacuation alarm system and components will be demonstrated.



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c. General Test Methods and Acceptance Criteria

Testing will demonstrate that the pumps, tanks, controls and valves including automatic isolation, diversion and protective features and instrumentation and alarms will operate and function in accordance with design requirements.

Response
to
b.9.

Testing will also verify that the WNP-2 Process Control Program results in an acceptable waste form as required by 10CFR61. Simulated waste will be verified to form a free-standing monolithic solid with no free liquid prior to implementation of the solidification process on radioactive waste. Liners containing solidified waste will be inspected prior to shipment to the disposal site to verify compliance with 10CFR61 requirements.

14.2.12.1.18 Reactor Protection System Preoperational Test

a. Purpose

To verify the proper operation of the Reactor Protection System (RPS), including sensor logic and their respective scram relays, scram reset time delay, the annunciators, and motor-generator set power supply.

b. Prerequisites

The System Lineup Tests have been completed and the TWG has reviewed and approved the procedure and the Test and Startup Manager has approved the initiation of testing.

c. General Test Methods and Acceptance Criteria

Verification of the RPS capability is demonstrated by the proper integrated operation of the following:

1. Motor-generator set performance
2. Sensor logic and scram relay logic
3. Scram reset time delay



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